

NOV 5 1982

DCS MS-016

Docket No. 50-368

DISTRIBUTION:

- ✓ Docket File
- NRC PDR
- L PDR
- NSIC
- ORB#3 Rdg T Barnhart-4
- DEisenhut
- OELD
- ACRS-10
- SECY
- LHarmon-2
- RACLark
- PMKreutzer-3
- JStevens
- OPA-CMiles
- RDiggs
- ASLAB
- Gray File +4

Mr. William Cavanaugh, III  
 Senior Vice President, Energy  
 Supply  
 Arkansas Power & Light Company  
 P. O. Box 551  
 Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. NPF-6 for the Arkansas Nuclear One, Unit 2 plant. The amendment consists of changes to the Technical Specifications in response to your application dated July 8, 1982, as supplemented October 6, 1982.

The amendment restricts the movement of the full length and part length control element assembly (CEA) groups and allows for a special test exception for the part length CEA group. The analytical complexity of the Core Protection Calculator System software validation for future cycles is eased due to the restricted movement of the full length and part length CEA groups. The part length CEA insertion limit test exception is required for performance of startup physics test measurements of power distributions.

Copies of the Safety Evaluation and the Notice of Issuance are enclosed.

Sincerely,

Original signed by

Janice A. Stevens, Project Manager  
 Operating Reactors Branch #3  
 Division of Licensing

Enclosures:

1. Amendment No. 37 to NPF-6
2. Safety Evaluation
3. Notice of Issuance

cc: See next page

F.R. NOTICE  
 &  
 AMENDMENT

8211180015 821105  
 PDR ADOCK 05000368  
 P PDR

OFFICE	ORB#3:DL PMKreutzer	ORB#3:DL JStevens/pn	ORB#3:DL RACLark	AD:DR:DL GCLainas	OELD N. HARMON		
SURNAME							
DATE	11/3/82	11/3/82	11/3/82	11/3/82	11/3/82		

Arkansas Power & Light Company

cc:

Mr. John Marshall  
Manager, Licensing  
Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

Mr. James P. O'Hanlon  
General Manager  
Arkansas Nuclear One  
P. O. Box 608  
Russellville, Arkansas 72801

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 220  
7910 Woodmont Avenue  
Bethesda, Maryland 20814

Nicholas S. Reynolds, Esq.  
c/o DeBevoise & Liberman  
1200 Seventeenth Street, N.W.  
Washington, D. C. 20036

Arkansas Polytechnic College  
Russellville, Arkansas 72801

Mr. Charles B. Brinkman  
Manager - Washington Nuclear  
Operations  
C-E Power Systems  
4853 Cordell Avenue, Suite A-1  
Bethesda, Maryland 20014

Regional Administrator  
Nuclear Regulatory Commission, Region IV  
Office of Executive Director for Operations  
611 Ryan Plaza Drive, Suite 1000  
Arlington, Texas 76011

Mr. W. Johnson  
U.S. NRC  
P. O. Box 2090  
Russellville, Arkansas 72801

U.S. Environmental Protection Agency  
Region VI Office  
ATTN: Regional Radiation  
Representative  
1201 Elm Street  
Dallas, Texas 75270

S. L. Smith, Operations Officer  
Arkansas Nuclear Planning &  
Response Program  
P. O. Box 1749  
Russellville, Arkansas 72801



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DISTRIBUTION:  
Docket File  
ORB#3 Rdg  
PMKreutzer

Docket No. 50-368

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: ARKANSAS POWER & LIGHT COMPANY, Arkansas Nuclear One, Unit 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ( 12 ) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment No. 37  
Referenced documents have been provided PDR.

Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosure:  
As Stated

OFFICE →	ORB#3:DI					
SURNAME →	PMKreutzer/pn					
DATE →	11/5/82					



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37  
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated July 8, 1982, as supplemented October 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public, and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8211180024 821105  
PDR ADDCK 05000368  
P PDR

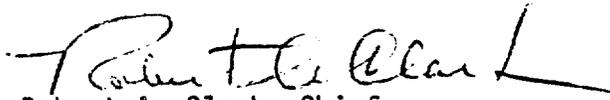
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: November 5, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

Remove Pages

III  
3/4 1-25  
3/4 1-26  
3/4 1-27  
-  
-  
3/4 10-2  
B 3/4 1-5

Insert Pages

III  
3/4 1-25  
3/4 1-26  
3/4 1-27  
3/4 1-28  
3/4 1-29  
3/4 10-2  
B 3/4 1-5

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.0 APPLICABILITY</u> .....	3/4 0-1
<u>3/4.1 REACTIVITY CONTROL SYSTEMS</u>	
3/4.1.1 BORATION CONTROL	
Shutdown Margin - $T_{avg} > 200^{\circ}\text{F}$ .....	3/4 1-1
Shutdown Margin - $T_{avg} \leq 200^{\circ}\text{F}$ .....	3/4 1-3
Boron Dilution .....	3/4 1-4
Moderator Temperature Coefficient .....	3/4 1-5
Minimum Temperature for Criticality.....	3/4 1-6
3/4.1.2 BORATION SYSTEMS	
Flow Paths - Shutdown.....	3/4 1-7
Flow Paths - Operating.....	3/4 1-8
Charging Pump - Shutdown.....	3/4 1-9
Charging Pumps - Operating.....	3/4 1-10
Boric Acid Makeup Pumps - Shutdown.....	3/4 1-11
Boric Acid Makeup Pumps - Operating.....	3/4 1-12
Borated Water Sources - Shutdown.....	3/4 1-13
Borated Water Sources - Operating.....	3/4 1-15
3/4.1.3 MOVABLE CONTROL ASSEMBLIES	
CEA Position.....	3/4 1-17
Position Indicator Channels - Operating.....	3/4 1-20
Position Indicator Channels - Shutdown.....	3/4 1-22
CEA Drop Time.....	3/4 1-23
Shutdown CEA Insertion Limit.....	3/4 1-24
Regulating CEA Insertion Limits.....	3/4 1-25
Part Length CEA Insertion Limits.....	3/4 1-28

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 LINEAR HEAT RATE .....	3/4 2-1
3/4.2.2 RADIAL PEAKING FACTORS .....	3/4 2-4
3/4.2.3 AZIMUTHAL POWER TILT .....	3/4 2-5
3/4.2.4 DNBR MARGIN .....	3/4 2-7
3/4.2.5 RCS FLOW RATE .....	3/4 2-11
3/4.2.6 REACTOR COOLANT COLD LEG TEMPERATURE .....	3/4 2-12
3/4.2.7 AXIAL SHAPE INDEX .....	3/4 2-13
3/4.2.8 PRESSURIZER PRESSURE .....	3/4 2-14
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR PROTECTIVE INSTRUMENTATION .....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION .....	3/4 3-10
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring Instrumentation .....	3/4 3-24
Incore Detectors .....	3/4 3-28
Seismic Instrumentation .....	3/4 3-30
Meteorological Instrumentation .....	3/4 3-33
Remote Shutdown Instrumentation .....	3/4 3-36
Post-Accident Instrumentation .....	3/4 3-39
Chlorine Detection Systems .....	3/4 3-42
Fire Detection Instrumentation .....	3/4 3-43
3/4.3.4 TURBINE OVERSPEED PROTECTION .....	3/4 3-45

## REACTIVITY CONTROL SYSTEMS

### REGULATING CEA INSERTION LIMITS

#### LIMITING CONDITION FOR OPERATION

---

3.1.3.6 The regulating CEA groups shall be limited to the withdrawal sequence and to the insertion limits shown on Figure 3.1-2 with:

- a. CEA insertion between the Long Term Steady State Insertion Limit and the Transient Insertion Limit restricted to:
  1.  $\leq 5$  Effective Full Power Days per 30 Effective Full Power Day intervals, and
  2.  $\leq 14$  Effective Full Power Days per calendar year.
- b. CEA insertion between the Short Term Steady State Insertion Limit and the Transient Insertion Limit shall be restricted to  $\leq 4$  hours per 24 hour interval.

APPLICABILITY: MODES 1\* and 2\*#.

#### ACTION:

- a. With the regulating CEA groups inserted beyond the Transient Insertion Limit, except for surveillance testing pursuant to Specification 4.1.3.1.2, within two hours either:
  1. Restore the regulating CEA groups to within the limits, or
  2. Reduce THERMAL POWER to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the CEA group position using the above figure.
- b. With the regulating CEA groups inserted between the Long Term Steady State Insertion Limit and the Transient Insertion Limit for intervals  $> 5$  EFPD per 30 EFPD interval or  $> 14$  EFPD per calendar year, either:
  1. Restore the regulating groups to within the Long Term Steady State Insertion Limit within two hours, or
  2. Be in at least HOT STANDBY within 6 hours.

\* See Special Test Exceptions 3.10.2 and 3.10.4.

# With  $K_{eff} \geq 1.0$ .

## REACTIVITY CONTROL SYSTEMS

### LIMITING CONDITION FOR OPERATION

---

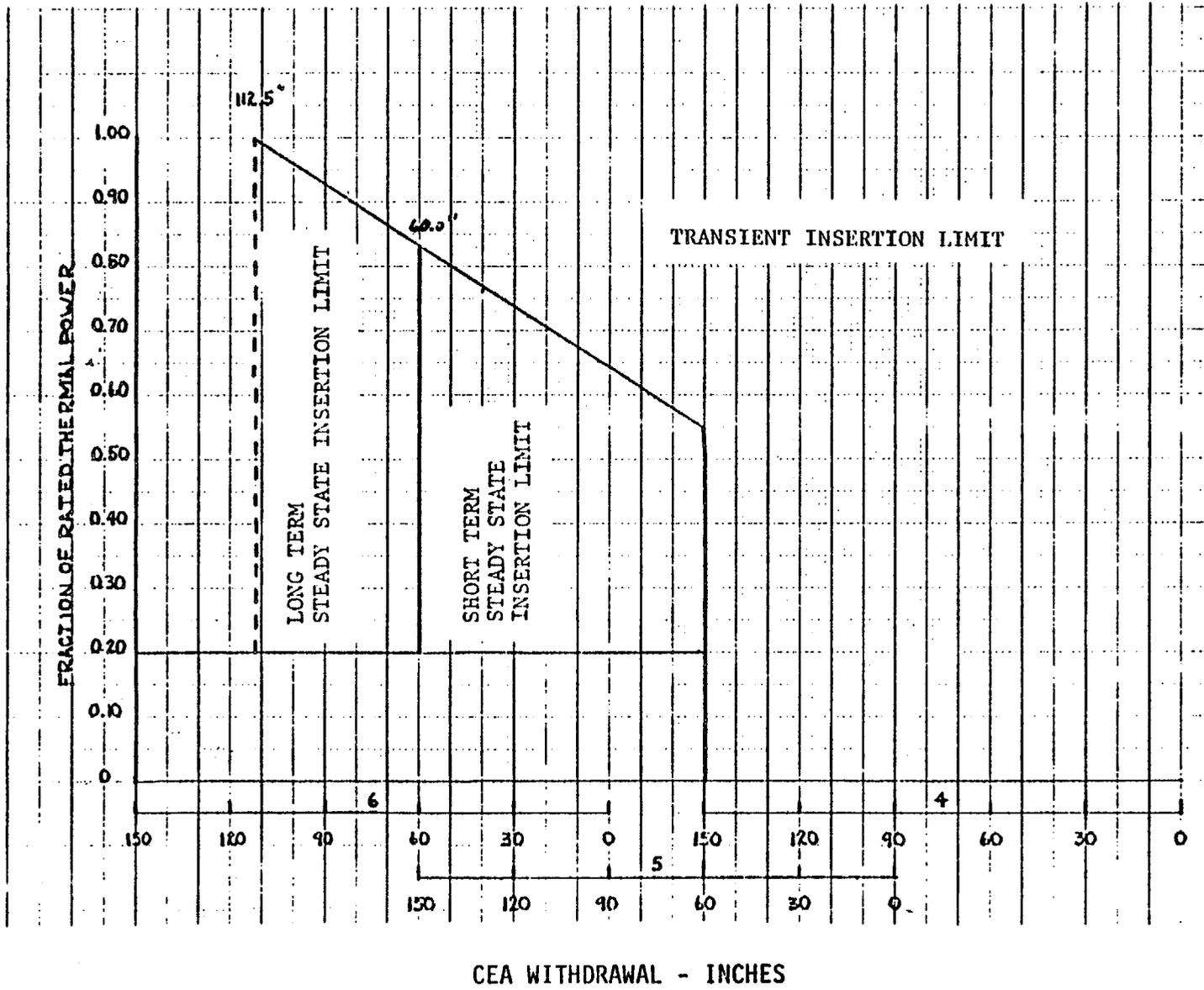
#### ACTION: (Continued)

- c. With the regulating CEA groups inserted between the Short Term Steady State Insertion Limit and the Transient Insertion Limit for intervals > 4 hours per 24 hour interval, operation may proceed provided any subsequent increase in thermal power is restricted to  $\leq 5\%$  of rated thermal power per hour.

### SURVEILLANCE REQUIREMENTS

---

4.1.3.6 The position of each regulating CEA group shall be determined to be within the Transient Insertion Limit at least once per 12 hours except during time intervals when the PDIL Alarm is inoperable, then verify the individual CEA positions at least once per 4 hours. The accumulated times during which the regulating CEA groups are inserted beyond the Long Term Steady State Insertion Limit or the Short Term Steady State Insertion Limit but within the Transient Insertion Limit shall be determined at least once per 24 hours.



CEA WITHDRAWAL - INCHES

FIGURE 3.1-2

CEA Insertion Limits vs THERMAL POWER

## REACTIVITY CONTROL SYSTEMS

### PART LENGTH CEA INSERTION LIMITS

#### LIMITING CONDITION FOR OPERATION

3.1.3.7 The part length CEA group shall be limited to the insertion limits shown on Figure 3.1-3 with PLCEA insertion between the Long Term Steady State Insertion Limit and the Transient Insertion Limit restricted to:

- a.  $\leq 5$  Effective Full Power Days per 30 Effective Full Power Day interval, and
- b.  $\leq 14$  Effective Full Power Days per calendar year.

APPLICABILITY: MODE 1\*.

#### ACTION:

- a. With the part length CEA groups inserted beyond the Transient Insertion Limit, except for surveillance testing pursuant to Specification 4.1.3.1.2, within two hours either:
  1. Restore the part length CEA group to within the limits, or
  2. Reduce THERMAL POWER to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the PLCEA group position using figure 3.1-3.
- b. With the part length CEA groups inserted between the Long Term Steady State Insertion Limit and the Transient Insertion Limit for intervals  $> 5$  EFPD per 30 EFPD interval or  $> 14$  EFPD per calendar year, either:
  1. Restore the part length group to within the Long Term Steady State Insertion Limits within two hours, or
  2. Be in at least HOT STANDBY within 6 hours.

#### SURVEILLANCE REQUIREMENT

4.1.3.7 The position of the part length CEA group shall be determined to be within the Transient Insertion Limit at least once per 12 hours. The accumulated time during which the part length CEA group is inserted beyond the Long Term Steady State Insertion Limit but within the Transient Insertion Limit shall be determined at least once per 24 hours.

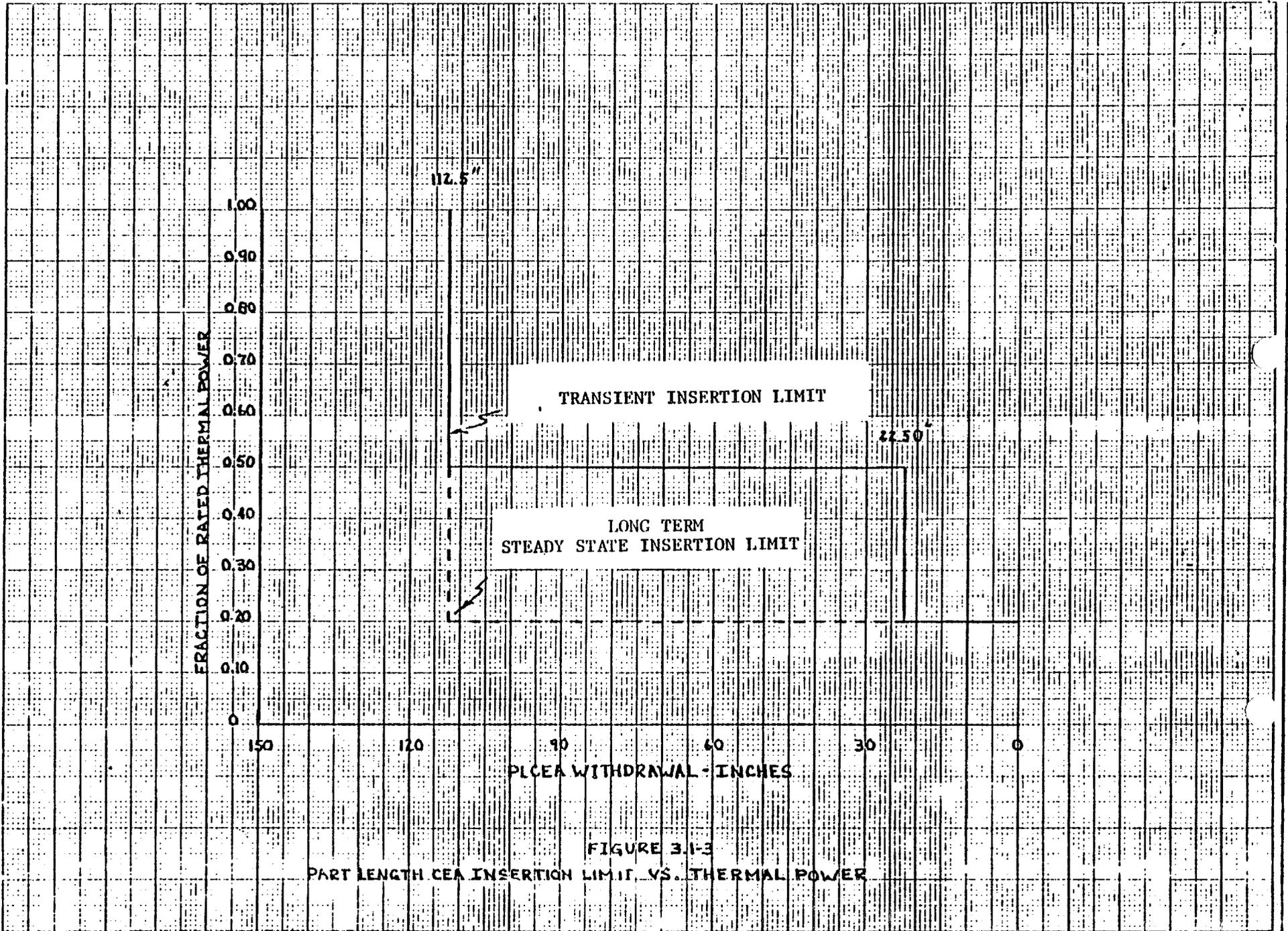


FIGURE 3.1-3  
PART LENGTH CEA INSERTION LIMIT VS. THERMAL POWER

### 3/4.10 SPECIAL TEST EXCEPTIONS

#### SHUTDOWN MARGIN

#### LIMITING CONDITION FOR OPERATION

---

3.10.1 The SHUTDOWN MARGIN requirement of Specification 3.1.1.1 may be suspended for measurement of CEA worth and shutdown margin provided reactivity equivalent to at least the highest estimated CEA worth is available for trip insertion from OPERABLE CEA(s).

APPLICABILITY: MODE 2.

#### ACTION:

- a. With any full length CEA not fully inserted and with less than the above reactivity equivalent available for trip insertion, immediately initiate and continue boration at  $> 40$  gpm of 1731 ppm boric acid solution or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.1 is restored.
- b. With all full length CEAs inserted and the reactor subcritical by less than the above reactivity equivalent, immediately initiate and continue boration at  $\geq 40$  gpm of 1731 ppm boric acid solution or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.1 is restored.

#### SURVEILLANCE REQUIREMENTS

---

4.10.1.1 The position of each full length CEA required either partially or fully withdrawn shall be determined at least once per 2 hours.

4.10.1.2 Each CEA not fully inserted shall be demonstrated capable of full insertion when tripped from at least the 50% withdrawn position within 24 hours prior to reducing the SHUTDOWN MARGIN to less than the limits of Specification 3.1.1.1.

## SPECIAL TEST EXCEPTIONS

### GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS

#### LIMITING CONDITION FOR OPERATION

3.10.2 The group height, insertion and power distribution limits of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.1.3.7, 3.2.2, 3.2.3, 3.2.7 and the Minimum Channels OPERABLE requirement of Functional Unit 15 of Table 3.3-1 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER is restricted to the test power plateau which shall not exceed 85% of RATED THERMAL POWER, and
- b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.2.2 below.

APPLICABILITY: During startup and PHYSICS TESTS.

#### ACTION:

With any of the limits of Specification 3.2.1 being exceeded while any of the above requirements are suspended, either:

- a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1, or
- b. Be in HOT STANDBY within 6 hours.

#### SURVEILLANCE REQUIREMENTS

4.10.2.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which any of the above requirements are suspended and shall be verified to be within the test power plateau.

4.10.2.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detector Monitoring System pursuant to the requirements of Specifications 4.2.1.3 and 3.3.3.2 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which any of the above requirements are suspended.

## REACTIVITY CONTROL SYSTEMS

### BASES

CEA positions and OPERABILITY of the CEA position indicators are required to be verified on a nominal basis of once per 12 hours with more frequent verifications required if an automatic monitoring channel is inoperable. These verification frequencies are adequate for assuring that the applicable LCOs are satisfied.

The maximum CEA drop time restriction is consistent with the assumed CEA drop time used in the accident analyses. Measurement with  $T_{avg} \geq 525^{\circ}\text{F}$  and with all reactor coolant pumps operating ensures that the measured drop times will be representative of insertion times experienced during a reactor trip at operating conditions.

The establishment of LSSS and LCOs require that the expected long and short term behavior of the radial peaking factors be determined. The long term behavior relates to the variation of the steady state radial peaking factors with core burnup and is affected by the amount of CEA insertion assumed, the portion of a burnup cycle over which such insertion is assumed and the expected power level variation throughout the cycle. The short term behavior relates to transient perturbations to the steady-state radial peaks due to radial xenon redistribution. The magnitudes of such perturbations depend upon the expected use of the CEAs during anticipated power reductions and load maneuvering. Analyses are performed based on the expected mode of operation of the NSSS (base load, load following, etc.) and from these analyses CEA insertions are determined and a consistent set of radial peaking factors are defined. The Long Term Steady State and Short Term Insertion Limits are determined based upon the assumed mode of operation used in the analyses and provide a means of preserving the assumptions on CEA insertions used. The limits specified serve to limit the behavior of the radial peaking factors within the bounds determined from analysis. The actions specified serve to limit the extent of radial xenon redistribution effects to those accommodated in the analyses. The Long and Short Term Insertion Limits of Specifications 3.1.3.6 and 3.1.3.7 are specified for the plant which has been designed for primarily base loaded operation but which has the ability to accommodate a limited amount of load maneuvering.

The Transient Insertion Limits of Specification 3.1.3.6 and the Shutdown CEA Insertion Limit of Specification 3.1.3.5 ensure that 1) the minimum SHUTDOWN MARGIN is maintained, and 2) the potential effects of a CEA ejection accident are limited to acceptable levels. Long term operation at the Transient Insertion Limits is not permitted since such operation could have effects on the core power distribution which could invalidate assumptions used to determine the behavior of the radial peaking factors.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-6

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

1.0 Introduction

By letter dated July 8, 1982, Arkansas Power and Light Company proposed a Technical Specification change for ANO-2 to restrict movement of the full length and part length control element assembly (CEA) groups in order to reduce the analytical complexity of the Core Protection Calculator System software validation required for future cycles. The licensee has provided figures 3.1-2 and 3.1-3 as the full length and part length CEA insertion limit curves. A special test exception to specification 3.1.3.7 was requested by AP&L for performance of beginning-of-cycle physics tests. AP&L also requested that specification 3.1.3.6 be revised to expand the LCO portion of the specification to encompass the allowable modes of operation and that the ACTION statement be revised to reflect only additional requirements to the LCO. Our evaluation of the proposed TS change follows.

2.0 Evaluation

Specification 3.1.3.6 presently restricts operation with the regulating CEA groups inserted between the Long Term Steady State Insertion Limit (LTIL) and the Transient Insertion Limit (TIL) to periods less than 4 hours per 24 hour interval. An ACTION statement allows operation to proceed if this time interval condition is exceeded as long as the Short Term Steady State Insertion Limit (STIL) is not exceeded. This Technical Specification has been reformulated to essentially combine the ACTION statement with the insertion limit Limiting Condition for Operation (LCO). We find this modification acceptable since operation with regulating CEA insertion between the STIL and the TIL for more than 4 hours per 24 hour interval is still prohibited.

The full length CEA insertion limit curve (Fig. 3.1-2) has been modified so that CEA groups 2, 3, and 4 are fully withdrawn above 20% of rated thermal power. This results in increased shutdown margin due to additional scram worth for low power steam line break events. It also tends to reduce CEA worths and thus mitigates the consequences of Anticipated Operational Occurrences (AOOs), such as the CEA group withdrawal, and of accidents such as the CEA ejection. This modification is, therefore, acceptable.

8211180045 821105  
PDR ADOCK 05000368  
PDR

Specification 3.1.3.7 governing the LCO for part length CEA (PLCEA) insertion limits has been added. The specification includes Figure 3.1-3 which gives allowable transient and long term part length CEA insertion limits as a function of thermal power. We find this additional Technical Specification acceptable since it incorporates the CE Standard Technical Specification (STS) LCO restriction on core residence time and ACTION statements as well as the STS surveillance requirements. Also, the insertion limits have been established by previously used and approved methods. In addition, any single PLCEA or PLCEA group drop from the allowable insertion limit is bounded by the ANO-2 FSAR safety analyses.

We also find the inclusion of Technical Specification 3.1.3.7 in specification 3.10-2 for special test exceptions to be acceptable since adequate surveillance requirements and appropriate actions are specified if the linear heat rate exceeds its limits as specified in Figure 3.2-1. The PLCEA insertion limit exception is required for performance of start-up physics test measurements of power distributions.

### 3.0 Summary

The staff has reviewed the proposed Technical Specification change relating to the regulating and part length CEA insertion limits. The revised technical specification 3.1.3.6 and the proposed technical specification 3.1.3.7 have been found acceptable.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health

and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 5, 1982

Principal Contributors:

L. I. Kopp  
J. A. Stevens

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-368ARKANSAS POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 37 to Facility Operating License No. NPF-6 issued to Arkansas Power & Light Company (the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Unit 2 (the facility), located in Pope County, Arkansas. The amendment is effective as of the date of issuance.

The amendment restricts the movement of the full length and part length control element assembly (CEA) groups and allows for a special test exception for the part length CEA group.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

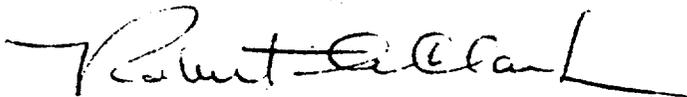
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 8, 1982, as supplemented October 6, 1982, (2) Amendment No. 37 to Facility Operating License No. NPF-6, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room at 1717 H Street, N.W., Washington, D.C. 20555 and at the Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda Maryland, this 5th day of November, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing