

July 18, 1990

Docket No. 50-368

Mr. Neil S. Carns
Vice President, Nuclear
Arkansas Nuclear One
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Carns:

SUBJECT: ISSUANCE OF AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE
NO. NPF-6 - ARKANSAS NUCLEAR ONE, UNIT NO. 2 (TAC NO. 75925)

The Commission has issued the enclosed Amendment No. 108 to Facility Operating License No. NPF-6 for the Arkansas Nuclear One, Unit No. 2 (ANO-2). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 15, 1989.

The amendment makes several administrative changes to the Arkansas Nuclear One, Unit 2 Technical Specifications (TSs) Table 3.6-1. These include correcting the indicated location of a containment isolation valve, and the relabelling of two other containment isolation valves to reflect a design change. The amendment also corrects a TS reference included in Specification 4.5.1.5.2. The effective date of this change is 30 days after issuance of this amendment to allow for distribution and procedure revisions by the licensee.

A copy of our related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

//s//

Chester Poslusny, Jr., Project Manager
Project Directorate IV-1
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 108 to NPF-6
2. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Chester Poslusny Jr.

Chester Poslusny, Jr., Project Manager
Project Directorate IV-1
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IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 108 to NPF-6
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company dated December 15, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

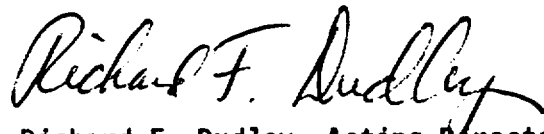
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 108, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective 30-days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard F. Dudley, Acting Director
Project Directorate IV-1
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Revise the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES

3/4 6-9

3/4 6-18

3/4 6-20

3/4 6-21

INSERT PAGES

3/4 6-9

3/4 6-18

3/4 6-20

3/4 6-21

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.1.5.2 End Anchorages and Adjacent Concrete Surfaces The structural integrity of the end anchorages of all tendons inspected pursuant to Specification 4.6.1.5.1 and the adjacent concrete surfaces shall be demonstrated by determining through inspection that no apparent changes have occurred in the visual appearance of the end anchorage or the concrete crack patterns adjacent to the end anchorages. Inspections of the concrete shall be performed during the Type A containment leakage rate tests (reference Specification 4.6.1.2) while the containment is at its maximum test pressure.

4.6.1.5.3 Containment Surfaces The structural integrity of the exposed accessible interior and exterior surfaces of the containment, including the liner plate, shall be determined during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2) by a visual inspection of these surfaces and verifying no apparent changes in appearance or other abnormal degradation.

4.6.1.5.4 Deleted

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by verifying that on a containment isolation test signal, each isolation valve actuates to its isolation position.

4.6.3.1.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.1.4 Prior to exceeding conditions which require establishment of reactor building integrity per TS 3.6.1.1, the leak rate of the containment purge supply and exhaust isolation valves listed in Table 3.6-1 Part B shall be verified to be within acceptable limits per TS 4.6.1.2, unless the test has been successfully completed within the last three months.

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC)</u>
A. CONTAINMENT ISOLATION			
2P7	2CV-5852-2#	"A" S/G Sample Isolation (outside)	≤ 20
	2CV-5859-2#	"B" S/G Sample Isolation (outside)	≤ 20
2P8	2SV-5833-1	RCS & Pressurizer Sample Isolation (inside)	≤ 20
	2SV-5843-2	RCS & Pressurizer Sample Isolation (outside)	≤ 20
2P9	2CV-6207-2	H.P. Nitrogen to SI Tanks (outside)	≤ 20
2P14	2CV-4821-1	CVCS L/D Isolation (inside)	≤ 35
	2CV-4823-2	CVCS L/D Isolation (outside)	≤ 20
2P18	2CV-4846-1	RCP Seal Return Isolation (inside)	≤ 25
	2CV-4847-2	RCP Seal Return Isolation (outside)	≤ 20
2P31	2CV-2401-1	Containment Vent Header (inside)	≤ 20
	2CV-2400-2	Containment Vent Header (outside)	≤ 20
2P37	2SV-5878-1	Quench Tank Liquid Sample (inside)	≤ 20
	2SV-5871-2	Quench Tank Liquid Sample (outside)	≤ 20
	2SV-5876-2	SI Tanks Sample Isolation (outside)	≤ 20
2P39	2CV-4690-2	Quench Tank Makeup & Demin Water Supply Isolation (outside)	≤ 20
2P40	2CV-3200-2	Fire Water Isolation (outside)	≤ 20
2P41	2CV-6213-2	L.P. Nitrogen Supply Isolation (outside)	≤ 20
2P51	2CV-3852-1	Chilled Water Supply Isolation (outside)	≤ 20
2P52	2CV-5236-1	CCW to RCP Coolers Isolation (outside)	≤ 20
2P59	2CV-3850-2	Chilled Water Return Isolation (inside)	≤ 20
	2CV-3851-1	Chilled Water Return Isolation (outside)	≤ 20
2P60	2CV-5254-2	CCW from RCP Coolers Isolation (inside)	≤ 20
	2CV-5255-1	CCW from RCP Coolers Isolation (outside)	≤ 20

TABLE 3.6-1 (Cont.)

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC)</u>
A. CONTAINMENT ISOLATION (Cont.)			
2P68	2CV-2060-1	Containment Sump Drain Isolation (inside)	< 20
	2CV-2061-2	Containment Sump Drain Isolation (outside)	< 20
2P69	2CV-2202-1	Reactor Drain Tank Discharge (inside)	< 20
	2CV-2201-2	Reactor Drain Tank Discharge (outside)	< 20
B. CONTAINMENT PURGE			
2P6	2SV-8231-2	H ₂ Purge Inlet Isolation (outside)	< 20
	2SV-8273-1	H ₂ Purge Outlet Isolation (inside)	< 20
	2SV-8271-2	H ₂ Purge Outlet Isolation (outside)	< 20
2P58	2SV-8261-2	Containment Atmosphere Sample (outside)	< 20
	2SV-8265-1	Containment Atmosphere Sample (inside)	< 20
	2SV-8263-2	Containment Atmosphere Sample (outside)	< 20
2V1	2CV-8289-1	Containment Purge Inlet Isolation (inside)	< 5
	2CV-8284-2	Containment Purge Inlet Isolation (outside)	< 5
	2CV-8283-1	Containment Purge Inlet Isolation (outside)	< 5
2V2	2CV-8291-1	Containment Purge Outlet Isolation (inside)	< 5
	2CV-8286-2	Containment Purge Outlet Isolation (outside)	< 5
	2CV-8285-1	Containment Purge Outlet Isolation (outside)	< 5

TABLE 3.6-1 (Cont.)

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC)</u>
C. MANUAL			
2P7	2CV-5850#*	"A" S/G Sample Isolation (inside), operated from 2C17	N.A.
	2CV-5858#*	"B" S/G Sample Isolation (inside), operated from 2C17	N.A.
2P19	2FP-35#	Refueling Canal Recirculation Line (inside)	N.A.
	2FP-34	Refueling Canal Recirculation Line (outside)	N.A.
2P32	2CV-1015#*	"A" S/G Blowdown Isolation (inside), operated from 2C33	N.A.
2P37	2SV-5872#*	SI Tank 2T2A Sample (inside)	N.A.
	2SV-5873#*	SI Tank 2T2B Sample (inside)	N.A.
	2SV-5874#*	SI Tank 2T2C Sample (inside)	N.A.
	2SV-5875#*	SI Tank 2T2D Sample (inside)	N.A.
2P42	2PH-44#	Plant Heating Return Isolation (inside)	N.A.
	2PH-45	Plant Heating Return Isolation (outside)	N.A.
2P43	2SA-68	Service Air Supply Isolation (outside)	
2P46	2BA-217	Breathing Air Supply Isolation (outside)	
2P48	2PH-22	Plant Heating Supply Isolation (outside)	N.A.
	2PH-23#	Plant Heating Supply Isolation (inside)	N.A.
2P64	2CV-1065#*	"B" S/G Blowdown Isolation (inside), operated from 2C33	N.A.

TABLE 3.6-1 (Cont.)CONTAINMENT ISOLATION VALVES

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC)</u>
C. MANUAL (Cont.)			
2C3	2CV-5432#	Fuel Transfer Tube Isolation (outside)	N.A.
D. OTHER (Check Valves)			
2P9	2N ₂ -18#	H.P. Nitrogen to SI Tanks (inside)	N.A.
2P39	2CVC-78#	Quench Tank Makeup & Demin Water Supply (inside)	N.A.
2P40	2FS-37#	Fire Water Isolation (inside)	N.A.
2P41	2N ₂ -1#	L.P. Nitrogen Supply Isolation (inside)	N.A.
2P43	2SA-69	Service Air Supply Isolation (inside)	N.A.
2P46	2BA-216	Breathing Air Supply Isolation (inside)	N.A.
2P51	2AC-49#	Chilled Water Supply Isolation (inside)	N.A.
2P52	2CCW-38#	CCW to RCP Coolers Isolation (inside)	N.A.

Not subject to Type C leakage tests.

*May be opened on an intermittent basis under administrative control.

CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two independent containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

With one hydrogen analyzer inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases containing:

- a. Zero volume percent hydrogen, balance nitrogen, and
- b. Four volume percent (nominal) hydrogen, balance nitrogen.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 108 TO

FACILITY OPERATING LICENSE NO. NPF-6

ENTERGY OPERATION, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

INTRODUCTION

By letter dated December 15, 1989, Arkansas Power and Light Company (AP&L) requested amendments to the Technical Specifications (TSs) appended to Facility Operating License No. NPF-6 for Arkansas Nuclear One, Unit 2 (ANO-2). The proposed amendment would modify Table 3.6-1, Containment Isolation Valves, to denote several administrative corrections associated with the physical location and with the relabelling of valves. Table 3.6-1 provides the list of containment isolation valves which are subject to various surveillance requirements. In addition, the amendment also corrects a typographical error in Specification 4.5.1.5.2.

EVALUATION

AP&L stated in its submittal that the current functional description of penetration 2P60 valve # 2CV-5255-1 in Table 3.6-1 is inaccurate. According to the physical location of the valve, the functional description should be "CCW from RCP Coolers Isolation (Outside)" instead of the existing description which is "CCW from RCP Coolers Isolation (Inside)". This change is acceptable because the revised wording corrects the specified as installed location of the valve. Also in Table 3.6-1, as a result of design changes which involved disconnecting the instrument air line to the reactor building, reconnecting the reactor instrument air line to breathing air system, and redesigning the affected valve and piping as part of the breathing air system, penetration 2P46 valves #21A-14 and #21A-15 were renumbered as valves #2BA-217 and #2BA-216, respectively. This change is acceptable because the listing of valve numbers is revised to accurately reflect the new labelling nomenclature and the surveillance requirements remain unchanged.

In addition, AP&L also proposed to correct a typographical error in Specification 4.6.1.5.2. Specification 4.6.1.5.2 previously stated that the structural integrity of the anchorages of all tendons be inspected pursuant to Specification 4.6.1.6.1. However, there is no such Specification 4.6.1.6.1. The correct reference should be Specification 4.6.1.5.1.

The staff has reviewed the changes requested by the AP&L and has determined that each of the above items are purely administrative in nature and result in improving clarity and accuracy of the information in the Technical Specifications. The changes do not decrease or otherwise modify existing requirements. Therefore, they are acceptable.

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ENVIRONMENTAL CONSIDERATION

The amendment related to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 18, 1990

Principal Contributor: L. Tran