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Docket Nos. 50-277
and 50-278

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TJCarter
EP Licensing Assistant
EP Project Manager

Philadelphia Electric Company
ATTN: Mr. Edward G. Bauer, Jr., Esquire
Vice President and General Counsel
2301 Market Street
Philadelphia, Pennsylvania 19101

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 13 and 11 to Facility Operating Licenses Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Units 2 and 3. The amendments also incorporate Changes Nos. 14 and 11 in the Technical Specifications in accordance with your application dated September 19, 1975.

The amendments modify the Technical Specifications to accommodate a plant modification to replace existing reactor pressure sensing switches with analog loops, each consisting of a pressure transmitter, a pressure indicator and a trip unit.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

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George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

1. Amendments Nos. 13 and 11
2. Safety Evaluation
3. Federal Register Notice

cc w/encs:
See next page

[Handwritten signatures]

OFFICE →	RL:ORB-3	RL:ORB-3	OELD	RL:ORB-3		
x7872 SURNAME →	CParrish	TVerdery:esp	S. SCHINKI	GLear		
DATE →	10/ /75	10/ /75	10/ /75	10/ /75		

For further details with respect to this action, see (1) the application for amendments dated September 19, 1975, (2) Amendments Nos. 13 and 11 to Licenses Nos. DPR-44 and DPR-56, with Changes Nos. 14 and 11, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Martin Memorial Library, 159 E. Market Street, York, Pennsylvania 17401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 30th day of October, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

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George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

OFFICE →	RL:ORB-3	RL:ORB-3	OELD <i>SM</i>	RL:ORB-3		
<i>x7872</i> SURNAME →	CParrish <i>cp</i>	EVERdery:esp	<i>S. SOL</i>	GLear <i>GL</i>		
DATE →	10/22/75	10/27/75	10/29/75	10/30/75		

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 44 and 56 to Facility Operating Licenses Nos. DPR-44 and DPR-56, respectively, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, which revised Technical Specifications for operation of the Peach Bottom Atomic Power Station, Units 2 and 3, located in York County, Pennsylvania. The amendments are effective as of one week following the completion of the modification to the reactor pressure sensing devices.

The amendments modify the Technical Specifications to accommodate a plant modification to replace existing reactor pressure sensing switches with analog loops, each consisting of a pressure transmitter, a pressure indicator and a trip unit.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of these amendments is not required since the amendments do

not involve a significant hazards consideration.

OFFICE →

SURNAME →

DATE →

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. DPR-56

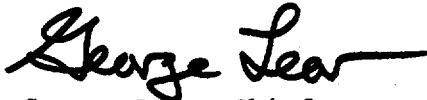
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company (the licensees) dated September 19, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.(C).2 of Facility License No. DPR-56 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 11."

3. This license amendment is effective as of one week following the completion of the modification to the reactor pressure sensing devices.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 11 to the
Technical Specifications

Date of Issuance: OCT 30 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 11
CHANGE NO. 11 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-56
DOCKET NO. 50-278

Replace page 41, page 43, and page 44 with the attached revised pages. No change has been made on page 42.

TABLE 4.1.1

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT FUNCTIONAL TESTS
 MINIMUM FUNCTIONAL TEST FREQUENCIES FOR SAFETY INSTRUMENT AND CONTROL CIRCUITS

	Group (2)	Functional Test	Minimum Frequency (3)
Mode Switch in Shutdown	A	Place Mode Switch in Shut-down.	Each refueling outage.
Manual Scram	A	Trip Channel and Alarm	Every 3 months.
RPS Channel Test Switch	A	Trip Channel and Alarm	Every refueling outage or after channel maintenance.
IRM			
High Flux	C	Trip Channel and Alarm (4)	One per week during refueling or startup and before each startup.
Inoperative	C	Trip Channel and Alarm (4)	Once per week during refueling or startup and before each start
APRM			
High Flux	B	Trip Output Relays (4)	Once/week.
Inoperative	B	Trip Output Relays (4)	Once/week.
Downscale	B	Trip Output Relays (4)	Once/week.
Flow Bias	B	Calibrate Flow Bias Signal (4)	Once/month (1).
High Flux in Startup or Refuel	C	Trip Output Relays (4)	Once per week during refueling or startup and before each startup.
High Reactor Pressure (6)	B	Trip Channel and Alarm (4)	Every 1 month (1).
High Drywell Pressure	A	Trip Channel and Alarm	Every 1 month (1).
Reactor Low Water Level (5)	A	Trip Channel and Alarm	Every 1 month (1).

TABLE 4.1.1 (cont'd)

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT FUNCTIONAL TESTS
 MINIMUM FUNCTIONAL TEST FREQUENCIES FOR SAFETY INSTRUMENT AND CONTROL CIRCUITS

	Group (2)	Functional Test	Minimum Frequency (3)
High Water Level in Scram Discharge Tank	A	Trip Channel and Alarm	Every 3 months.
Turbine Condenser Low Vacuum	A	Trip Channel and Alarm	Every 1 month (1).
Main Steam Line High Radiation	B	Trip Channel and Alarm (4)	Once/week.
Main Steam Line Isolation Valve Closure	A	Trip Channel and Alarm	Every 1 month (1).
Turbine Control Valve EHC Oil Pressure	A	Trip Channel and Alarm	Every 1 month
Turbine First Stage Pressure Permissive	A	Trip Channel and Alarm	Every 3 months (1)
Turbine Stop Valve Closure	A	Trip Channel and Alarm	Every 1 month (1)
Reactor Pressure Permissive	A	Trip Channel and Alarm	Every 3 months.

NOTES FOR TABLE 4.1.1

1. Initially once every month. The compilation of instrument failure rate data may include data obtained from other boiling water reactors for which the same design instrument operates in an environment similar to that of PBAPS. The failure rate data must be reviewed and approved by the NRC prior to any change in the once-a-month frequency.
2. A description of the three groups is included in the Bases of this Specification.
3. Functional tests are not required on the part of the system that is not required to be operable or are tripped.

If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.

4. This instrumentation is exempted from the instrument channel test definition. This instrument channel functional test will consist of injecting a simulated electrical signal into the measurement channels.
5. The water level in the reactor vessel will be perturbed and the corresponding level indicator changes will be monitored. This perturbation test will be performed every month after completion of the functional test program.
6. These channels consist of analog pressure transmitters, pressure indicators and electronic trip units. Instrument checks shall be performed once per day.

TABLE 4.1 2

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT CALIBRATION
 MINIMUM CALIBRATION FREQUENCIES FOR REACTOR PROTECTION INSTRUMENT CHANNELS

Instrument Channel	Group (1)	Calibration (4)	Minimum Frequency (2)
IRM High Flux	C	Comparison to APRM on Controlled Shutdowns	Maximum frequency once per week.
APRM High Flux	B	Heat Balance	Twice per week.
Output Signal	B	With Standard Pressure Source	Every refueling outage.
Flow Bias Signal			
LPRM Signal	B	TIP System Traverse	Every 6 weeks.
High Reactor Pressure	B	Standard Pressure Source	Once per operating cycle. 11
High Drywell Pressure	A	Standard Pressure Source	Every 3 months.
Reactor Low Water Level	A	Pressure Standard	Every 3 months.
High Water Level in Scram Discharge Volume	A	Water Column	Every refueling outage.
Turbine Condenser Low Vacuum	A	Standard Vacuum Source	Every 3 months.
Main Steam Line Isolation Valve Closure	A	Note (5)	Note (5).
Main Steam Line High Radiation	B	Standard Current Source (3)	Every 3 months.
Turbine First Stage Pressure Permissive	A	Standard Pressure Source	Every 6 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 13 AND 11 TO LICENSES NOS. DPR-44 AND DPR-56

(CHANGES NOS. 14 AND 11 TO THE TECHNICAL SPECIFICATIONS)

PHILADELPHIA ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 AND 3

DOCKETS NOS. 50-277 AND 50-278

Introduction

On September 19, 1975, the Philadelphia Electric Company (PECo) submitted an application to amend the licenses of Peach Bottom Units 2 and 3. The proposed amendments will modify the Peach Bottom Technical Specifications by revising Tables 4.1.1 and 4.1.2 and by adding Note 6 to the "Notes for Table 4.1.1" to accommodate a plant modification to replace existing reactor pressure sensing switches with analog loops, each consisting of a pressure transmitter, a pressure indicator and a trip unit.

Discussion

The presently installed pressure switches sensing reactor pressure have exhibited high sensitivity to light mechanical shock. The reactor pressure at normal full load conditions is 1000 psig; the High Reactor Pressure Trip (Scram) set point is 1055 psig. As a result, the switches are in the "almost tripped" condition during normal operation and only slight mechanical shock is required to cause a spurious reactor trip. In order to reduce the probability of spurious signals initiating reactor scram, the proposed modifications will replace the existing pressure switches with analog loops, each consisting of a pressure transmitter, pressure indicator, and trip unit which are not sensitive to mechanical shock.

Evaluation

Since the modification involves removing one device and substituting other devices to perform the same function, the evaluation can be limited to demonstrating that the replacement devices are as reliable as the existing devices. The modification involves no changes in design

basis, protective function, redundancy trip set point, logic, or the Commission's requirements which were applicable to the original design. The design reliability goal of the existing device is 0.993. Our analysis of the replacement instrument channels verifies that the product of the individual reliabilities of the transmitter and the trip unit yields a channel reliability greater than 0.993, which satisfies the design reliability goal of the existing instrumentation.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: OCT 30 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

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The amendments modify the Technical Specifications to accommodate a plant modification to replace existing reactor pressure sensing switches with analog loops, each consisting of a pressure transmitter, a pressure indicator and a trip unit.

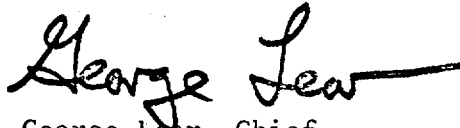
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendments dated September 19, 1975, (2) Amendments Nos. 13 and 11 to Licenses Nos. DPR-44 and DPR-56, with Changes Nos. 14 and 11, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Martin Memorial Library, 159 E. Market Street, York, Pennsylvania 17401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 30th day of October, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing