

CP
SI

DCS MS-016

OCT 19 1982

Docket No. 50-368

DISTRIBUTION:

- ✓ Docket File
- NRC PDR
- L PDR
- NSIC
- SECY
- ORB#3 Rdg
- DEisenhut
- RDiggs
- PMKreutzer (3)
- JAStevens
- RAClark
- OELD
- I&E (2) LJHarmon
- TBarnhardt (4)
- LSchneider
- DBrinkman
- ACRS (10)
- CMiles
- JHeltemes
- Gray File (+4)
- ASLAB

Mr. William Cavanaugh, III
 Senior Vice President, Energy
 Supply
 Arkansas Power & Light Company
 P. O. Box 551
 Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendment No. 36 to Facility Operating License No. NPF-6 for the Arkansas Power & Light Company for the Arkansas Nuclear One, Unit 2 plant. The amendment consists of changes to the Technical Specifications in response to your application dated August 29, 1980 as supplemented on May 6, 1981 on the pressurizer safety valve surveillance requirements.

During our review of your proposed amendments we found that certain modifications were necessary to meet our requirements. Your staff agreed to these modifications and they have been incorporated in this amendment.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Janice A. Stevens, Project Manager
 Operating Reactors Branch #3
 Division of Licensing

Enclosures:

1. Amendment No. 36 to NPF-6
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page

8211050169 821019
 PDR ADOCK 05000368
 PDR

F.R. NOTICE
 +
 AMENDMENT

OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:DL	OELD		
SURNAME	PMKreutzer	JAStevens/pn	RAClark	GCLainas	M. KARIN		
DATE	10/5/82	10/8/82	10/12/82	10/12/82	10/17/82		

Arkansas Power & Light Company

cc:

**Mr. John Marshall
Manager, Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203**

**Mr. James P. O'Hanlon
General Manager
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801**

**Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220
7910 Woodmont Avenue
Bethesda, Maryland 20814**

**Nicholas S. Reynolds, Esq.
c/o DeBevoise & Liberman
1200 Seventeenth Street, N.W.
Washington, D. C. 20036**

**Arkansas Polytechnic College
Russellville, Arkansas 72801**

**Mr. Charles B. Brinkman
Manager - Washington Nuclear
Operations
C-E Power Systems
4853 Cordell Avenue, Suite A-1
Bethesda, Maryland 20014**

**Regional Administrator
Nuclear Regulatory Commission, Region IV
Office of Executive Director for Operations
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011**

**Mr. W. Johnson
U.S. NRC
P. O. Box 2090
Russellville, Arkansas 72801**

**U.S. Environmental Protection Agency
Region VI Office
ATTN: Regional Radiation
Representative
1201 Elm Street
Dallas, Texas 75270**

**S. L. Smith, Operations Officer
Arkansas Nuclear Planning &
Response Program
P. O. Box 1749
Russellville, Arkansas 72801**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated August 29, 1980 as supplemented May 6, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8211050173 821019
PDR ADOCK 05000368
P PDR

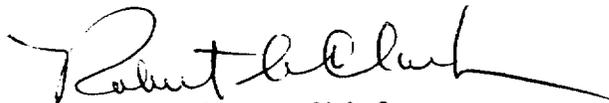
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 36, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 30 days of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: October 19, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is provided to maintain document completeness.

Page

3/4 4-4

REACTOR COOLANT SYSTEM

SAFETY VALVES - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.2 A minimum of one pressurizer code safety valve shall be OPERABLE with a lift setting of 2500 PSIA \pm 1%*.

APPLICABILITY: MODES 4 and 5.

ACTION:

With no pressurizer code safety valve OPERABLE, immediately suspend all operations involving positive reactivity changes and place an OPERABLE shutdown cooling loop into operation.

SURVEILLANCE REQUIREMENTS

4.4.2 No additional Surveillance Requirements other than those required by Specification 4.0.5.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

REACTOR COOLANT SYSTEM

SAFETY VALVES - OPERATING

LIMITING CONDITION FOR OPERATION

3.4.3 All pressurizer code safety valves shall be OPERABLE with a lift setting of $2500 \pm 1\%*$.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With one pressurizer code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in HOT SHUTDOWN within 12 hours.
- b. The provisions of Specification 3.0.4 may be suspended for one valve at a time for up to 18 hours for entry into and during operation in MODE 3 for the purpose of setting the pressurizer code safety valves under ambient (hot) conditions provided a preliminary cold setting was made prior to heatup.

SURVEILLANCE REQUIREMENTS

4.4.3 No additional Surveillance Requirements other than those required by Specification 4.0.5.

* The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-6

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT 2

DOCKET NO. 50-368

Introduction

The licensee proposed by letter dated August 29, 1980 to change the Technical Specifications (TS) for ANO-2 relating to testing and adjusting the pressurizer safety valve set point.

Pursuant to the surveillance requirements imposed upon these valves by TS 4.0.5, periodic testing of the valves' opening setpoint is required in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. The current T.S.LCO 3.4.3 requires the valves to be operable for Modes 1, 2 and 3. The current TS 3.4.3 Action Statement 3.4.3.b also provides for suspension of the LCO for up to 12 hours for entry into and operations in MODE 3 to test and adjust the valves. This verification of adjustment of the valve's setpoint at hot ambient MODE 3 conditions is performed subsequent to a preliminary setting made under cold conditions prior to heatup.

The licensee's proposed TS change would increase the 12 hours allowed for testing and adjustment up to 24 hours. The basis for the request is that 12 hours does not allow adequate time in all cases for the testing/adjusting activities to be completed, particularly if problems are encountered with test equipment or procedures.

The staff requested additional information by letter dated April 10, 1981 to which the licensee responded by letter dated May 6, 1981. The response identified six instances of testing the valves. Five of these tests had been completed within the specified twelve hour period while one required sixteen hours. The licensee stated that only one of the two pressurizer safety valves would be inoperable at any one time for testing and adjusting purposes. The licensee also stated that they are likely to minimize, to the degree practical, the time during which these valves are in a test status since the testing and adjustment activities are generally on the critical path for return of the plant to power operations.

In consideration of the importance of properly adjusted safety valves to the safe operation of the plant, the staff feels that the allowance of additional time to provide a margin for testing personnel to complete the testing activities, including resolution of some limited problems with test equipment and procedures, is appropriate. Based on the information provided, the staff concludes that the provision of an additional six hours for completion of these tests is sufficient to provide adequate

8211050175 821019
PDR ADOCK 05000368
PDR

margin to ensure the accuracy and reliability of the testing and adjustment of the valve's setpoint. Accordingly, we conclude that an extension of the time allowed for the MODE 3 setpoint adjustment from 12 to 18 hours is acceptable. This represents a modification of the licensee's proposed Technical Specification of 24 hours. This modification has been discussed with and concurred in by the licensee.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 19, 1982

Principal Contributors:

H. Shaw
R. Martin

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-368ARKANSAS POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 36 to Facility Operating License No. NPF-6 issued to Arkansas Power & Light Company (the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Unit 2 (the facility), located in Pope County, Arkansas. The amendment is effective within 30 days of its date of issuance.

The amendment modifies the ANO-2 Appendix A Technical Specifications dealing with the surveillance requirements for the pressurizer safety valves.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 29, 1980 as supplemented May 6, 1981, (2) Amendment No. 36 to Facility Operating License No. NPF-6, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room at 1717 H Street, N.W., Washington, D.C. 20555 and at the Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of October, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing