

Short Notice

AVR Operational Experience, Overview

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Executive Statements Summary

1 History

1956 Engagement of AVR in HTGR from the very beginning

1961 Begin plant construction

1964 First core ordered (UCC)

1966 First core delivered, First criticality

2 Overall achievements

- 2.1 The plant has been operated for 21 years (1967 1988). Given the experimental and first-of-its-kind character of the plant, the achieved time availability with a record value of 92 % in 1976 is quite remarkable (Fig. 1).
- 2.2 Starting in 1974, operation at 950 °C had a share of nearly 30 %.
- 2.3 The personnel dose uptake records show significant improvements in the course of the years due to better components and procedures (lessons learned, Fig. 2).
- 2.4 Radioactivity release to the atmosphere remained well below licenced levels (Fig. 3).

3 Fuel

- 3.1 AVR was the indispensable mass test facility for all development steps of pebble fuel (Table 1).
- 3.2 Pebbles with oxide fuel, and no matter if HEU or LEU, BISO or TRISO coatings, showed at max. fuel temperatures of > 1300 °C and burnups of partly > 20 % fima excellent fission product retention (Table 2).

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3.3 In the modern UO₂, TRISO pebble the fission product release is practically exclusively determined by the little as-manufactured free uranium outside of the coatings.

4 Pebble cycling

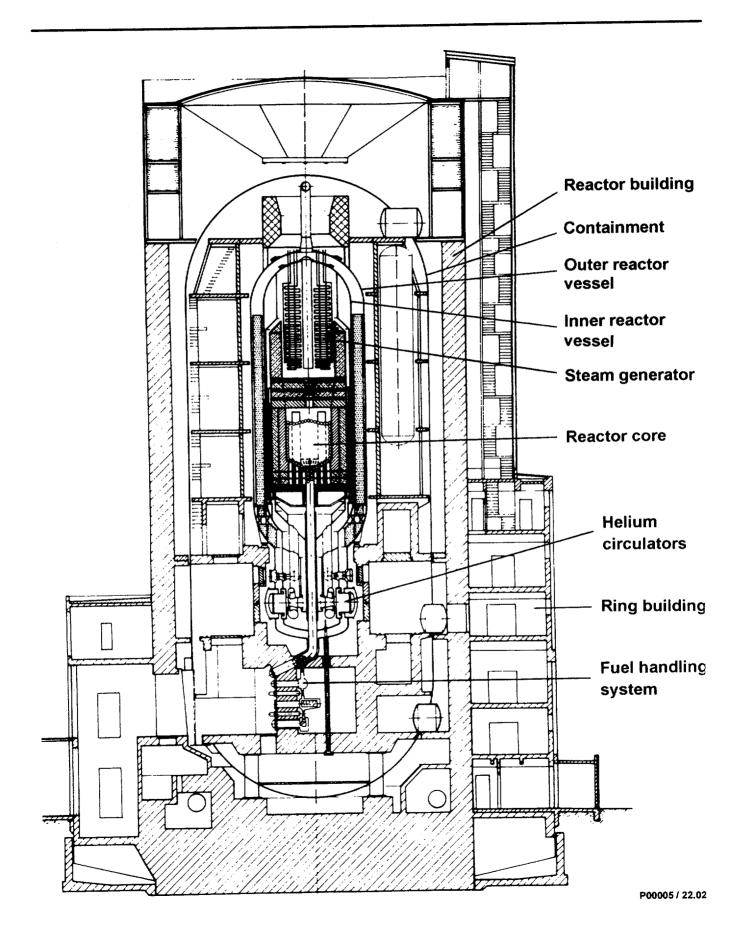
During plant operation, 2.4 million pebbles were cycled. The cycling system as a first of its kind needed frequent maintenance but worked well after various improvements, and accounted for only 3 % of the generator non-availability (Fig. 4).

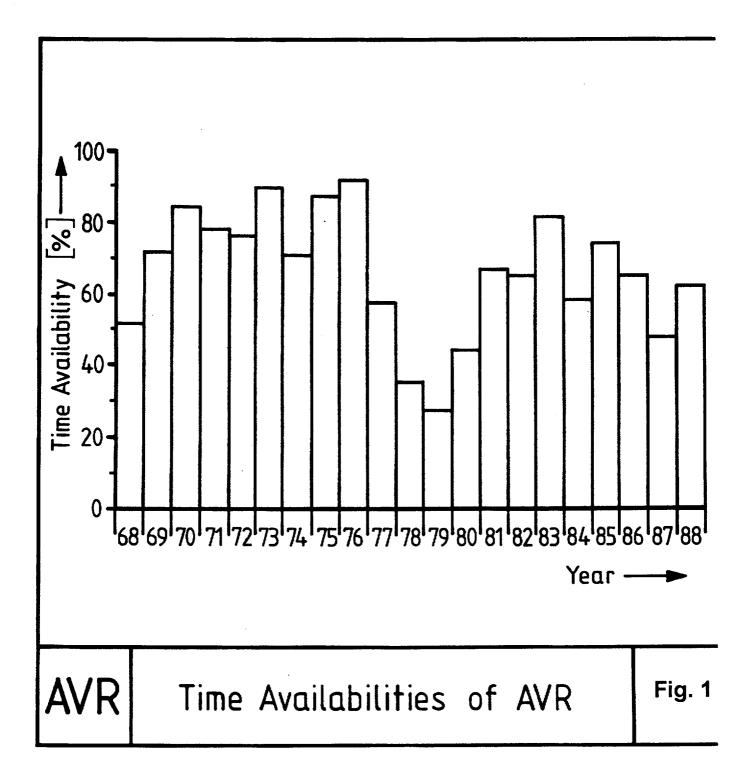
5 Water ingress

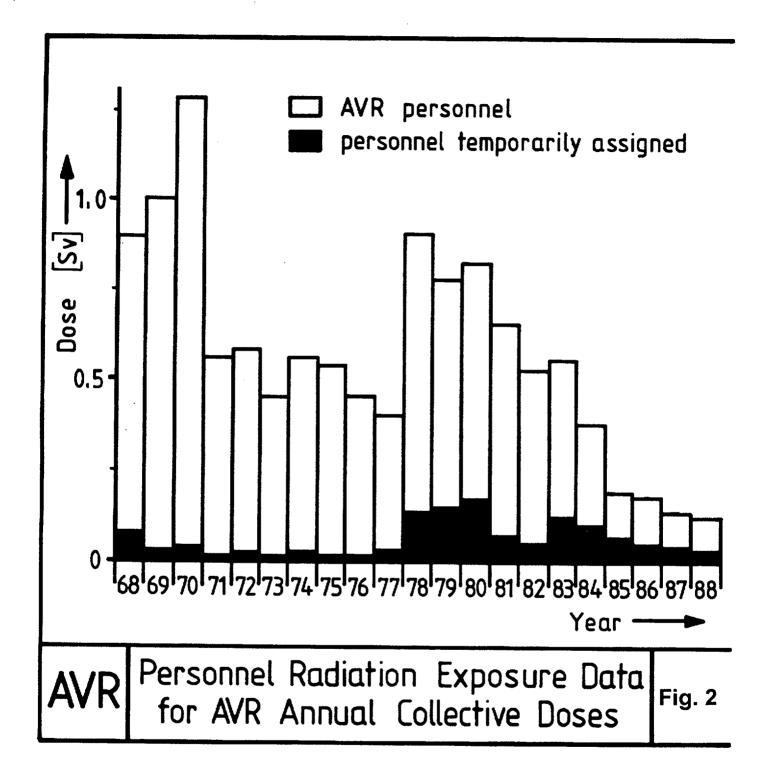
A water ingress in 1978 due to a steam generator leak did in the end not affect continued plant operation, and there was no need to replace fuel.

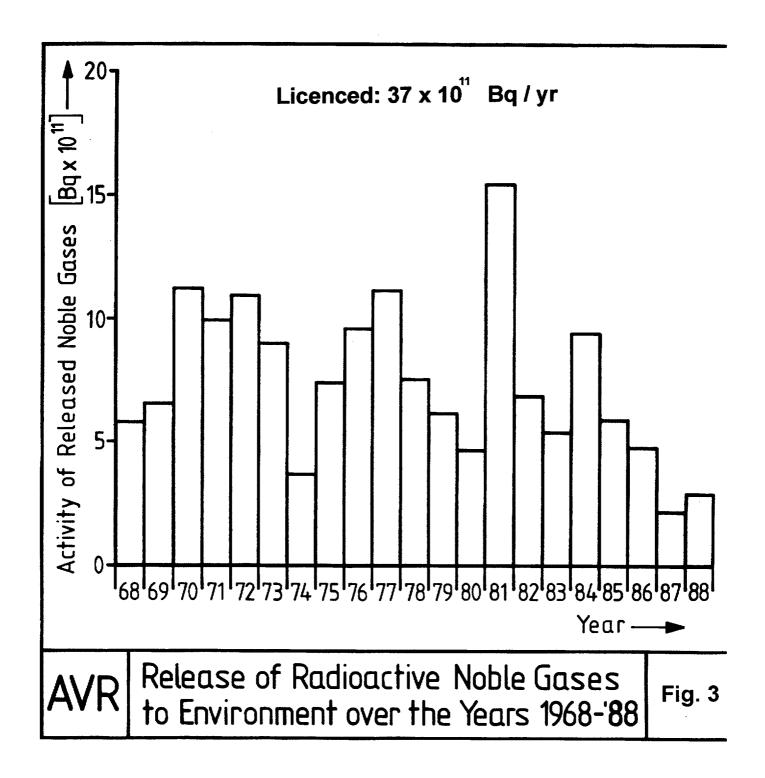
6 Safety demonstrations

- 6.1 Experiments simultaneously simulating the loss of forced cooling and stuck rods resulted in a simple shut-down and, with the rods kept withdrawn, in a recriticality with the reactor stabilizing at a very low core power (Fig. 5).
- 6.2 A complete loss-of-coolant accident was realistically simulated with the AVR at depressurized conditions (Fig. 6). A maximum temp. of 1090 °C occurred in the core center in less than 10 hrs after accident initiation.









AVR	Pebble fuel variety	Table 1
	AVR as mass test bed	
Pebble struct	ture	
 Shell type 	;	37,700
Pressed ty	pe	253,000
Fuel design HEU		
• (U/T	h) C ₂ with 5 g Th	87,600
• (U/T	h) O ₂ with 5 or 10 g Th	129,400
• Feed /	Breed, UO ₂ , UC ₂ , UCO, ThO ₂	20,300
LEU		
• UO ₂ , d	lifferent enrichments	53,400
Coating desi	gn	

	BISO type	202,900
-	TRISO type	74,300
•	Feed / Breed (TRISO / BISO mixed)	13,500

Stationary operation at 950 °C

Activity concentrations in Bq/m³ and descending order

 Total fission gases 	4.6 E 08
 Tritium 	3.7 E 07
• C 14	1.9 E 07
• J 131	5.2 E 02
• Cs 137	3.0 E 02
■ Sr 90	2.0 E 02
• Ag 110m	4.9 E 01
• Co 60	1.0 E 01

FUEL HANDLING SYSTEM

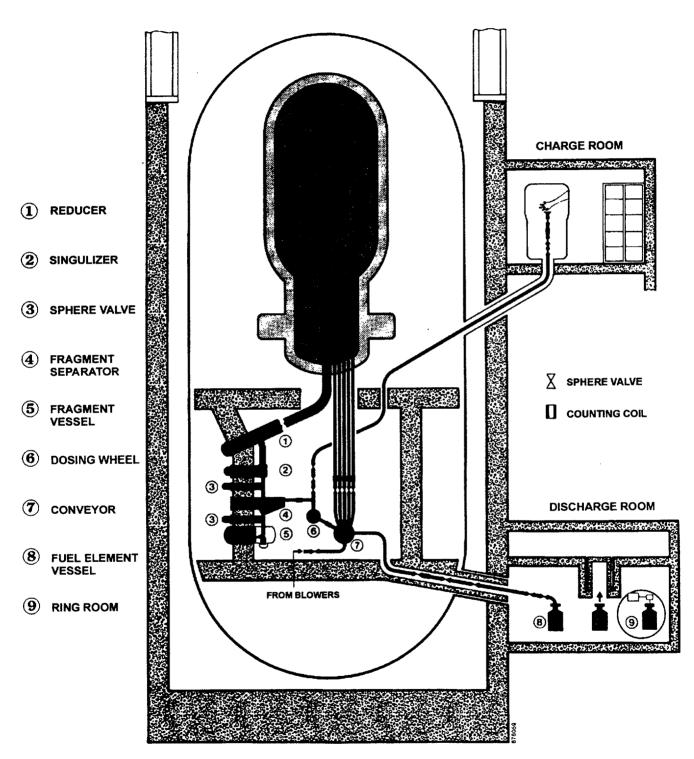
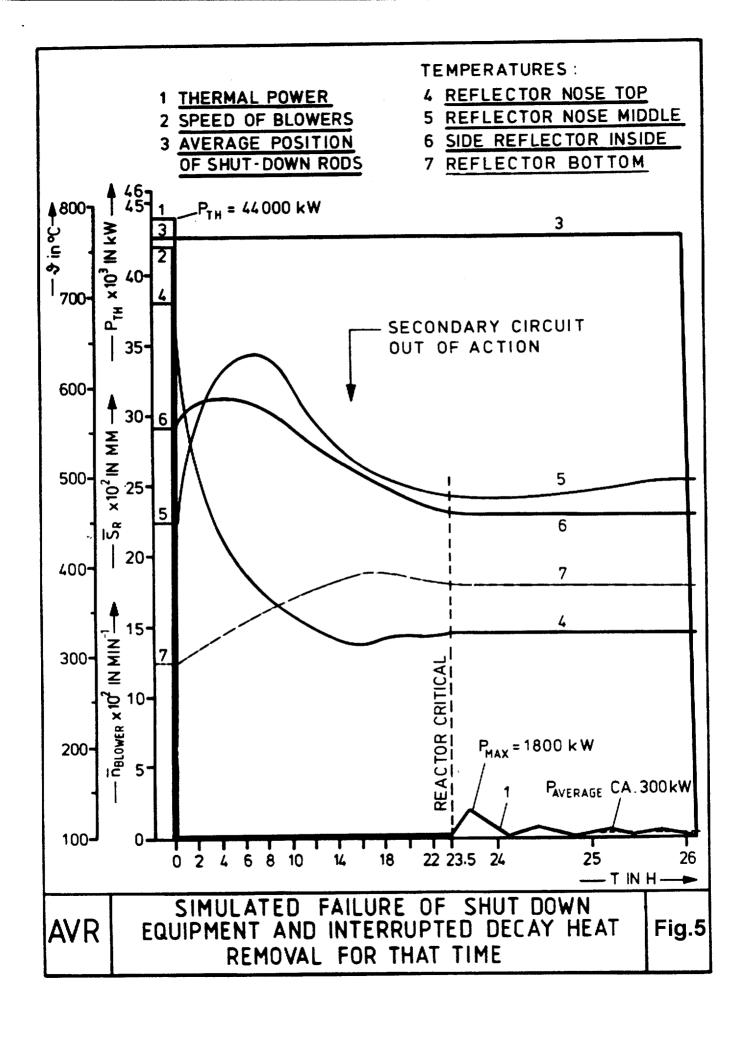
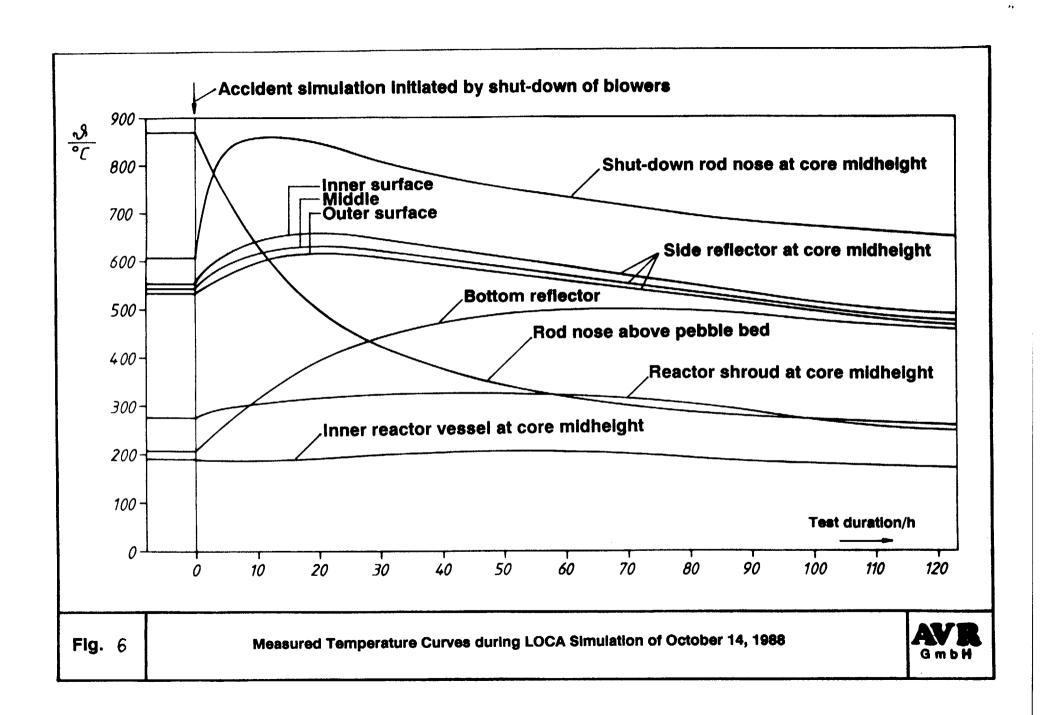


Fig. 4





Plant history

1956	Engagement of AVR in HTGR, from the very beginning
1961	Begin plant construction
1964	First core ordered from Union Carbide
1966	First criticality
■ 1967 to 1988	In operation as Experimental and Pebble Test Reactor

Electricity production 1,670 GWh with about 300,000 pebbles.
 That means an average production of 6 MWh from each pebble.