

August 17, 1993

Mr. Jerry W. Yelverton  
Vice President, Operations ANO  
Entergy Operations, Inc.  
Route 3 Box 137G  
Russellville, Arkansas 72801

Dear Mr. Yelverton:

SUBJECT: CORRECTION TO AMENDMENT NO. 149 TO FACILITY OPERATING LICENSE NO.  
NPF-6 - ARKANSAS NUCLEAR ONE, UNIT NO. 2 (TAC NO. M86420)

On July 26, 1993, the Commission issued Amendment No. 149 to Facility Operating License No. NPF-6 to Arkansas Nuclear One, Unit No. 2 (ANO-2). The amendment corrected typographical errors that were introduced in the original Technical Specifications (TSs) and in subsequent amendments.

As a result of discussions with your staff, an error was identified near the top of TS Page 3/4 1-1. It says T-avg is less than or equal to 200 degrees F. It should say T-avg is greater than 200 degrees F. The intent of Amendment No. 149 was not to change this portion of TS Page 3/4 1-1. Based on discussions with your staff, it appears that this error was inadvertently made in your application and subsequently not identified by the NRC staff. The NRC staff was concentrating its review on the areas of pages where requested changes were identified with vertical change bars, and no change was requested for this portion of TS Page 3/4 1-1.

Accordingly, a revised TS Page 3/4 1-1 is being issued. The corresponding overleaf page is also provided to maintain document completeness. Please accept our apology for any inconvenience this error may have caused you.

Sincerely,

ORIGINAL SIGNED BY:  
Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

Enclosure:  
TS Page 3/4 1-1

cc w/enclosure:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 17, 1993

Docket No. 50-368

Mr. Jerry W. Yelverton  
Vice President, Operations ANO  
Entergy Operations, Inc.  
Route 3 Box 137G  
Russellville, Arkansas 72801

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Sincerely,

A handwritten signature in cursive script that reads "Thomas W. Alexion".

Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

Enclosure:  
TS Page 3/4 1-1

cc w/enclosure:  
See next page

Mr. Jerry W. Yelverton  
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Arkansas Nuclear One, Unit 2

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County Judge of Pope County  
Pope County Courthouse  
Russellville, Arkansas 72801

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and Emergency Management  
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Little Rock, Arkansas 72205-3867

### 3/4.1 REACTIVITY CONTROL SYSTEMS

#### 3/4.1.1 BORATION CONTROL

SHUTDOWN MARGIN T<sub>avg</sub> > 200°F

#### LIMITING CONDITION FOR OPERATION

3.1.1.1 The SHUTDOWN MARGIN shall be  $\geq 5.5\% \Delta k/k$ .

APPLICABILITY: MODES 1, 2\*, 3 and 4.

ACTION:

With the SHUTDOWN MARGIN  $< 5.5\% \Delta k/k$ , immediately initiate and continue boration at  $\geq 40$  gpm of 2500 ppm boric acid solution or equivalent until the required SHUTDOWN MARGIN is restored.

#### SURVEILLANCE REQUIREMENTS

4.1.1.1.1 The SHUTDOWN MARGIN shall be determined to be  $\geq 5.5\% \Delta k/k$ .

- a. Within one hour after detection of an inoperable CEA(s) and at least once per 12 hours thereafter while the CEA(s) is inoperable. If the inoperable CEA is immovable or untrippable, the above required SHUTDOWN MARGIN shall be increased by an amount at least equal to the withdrawn worth of the immovable or untrippable CEA(s).
- b. When in MODES 1 or 2<sup>#</sup>, at least once per 12 hours by verifying that CEA group withdrawal is within the Transient Insertion Limits of Specification 3.1.3.6.
- c. When in MODE 2<sup>##</sup>, within 4 hours prior to achieving reactor criticality by verifying that the predicted critical CEA position is within the limits of Specification 3.1.3.6.
- d. Prior to initial operation above 5% RATED THERMAL POWER after each fuel loading, by consideration of the factors of (e) below, with the CEA groups at the Transient Insertion Limits of Specification 3.1.3.6.

\* See Special Test Exception 3.10.1.

# With  $K_{eff} \geq 1.0$ .

## With  $K_{eff} < 1.0$ .

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## REACTIVITY CONTROL SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

e. When in MODES 3 or 4, at least once per 24 hours by consideration of at least the following factors:

1. Reactor coolant system boron concentration,
2. CEA position,
3. Reactor coolant system average temperature,
4. Fuel burnup based on gross thermal energy generation,
5. Xenon concentration, and
6. Samarium concentration.

4.1.1.1.2 The overall core reactivity balance shall be compared to predicted values to demonstrate agreement within  $\pm 1.0\% \Delta k/k$  at least once per 31 Effective Full Power Days (EFPD). This comparison shall consider at least those factors stated in Specification 4.1.1.1.1.e, above. The predicted reactivity values shall be adjusted (normalized) to correspond to the actual core conditions prior to exceeding a fuel burnup of 60 Effective Full Power Days after each fuel loading.