

September 4, 1996

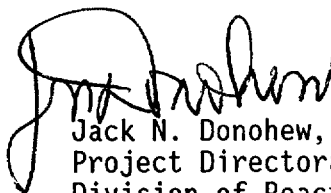
Mr. C. Randy Hutchinson
Vice President, Operations GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION
DETERMINATION, AND OPPORTUNITY FOR HEARING - GRAND GULF NUCLEAR
STATION, UNIT 1 (TAC NO. M95403)

Dear Mr. Hutchinson:

The Commission has requested the Office of the Federal Register to publish the enclosed "Notice of Consideration of Issuance of Amendment, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing." This notice relates to your application for amendment dated May 9, 1996, as supplemented by letter dated August 27, 1996, which requested a revision to the Technical Specifications to allow the surveillance of the relief mode of operation of each of the 20 safety/relief valves (S/RVs) without physically lifting the disk off the seat at power.

Sincerely,



Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Notice

cc w/encl: See next page

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W. Beckner
P. Tressler

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OFC	PM/PD4-1	LA/PD4-1
NAME	JDonohew/vw	PTressler
DATE	9/4/96	9/7/96
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 4, 1996

Mr. C. Randy Hutchinson
Vice President, Operations GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

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Sincerely,

A handwritten signature in black ink, reading "Jack N. Donohew", is written over the typed name and title.

Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Notice

cc w/encl: See next page

Mr. C. Randy Hutchinson
Entergy Operations, Inc.

Grand Gulf Nuclear Station

cc:

Executive Vice President
& Chief Operating Officer
Entergy Operations, Inc.
P. O. Box 31995
Jackson, MS 39286-1995

General Manager, GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

Wise, Carter, Child & Caraway
P. O. Box 651
Jackson, MS 39205

Attorney General
Department of Justice
State of Louisiana
P. O. Box 94005
Baton Rouge, LA 70804-9005

Winston & Strawn
1400 L Street, N.W. - 12th Floor
Washington, DC 20005-3502

State Health Officer
State Board of Health
P. O. Box 1700
Jackson, MS 39205

Director
Division of Solid Waste Management
Mississippi Department of Natural
Resources
P. O. Box 10385
Jackson, MS 39209

Office of the Governor
State of Mississippi
Jackson, MS 39201

President,
Claiborne County Board of Supervisors
Port Gibson, MS 39150

Attorney General
Asst. Attorney General
State of Mississippi
P. O. Box 22947
Jackson, MS 39225

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Vice President, Operations Support
Entergy Operations, Inc.
P.O. Box 31995
Jackson, MS 39286-1995

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
Route 2, Box 399
Port Gibson, MS 39150

Director, Nuclear Safety
and Regulatory Affairs
Entergy Operations, Inc.
P.O. Box 756
Port Gibson, MS 39150

Nuclear Operating Plant Services
Bechtel Power Corporation
9801 Washington Boulevard
Gaithersburg, MD 20878

UNITED STATES NUCLEAR REGULATORY COMMISSIONENTERGY OPERATIONS, INC.DOCKET NO. 50-415 ✓

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-29 issued to Entergy Operations, Inc. (the licensee) for operation of Grand Gulf Nuclear Station, Unit 1 (GGNS) located in Claiborne County, Mississippi.

The proposed amendment would allow the licensee to perform the surveillance of the relief mode of operation of each of the 20 safety/relief valves (S/RVs) on the four main steam lines without physically lifting the disk of the valve off the seat at power. The proposed changes are to Surveillance Requirements (SRs) 3.4.4.3, Safety/Relief Valves, 3.5.1.7, Automatic Depressurization System Valves, and 3.6.1.6.1, Low-Low Set Valves, of the Technical Specifications, and the changes would state that the required operation of the valve to verify is that the relief-mode actuator strokes when the valve is manually actuated and the frequency of the SRs would be in accordance with the inservice testing program for the valves.

Each S/RV is a Dikkers, 8 X 10, direct-acting, spring loaded, safety valve with attached pneumatic actuator for relief-mode operation. Eight of the S/RVs use the relief mode to perform the Automatic Depressurization System (ADS) function. Also, six S/RVs, two of which are also ADS S/RVs, use the relief mode to perform the Low-Low Valve set function.

The licensee submitted an application for amendment dated May 9, 1996, as supplemented by letter August 27, 1996. A Notice of Consideration of Issuance

of an Amendment to the license was issued in the FEDERAL REGISTER on June 19, 1996 (61 FR 31177), for the letter of May 9, 1996. The modification to the application in the letter of August 27, 1996, is to (1) state that the frequency of performing the SRs will be "in accordance with the inservice testing program" for the valves and (2) delete the word "required" for SRs 3.5.1.7 and 3.6.1.6.1.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the request for amendment involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

- a. No significant increase in the probability or consequences of an accident previously evaluated results from this change.

Each refueling outage, a test sample of the population of S/RVs is removed from the plant to perform testing as required by ASME Boiler and Pressure Vessel Code, Section XI. These S/RVs will be stroked in the relief mode during as-found testing, and are therefore verified to operate properly when each S/RV stem is raised by the relief-mode pneumatic actuator. This proposed surveillance verifies proper S/RV relief-mode operation of all installed S/RVs based upon this test sample. This testing, in conjunction with replacement of each S/RV prior to the

end of its expected service life, provides reasonable assurance that the installed S/RVs will perform as well as the test population of S/RVs.

After the S/RVs have been replaced in the plant, and after all controls are reconnected, the relief-mode actuator on each newly-installed S/RV will be uncoupled from the S/RV stem, and stroked. This actuator stroke will verify that no damage has occurred to the relief-mode actuator during S/RV transportation from its storage location to its operating location. The direct coupling of the valve stem to disk provides assurance that proper relief actuation will occur when the actuator is operated. The safety-mode components are completely encased within the valve body and bonnet, which provides a rugged structure to prevent damage to these components. The remaining installed S/RVs will continue to be tested for proper control system function as previously required by Technical Specifications. The direct coupling of the S/RV stem to disk provides assurance that proper relief-mode actuation will occur when the actuator is operated. The safety mode of the GGNS S/RVs is not affected by a malfunction of the relief-mode components.

Blockage of each S/RV discharge line will be prevented by the same Foreign Material Exclusion (FME) controls that exist for other reactor vessel and support systems. These FME controls, combined with the horizontal orientation of the S/RV discharge piping mating surfaces, provide reasonable assurance that discharge line blockage will not occur.

Therefore, no significant increase in the probability or consequences of an accident previously evaluated results from this proposed change.

- b. This change would not create the possibility of a new or different kind of accident from any previously analyzed.

The proposed change demonstrates that each S/RV will perform its intended relief-mode function, which is the intent of the present surveillance. The relief mode of S/RV operation is demonstrated to be operable based upon successful performance of a test population, S/RV component service life, and existing Technical Specification surveillances. No new failure mechanisms to the relief-mode of operation are introduced, as the proposed surveillance verifies relief actuator operability. Plant FME controls, combined with the horizontal orientation of the S/RV discharge piping mating flange, provides reasonable assurance that discharge line blockage will not occur.

This proposed change does not add any new systems, structures, or components, nor does it introduce new S/RV operating modes.

Therefore, this change would not create the possibility of a new or different kind of accident from any previously analyzed.

- c. This change would not involve a significant reduction in the margin of safety.

This proposed change will verify that the relief mode of all installed S/RVs will operate properly based upon demonstrated relief mode performance of a sample of S/RVs. The failure mode of the S/RV relief function would require a failure of either the pneumatic actuator, lifting linkage, or solenoid block. Each of these items has been verified to have a service life exceeding the replacement cycle of each S/RV. Therefore, proper operation of a sample population of S/RVs provides reasonable assurance that the remaining S/RVs would perform identically, within the original margin of expected S/RV operability. In addition, each S/RVs solenoid block and control functions will continue to be tested and cycled each refueling outage. The removal of the valve stroke surveillance for all S/RVs does not increase the possibility of valve malfunction, since valve stroke is verified during the as-found testing of the sample population of S/RVs. This proposed surveillance test reduces the number of S/RV actuations, and therefore, reduces challenges to the system both mechanically and thermally. Also, the proposed alternative method of testing reduces the possibility of a stuck-open S/RV, since this proposed method will not stroke the S/RVs with the reactor pressurized during reactor power operations.

Therefore, this change would not involve a significant reduction in the margin of safety.

Based on the above evaluation, Entergy Operations, Inc. has concluded that operation in accordance with the proposed amendment involves no significant hazards considerations.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would

result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By October 11, 1996, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance

with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Judge George W. Armstrong Library, 220 S. Commerce Street, Natchez, Mississippi (39120). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days

prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to William D. Beckner: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Nicholas S.

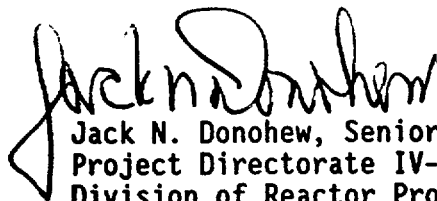
Reynolds, Esquire, Winston and Strawn, 1400 L Street, N.W., 12th Floor, Washington, DC (20005-3502), attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated May 9, 1996, as supplemented by letter dated August 27, 1996, which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Judge George W. Armstrong Library, 220 S. Commerce Street, Natchez, MS 39120.

Dated at Rockville, Maryland, this 4th day of September, 1996.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Jack N. Donohew". The signature is stylized with a large, looped initial "J" and a cursive script for the rest of the name.

Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation