Mr. C. Randy Hutchins Vice President, Operations GGNS Entergy Operations, Inc. P. O. Box 756 Port Gibson, MS 39150

SUBJECT: CORRECTION TO AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE

NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M88101)

Dear Mr. Hutchinson:

On February 21, 1995, the Nuclear Regulatory Commission issued Amendment No. 120 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment replaced entirely, the then current Technical Specifications (TSs) for Unit 1 with new TSs based on NUREG-1434, "Improved BWR-6 Technical Specifications," dated September 1992.

Your staff has identified a typographical error in the new TSs. The error is in the number of channels required to be operable for the main steam tunnel ambient temperature trip system, tripping on high temperature, in Table 3.3.6.1-1, Primary Containment and Drywell Isolation Instrumentation, Page 3.3-54. The correct number of channels per trip system is 2, as was stated in the TSs for this instrumentation before Amendment No. 120 was issued; however, the number stated in the new TSs is 8. The correct number, 2, is consistent with the discussion on this instrumentation in the Bases of the new TSs, on Page B 3.3-144.

In the safety evaluation enclosed with its February 21, 1995, letter, the NRC staff addressed the proposed changes to the then current TS requirements. The number of required channels was not addressed in Section 3.3, Instrumentation of the safety evaluation because the intent of Amendment No. 120 was not to change the required number of channels. Based on discussions with your staff, the typographical error occurred when the incorrect number, 8, was inadvertently taken from NUREG-1434 in the typing of the pages for the new TSs. The typographical error was not identified by the NRC staff before the new TSs were issued. A revised TS Page 3.3-54 is being issued with the correct number of required channels, 2, and is enclosed with this letter.

PD4-1 r/f

Sincerely, No. Two Williams Senior Project Manager Jack N. Donohew, Senior Project Manager Project Directorate IV-1 Division of Reactor Projects III/IV Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: TS Page 3.3-54

cc w/encl: See next page

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An error has been identified in the new TSs. The error is in the number of channels required to be operable for the main steam tunnel ambient temperature trip system, tripping on high temperature, in Table 3.3/6.1-1, Primary Containment and Drywell Isolation Instrumentation, Page 3.3-54. The correct number of channels is 2, as was stated in the TSs for this instrumentation before Amendment No. 120 was issued; however, the number stated in the current TSs is 8. The correct number is consistent with the discussion on this instrumentation in the Bases of the new TSs, on Page B 3.3-144.

In the safety evaluation enclosed to its February 21, 1995, letter, the staff addressed the proposed changes to the then current TS requirements. The number in question in Table 3.3.6.1-1 was not addressed in Section 3.3, Instrumentation of the safety evaluation because the intent of Amendment No. 120 was not to change this required number of channels. Based on discussions with your staff, the incorrect number was inadvertently taken from NUREG-1434 in the typing of the pages for the new TSs and was not identified by the staff before the new TSs were issued. A revised TS Page 3.3-54 is being issued with the correct number of required channels and is enclosed with this letter.

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Jack N. Donohew, Senior Project Manager Project Directorate IV-1 Division of Reactor Projects III/IV Office of Nuclear Reactor Regulation

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DATE	D //1/96	244/96	5/9/96	/ /96	
СОРУ	YES/NO	(YES/NO	(YES/NO	YES/NO	

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## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 17, 1996

Mr. C. Randy Hutchinson Vice President, Operations GGNS Entergy Operations, Inc. P. O. Box 756 Port Gibson, MS 39150

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Jack N. Donohew, Senfor Project Manager

Project Directorate IV-1

Division of Reactor Projects III/IV Office of Nuclear Reactor Regulation

Docket No. 50-416

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cc w/encl: See next page

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The Honorable William J. Guste, Jr. Attorney General Department of Justice State of Louisiana P. O. Box 94005 Baton Rouge, LA 70804-9005

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Office of the Governor State of Mississippi Jackson, MS 39201

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Mr. Jerrold G. Dewease Vice President, Operations Support Entergy Operations, Inc. P.O. Box 31995 Jackson, MS 39286-1995

Mr. Michael J. Meisner Director, Nuclear Safety and Regulatory Affairs Entergy Operations, Inc. P.O. Box 756 Port Gibson, MS 39150

Table 3.3.6.1-1 (page 1 of 5)
Primary Containment and Drywell Isolation Instrumentation

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Ma	in Steam Line Isolation					
a.	Reactor Vessel Water Level — Low Low Low, Level 1	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ -152.5 inches
b.	Main Steam Line Pressure – Low	1	2	E	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ 837 psig
c.	Main Steam Line Flow — High	1,2,3	2 per MSL	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 176.5 psid
d.	Condenser Vacuum — Low	1,2 <sup>(a)</sup> , 3 <sup>(a)</sup>	2	<b>D</b>	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ 8.7 inches Hg vacuum
e.	Main Steam Tunnel Ambient Temperature — High	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.5 SR 3.3.6.1.7	≤ 191°F
f.	Manual Initiation	1,2,3	2	G	SR 3.3.6.1.7	NA
	imary Containment and ywell Isolation					
a.	Reactor Vessel Water Level — Low Low, Level 2	1,2,3	2 <sup>(b)</sup>	<b>H</b>	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ -43.8 inches
						(continued)

<sup>(</sup>a) With any turbine stop valve not closed.

<sup>(</sup>b) Also required to initiate the associated drywell isolation function.