

November 13, 1978

Dockets Nos.: 50-313
and 50-368

Mr. William Cavanaugh, III
Executive Director, Generation
and Construction Department
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendments Nos. 37 and 5 to Facility Operating Licenses Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1 and 2 (ANO-1&2). The amendments consist of changes to the Technical Specifications in response to your application dated September 22, 1978, as supplemented October 17, 1978.

The amendments modify the ANO-1&2 Technical Specifications dealing with the plant organization structure, and revise the ANO-1 Administrative Controls of the Environmental Technical Specifications to make them current and consistent with ANO-2.

During our review, we determined that modifications to the requested changes were necessary. These changes were discussed with and agreed to by your staff and have been incorporated.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Signature by

John F. Stolz, Chief
Light Water Reactors Branch #1
Division of Project Management

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

1. Amendment No. 37 to DPR-51
2. Amendment No. 5 to NPF-6
3. Safety Evaluation
4. Notice

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SURNAME >	RIngram	GVissing	LEngle		JStolz	RReid
DATE >	11/8/78	11/8/78	11/8/78	11/9/78	11/13/78	11/13/78



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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and 50-368

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Executive Director, Generation
and Construction Department
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

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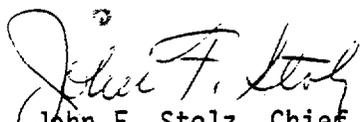
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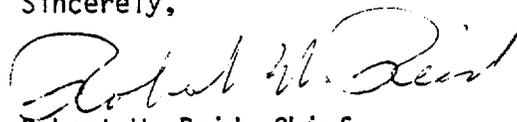
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John F. Stolz, Chief
Light Water Reactors Branch #1
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4. Notice

cc w/enclosures: See next page

Arkansas Power & Light Company

cc w/enclosures:

Phillip K. Lyon, Esquire
House, Holms & Jewell
1550 Tower Building
Little Rock, Arkansas 72201

Mr. Daniel H. Williams
Manager, Licensing
Arkansas Power & Light Company
Post Office Box 551
Little Rock, Arkansas 72203

Mr. John W. Anderson, Jr.
Plant Superintendent
Arkansas Nuclear One
Post Office Box 608
Russellville, Arkansas 72801

Arkansas Polytechnic College
Russellville, Arkansas 72801

Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Chief, Energy Systems Analyses
Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region VI Office
ATTN: EIS COORDINATOR
1201 Elm Street
First International Building
Dallas, Texas 75270

Mr. E. H. Smith, Project Engineer
Bechtel Power Corporation
San Francisco, California 94119

Mr. Fred Sernatinger, Project Manager
Combustion Engineering, Inc.
1000 Prospect Hill Road
Windsor, Connecticut 06095

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
C-E Power Systems
Combustion Engineering, Inc.
4853 Cordell Avenue, Suite A-1
Bethesda, Maryland 20014

cc w/enclosure(s) and incoming
dtd.: 9/22/78 and 10/17/78
Director, Bureau of Environmental
Health Services
4815 West Markham Street
Little Rock, Arkansas 72201



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated September 22, 1978, as supplemented October 17, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective no later than 90 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 13, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages are also provided to maintain document completeness.

Appendix "A" Pages

117

119

120

120-a (new)

121

121-a (new)

122 - 124

126 & 127

Appendix "B" Pages

ii & iii

2-5

5-1 - 5-9

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

- 6.2.1. The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figures 6.2-2A, B, C, and D. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for fire protection training shall be maintained and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 with the exception of frequency of training which shall be six times per year.

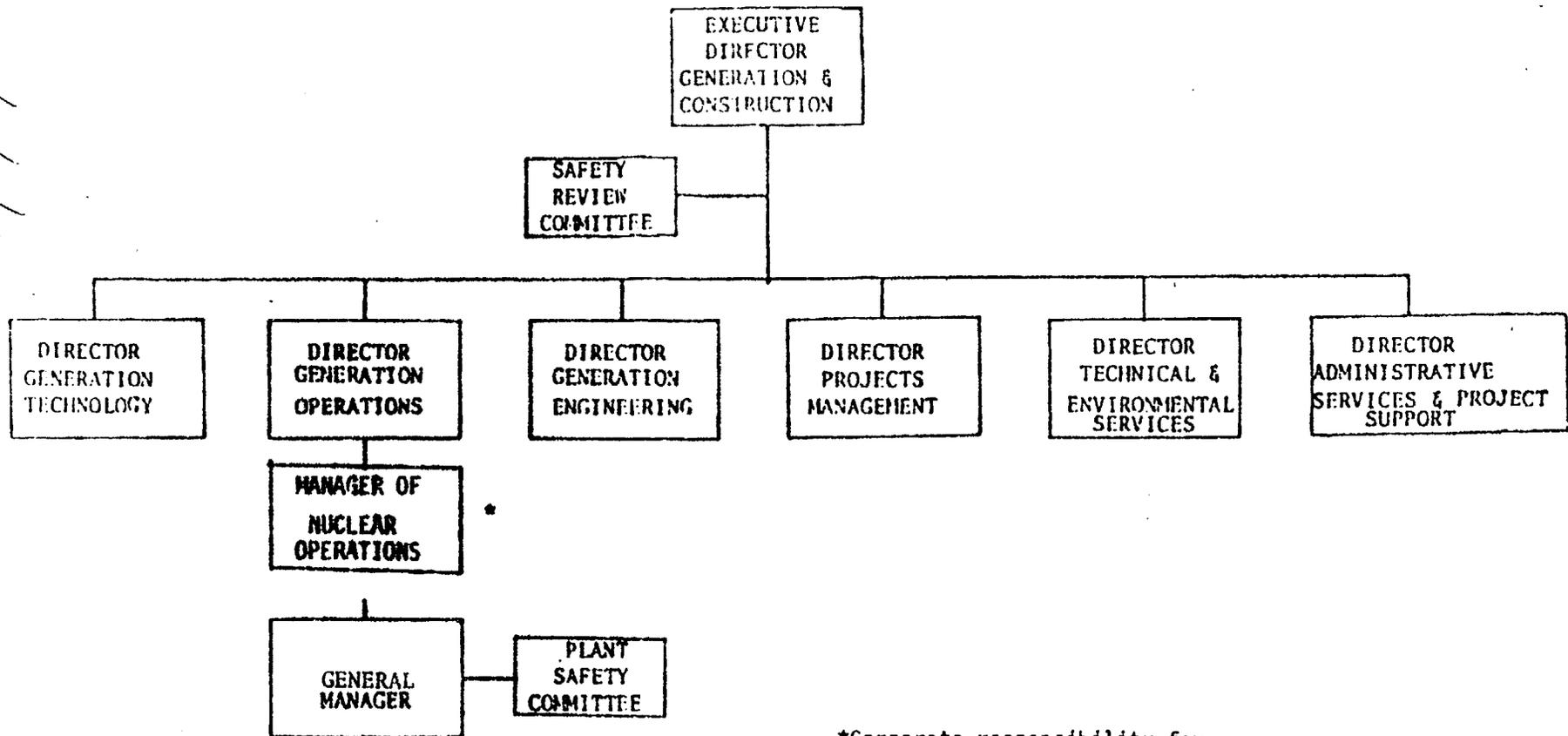
6.5 REVIEW AND AUDIT

6.5.1 Plant Safety Committee (PSC) Function

- 6.5.1.1 The Plant Safety Committee shall function to advise the General Manager on all matters related to nuclear safety.

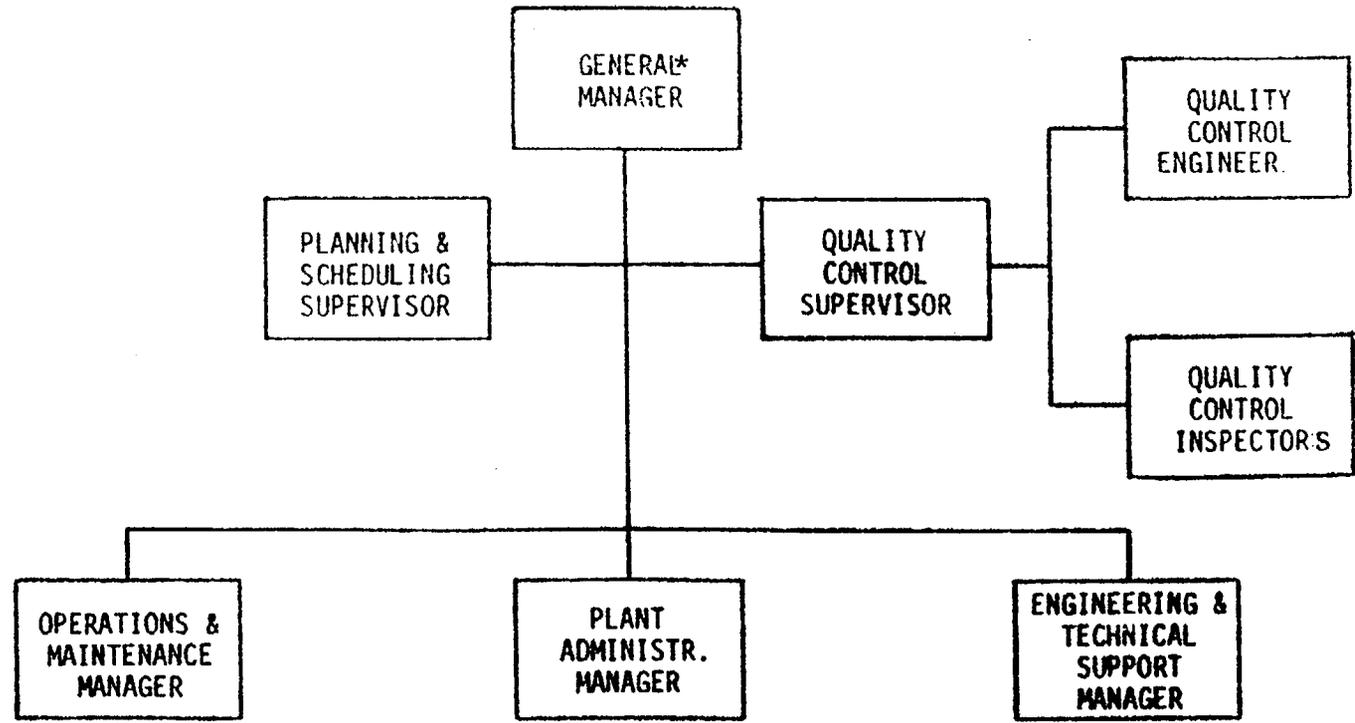
COMPOSITION

- 6.5.1.2 The Plant Safety Committee shall be composed of the:



*Corporate responsibility for Fire Protection Program

FIGURE 6.2-1

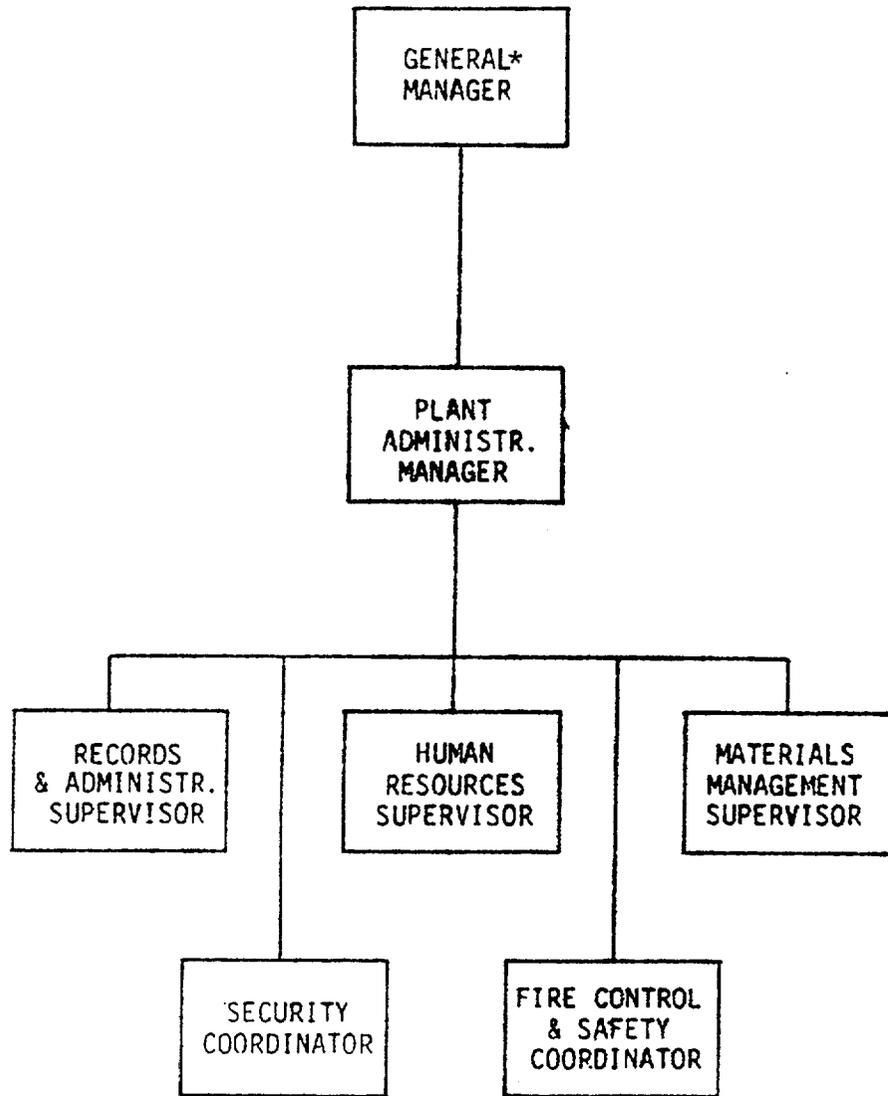


+ - Senior Operator License Required

- Operator License Required

* - Onsite Responsibility for Fire Protection Program

Figure 6.2-2A



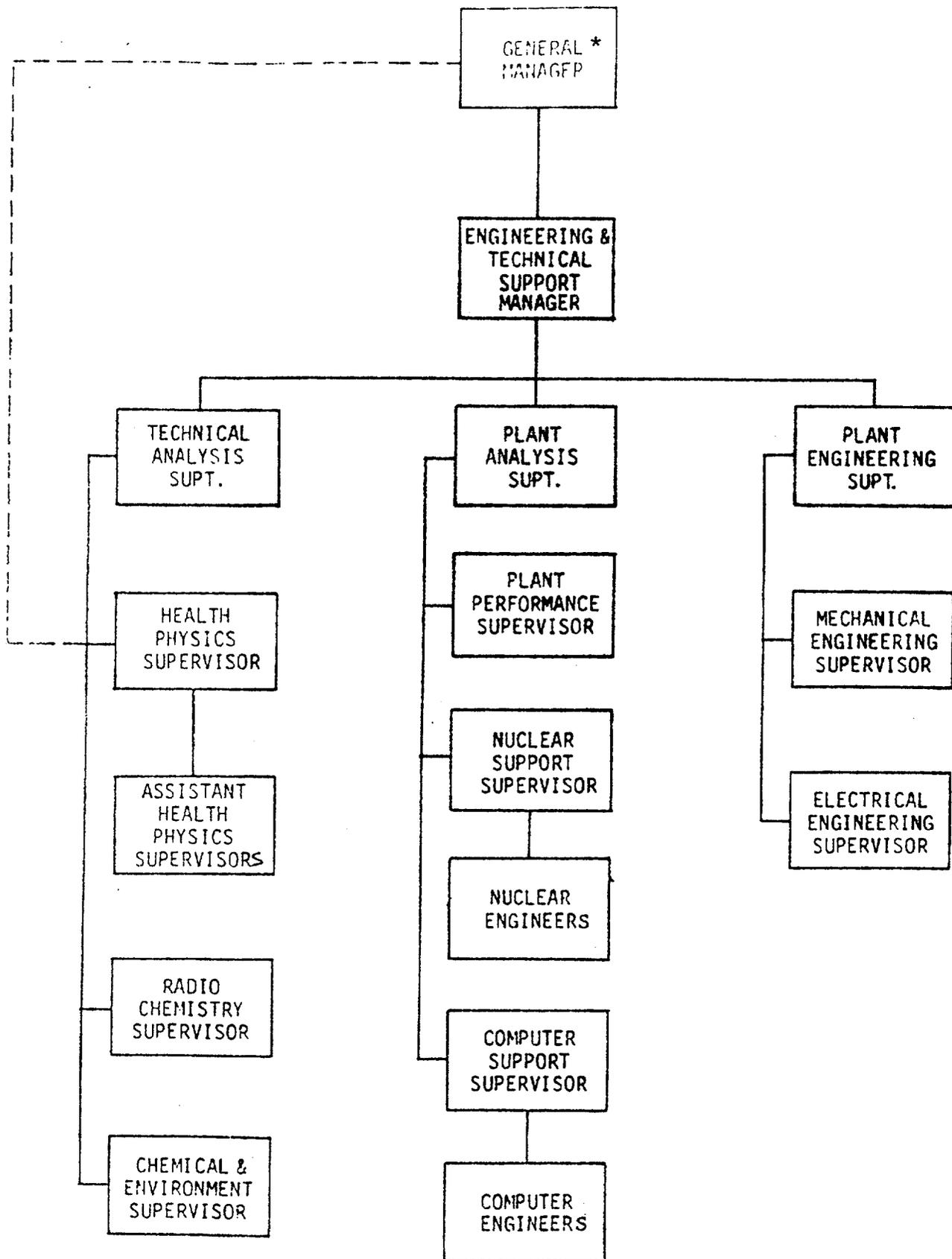
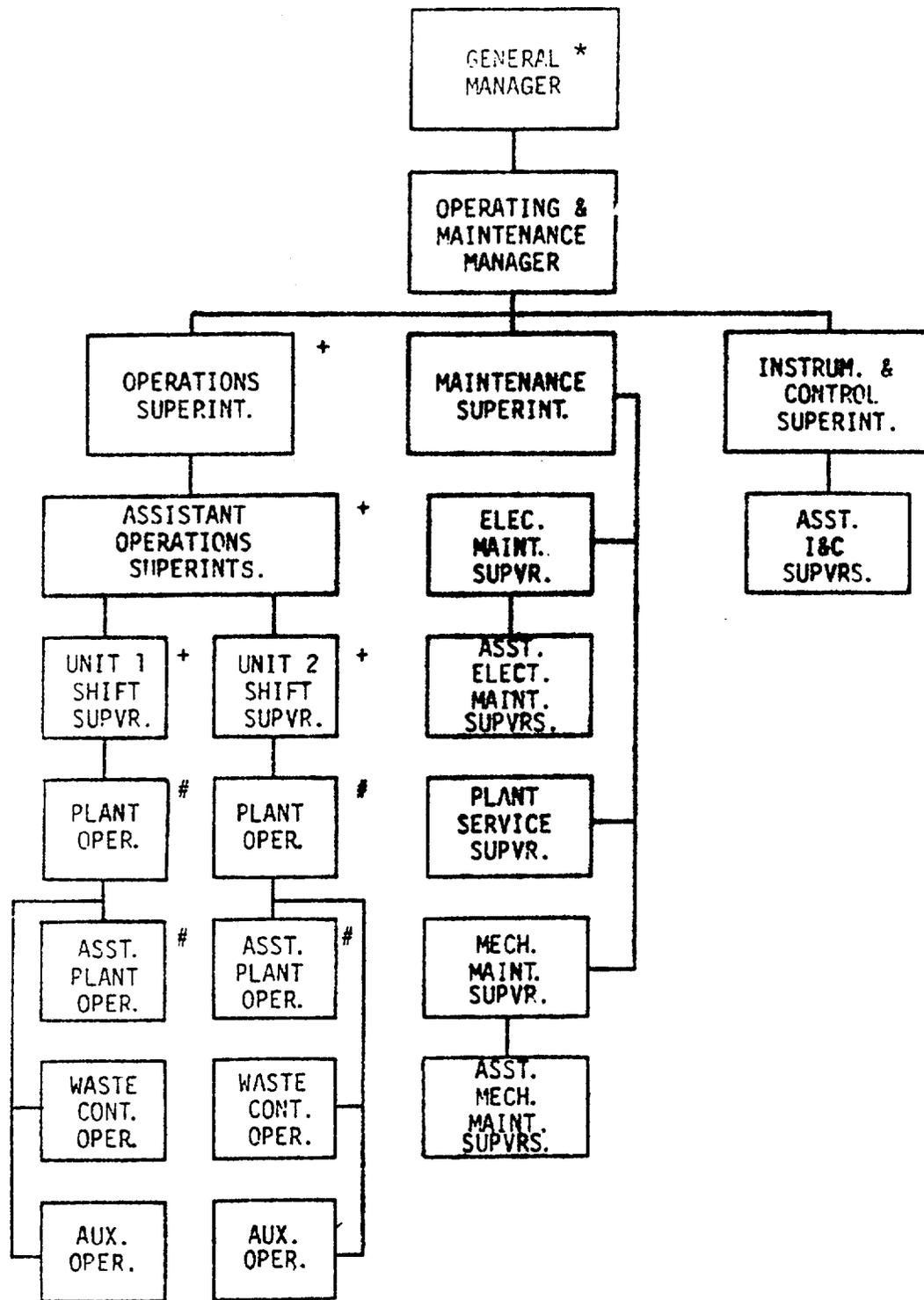


Figure 6.2-2C



Chairman: Operations and Maintenance Manager
Member: Technical Analysis Superintendent
Member: Instrumentation and Controls Superintendent
Member: Operations Superintendent
Member: Maintenance Superintendent
Member: Plant Analysis Superintendent
Member: Health Physics Supervisor

The General Manager shall appoint in writing an acting chairman in the absence of the Operations and Maintenance Manager.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the PSC shall consist of the chairman and three members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and revisions thereto, 2) any other proposed procedures or revisions thereto as determined by the General Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager.
- f. Review of those Reportable Occurrences requiring 24 hour notification of the Commission.

- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations and reports thereon as requested by the General Manager.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the General Manager.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the General Manager.

AUTHORITY

6.5.1.7.1 The Plant Safety Committee shall:

- a. Recommend to the General Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (c) above constitutes an unreviewed safety question.

6.5.1.7.2 In the event of a disagreement between the recommendations of the Plant Safety Committee and the actions contemplated by the General Manager, the course determined by the General Manager to be more conservative will be followed. Records of the disagreement will be sent for review to the Director, Generation Operations, or Manager, Nuclear Operations and the Chairman of the Safety Review Committee by the General Manager on the next working day.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each meeting and copies shall be provided to the Chairman of the Safety Review Committee by the General Manager.

6.5.2 Safety Review Committee (SRC)

FUNCTION

6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry

- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. environmental considerations
- i. other appropriate fields required by the unique characteristics of the nuclear power plant.

COMPOSITION

6.5.2.2 The SRC shall be composed of the:

Chairman: Director, Generation Operations
 Member: Director, Generation Technology
 Member: Director, Technical and Environmental Services
 Member: Manager of Safety
 Member: Arkansas Nuclear One General Manager
 Member: Manager, Technical Analysis
 Member: Arkansas Nuclear One Plant Analysis Superintendent
 Member: Director, Generation Engineering
 Member: Radiation and Health Physics Consultant
 Member: Nuclear Safety Consultant

In his absence, the Chairman shall appoint an Acting Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet a least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or his designated alternate and four members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the SRC or the Executive Director, Generation & Construction (EDG&C).

6.5.2.9 Special Inspections and Audits

- A. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

AUTHORITY

- 6.5.2.10 The SRC shall report to and advise the Executive Director, Generation and Construction (EDG&C) within 30 days following each meeting.

RECORDS

- 6.5.2.11 Records of SRC activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Executive Director, Generation & Construction (EDG&C) within 30 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g and h above, shall be prepared, approved and forwarded to the Executive Director, Generation & Construction, (EDG&C) within 30 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Director, Generation & Construction (EDG&C) and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for Reportable Occurrences:
- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.12.
 - b. Each Reportable Occurrence requiring 24 hour notification to the Commission shall be reviewed by the PSC and submitted to the SRC and the Manager, Nuclear Operations by the General Manager.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least hot shutdown within one hour.
- b. The Nuclear Regulatory Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.36 and Specification 6.12.3.1

6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:
 - a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
 - b. Refueling operations.
 - c. Surveillance and test activities of safety related equipment.
 - d. Security Plan implementation.
 - e. Emergency Plan implementation.
 - f. Fire Protection Program implementation.
- 6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.
- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
 - a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 - c. The change is documented, reviewed by the PSC and approved by the General Manager within 14 days of implementation.

Effective January 19, 1975, activities under the U. S. Atomic Energy Commission regulatory program were assumed by the U. S. Nuclear Regulatory Commission in accordance with the Energy Reorganization Act of 1974. Any references to the Atomic Energy Commission (AEC) contained herein should be interpreted as Nuclear Regulatory Commission (NRC).

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2.2 Hydraulic

2.2.1 Intake Velocity

A study will be undertaken as described in Section 4.1.2 to determine means of limiting fish impingement on the traveling water screens.

2.2.2 Discharge Velocity

Not Applicable.

2.2.3 Flow Rate Restrictions

Not Applicable.

2.2.4 Reservoir Drawdown

Not Applicable.

2.3 Chemical

Objective (General)

To protect the local biota from lethal and sublethal effects or chemical discharges. To assure that the most sensitive use of the receiving medium by human populations is protected. To minimize degradation of the quality of the receiving medium.

Specification (General)

All plant chemical discharges except that from the plant sanitary system shall be diluted by the plant circulating water during release to assure that the stated objective can be achieved. No release of demineralizer waste shall be made without a dilution equivalent to one-half (approximately 383,000 gpm) the full flow of the Unit 1 circulating water pumps.

Any limitation on the quality of plant effluents and requirements for monitoring the same imposed under conditions of the Federal Water Pollution Control Act shall be a part of these Technical Specifications. All reports to State or Federal agencies regarding compliance with any such limitation shall also be provided to NRC as described in Specification 5.4.

2.3.1 Biocides

Specification

a. Chlorine (Circulating Water System)

Chlorination of condenser cooling water shall be intermittent (1 to 2 hours each day or as may be necessary). Total available chlorine residual in the plant effluent shall be less than 0.1 mg/l. If the total available chlorine residual in the discharge canal exceeds 0.1 mg/l, the chlorine feed rate shall be reduced to a rate at which this specification can be met.

5.0 ADMINISTRATIVE CONTROLS

5.1 Responsibility

Corporate responsibility for implementation of the Environmental Technical Specifications, and for assuring that station operations are controlled to provide protection for the environment has been assigned to the Executive Director of Generation and Construction.

The ANO General Manager, through the Engineering and Technical Support Manager, and Technical Analysis Superintendent shall be responsible for compliance with the Environmental Technical Specifications at the plant level.

The Manager of Technical Analysis shall be responsible for radiological analysis of environmental samples.

5.2 Organization

Figure 5-1 shows the organization chart at both plant and corporate levels relative to environmental matters.

5.3 Review

5.3.1 Plant Safety Committee

The Plant Safety Committee (PSC) shall be responsible for review of the following:

- a. Proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- b. Proposed written procedures, as described in Specification 5.5, and proposed changes thereto which affect the plant's environmental impact.

- c. Proposed changes or modifications to plant systems or equipment which would affect the plant's environmental impact.
- d. Results of the Environmental Monitoring Programs.
- e. Investigation of all reported instances of violations of Environmental Technical Specifications. Where investigation warrants, instances shall be evaluated and recommendations formulated to prevent recurrence.

5.3.2 Safety Review Committee

The Safety Review Committee (SRC) shall be responsible for review of the following:

- a. The environmental evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under Section 5.7.3, to verify that such actions did not constitute an unreviewed environmental question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed environmental question as defined in Section 5.7.3.B.
- c. Proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- d. Results of the Environmental Monitoring Programs.
- e. Investigation of all reported instances of violations of Environmental Technical Specifications.

5.4

State and Federal Permits and Certificates

Copies of reports to federal and state agencies regarding compliance of limitation on quality of liquid effluent from ANO-1 shall be sent to Director of Regional Inspection and Enforcement Office (cc Director, NRR).

5.5

Procedures

Detailed written procedures shall be prepared and followed for all activities performed by Arkansas Power and Light involved in carrying out the sampling, instrument calibration, analysis, and actions to be taken when limits are approached or exceeded.

Testing frequency of any alarms shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturers' technical manuals.

Plant standard operating procedures shall include provisions to ensure the plant and all its systems and components are operated in compliance with the limiting conditions for operations established as part of the environmental technical specifications.

5.6 Station Reporting Requirements

5.6.1 Routine Reports

Annual Environmental Operating Report

A single report on the environmental monitoring programs conducted in association with ANO-1 (Docket No. 50-313) and ANO-2 (Docket No. 50-368) operations for the previous calendar year shall be submitted to the NRC by May 1 of each year. The report shall include summaries, analyses, interpretations, and, where appropriate, statistical evaluation of the results of the environmental monitoring and an assessment of the observed impacts of the station operation on the environment. If harmful effects or evidence of irreversible damage are suggested by the monitoring or special programs, the licensee shall provide a more detailed analysis of the data and a proposed course of action to alleviate the problem.

The Annual Report shall also include a summary of:

- 1) All ETS noncompliances and the corrective actions taken to remedy them.
- 2) Changes made to state and federal permits and certificates.
- 3) Changes made to the procedures or design described in accordance with Subsection 5.7.3.
- 4) Changes in ETS.

5.6.2 Nonroutine Reports

A report shall be submitted in the event that a "Limiting Condition for Operation" (Section 2), is exceeded, a report level as specified in Section 3 or 4 is reached, or if an unusual event involving a significant environmental impact occurs. Reports shall be submitted under one of the report schedules described below.

5.6.2.a Prompt Report

Those events specified as prompt report occurrences shall be reported within 24 hours by telephone, telegraph, or facsimile transmission to the NRC followed by a written report to the NRC within 30 days.

5.6.2.b Thirty Day Report

Non-routine events not requiring a prompt report as described in Subsection 5.6.2.a, shall be reported to NRC within 30 days of their occurrence.

5.6.2.c Content of Non-routine Reports

Written 30-day reports and, to the extent possible, the preliminary telephone, telegraph, or facsimile reports shall

- a) describe, analyze, and evaluate the occurrence, including extent and magnitude of the impact, (b) describe the cause of occurrence, and (c) indicate the corrective action (including any significant changes made in procedures) taken to preclude repetition of the occurrence and to prevent similar occurrences involving similar components or systems.

5.7 Changes

5.7.1 Changes in Environmental Technical Specifications

Request for changes in environmental technical specifications shall be submitted to the NRC for review and authorization per 10 CFR 50.90. The request shall include an evaluation of the environmental impact of the proposed changes and a supporting justification.

5.7.2 Changes in Permits and Certifications

Changes or additions to required federal, state, local, and regional authority permits and certificates for the protection of the environment that pertain to the requirements of these ETS shall be reported to the NRC within 30 days. In the event that the licensee initiates or becomes aware of a request for changes to any of the water quality requirements, limits or values stipulated in any certification or permit issued pursuant to Section 401 or 402 of PL 92-500 which is also the subject of an ETS reporting requirement under Section 2, or 4 of this ETS, NRC shall be notified within 30 days. If the proposed change is initiated by the licensee, the notification to the NRC shall include an evaluation of the environmental impact of the revised requirement, limit or value being sought.

5.7.3 Changes in Procedures, Station Design or Operation

- A. The licensee may 1) make changes in the station design and operation, 2) make changes in the procedures described in Subsection 5.5, and 3) conduct tests and experiments not described in accordance with Subsection 5.5, without prior Commission approval, unless the proposed change, test or experiment involves a change in the objectives of the ETS, or an unreviewed environmental question of substantive impact.
- B. A proposed change, test or experiment shall be deemed to involve an unreviewed environmental question if it concerns 1) a matter which may result in a significant increase in any adverse environmental impact previously evaluated in the final environmental impact statement as modified by staff's testimony to the Atomic Safety and Licensing Board, supplements thereto, environmental impact appraisals, or in initial or final adjudicatory decisions; or 2) a significant change in effluents or power level as specified in 10 CFR 51.5(b)(2); or 3) a matter not previously reviewed and evaluated in the documents specified in 1) of this section which may have a significant adverse environmental impact.
- C. The licensee shall maintain records of changes in procedures and in the facility design or operation made pursuant to this Subsection, to the extent that such changes constitute changes

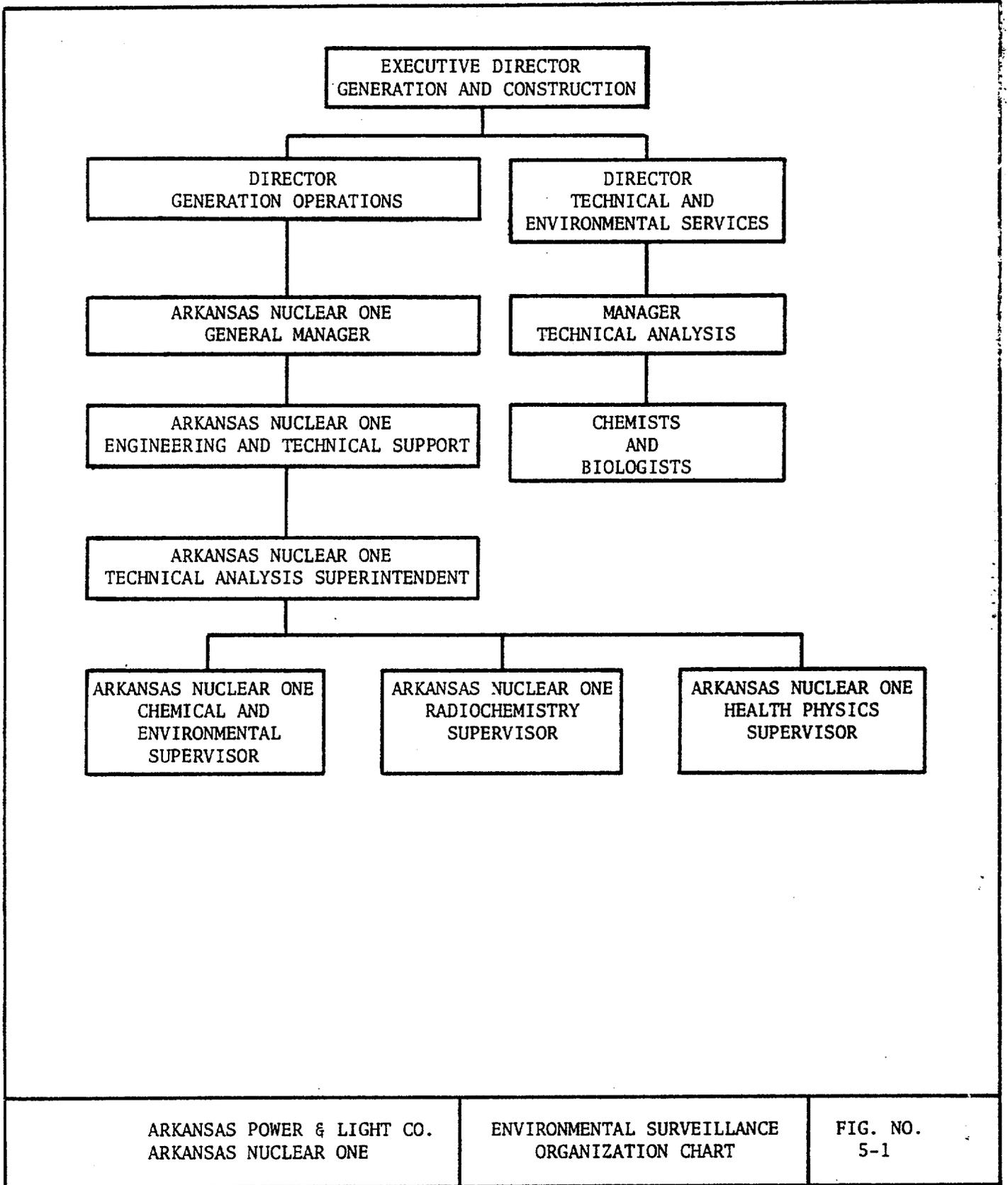
in procedures as described in accordance with Subsection 5.5. The licensee shall also maintain records of tests and experiments carried out pursuant to paragraph "A" of this Subsection. These records shall include a written evaluation which provides the bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question of substantive impact or constitute a change in the objectives of these ETS. The licensee shall furnish to the Commission, annually or at such shorter intervals as may be specified in the license, a report containing descriptions, analyses, interpretations, and evaluations of such changes, tests and experiments.

5.8 Records Retention

5.8.1 Records and logs relative to the following areas shall be retained for the life of the plant:

- a. Records and drawing changes reflecting plant design modifications made to systems and equipment as described in Specification 5.7.3.
- b. Records of environmental surveillance data.
- c. Records to demonstrate compliance with the limiting conditions for operation in Section 2.

5.8.2 All other records and logs relating to the environmental technical specifications shall be retained for five years.



ARKANSAS POWER & LIGHT CO.
ARKANSAS NUCLEAR ONE

ENVIRONMENTAL SURVEILLANCE
ORGANIZATION CHART

FIG. NO.
5-1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated September 22, 1978, as supplemented October 17, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

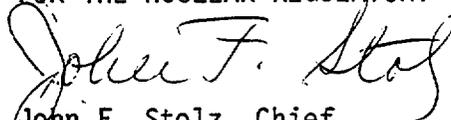
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 5, are hereby incorporated in the license. Arkansas Power and Light Company shall operate the facility in accordance with the Technical Specifications.

The licensee shall be exempted from compliance with the following Appendix A Technical Specification related to the steam generator low water level trip setpoint while conducting the steam generator feedwater system waterhammer testing during the initial startup and power ascension testing program. The value of the steam generator low water level trip setpoint in Item 8(b) of Technical Specification Table 3.3-4 may be reduced, during this testing only, from a value of greater than or equal to 49.4 percent to greater than or equal to 10.0 percent. The licensee shall be exempted from compliance with Appendix A Technical Specification 3.3.3.6 for the Containment Radiation Monitors during Mode 3 operations.

3. This license amendment becomes effective no later than 90 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Light Water Reactors Branch #1
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 13, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Appendix "A"

Pages

6-1
6-2
6-3
6-5
6-6
6-7
6-8
6-13
6-14
6-15

Appendix "B"

Pages

5-1
5-10

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

ARKANSAS POWER & LIGHT COMPANY
ARKANSAS NUCLEAR ONE - UNIT 2

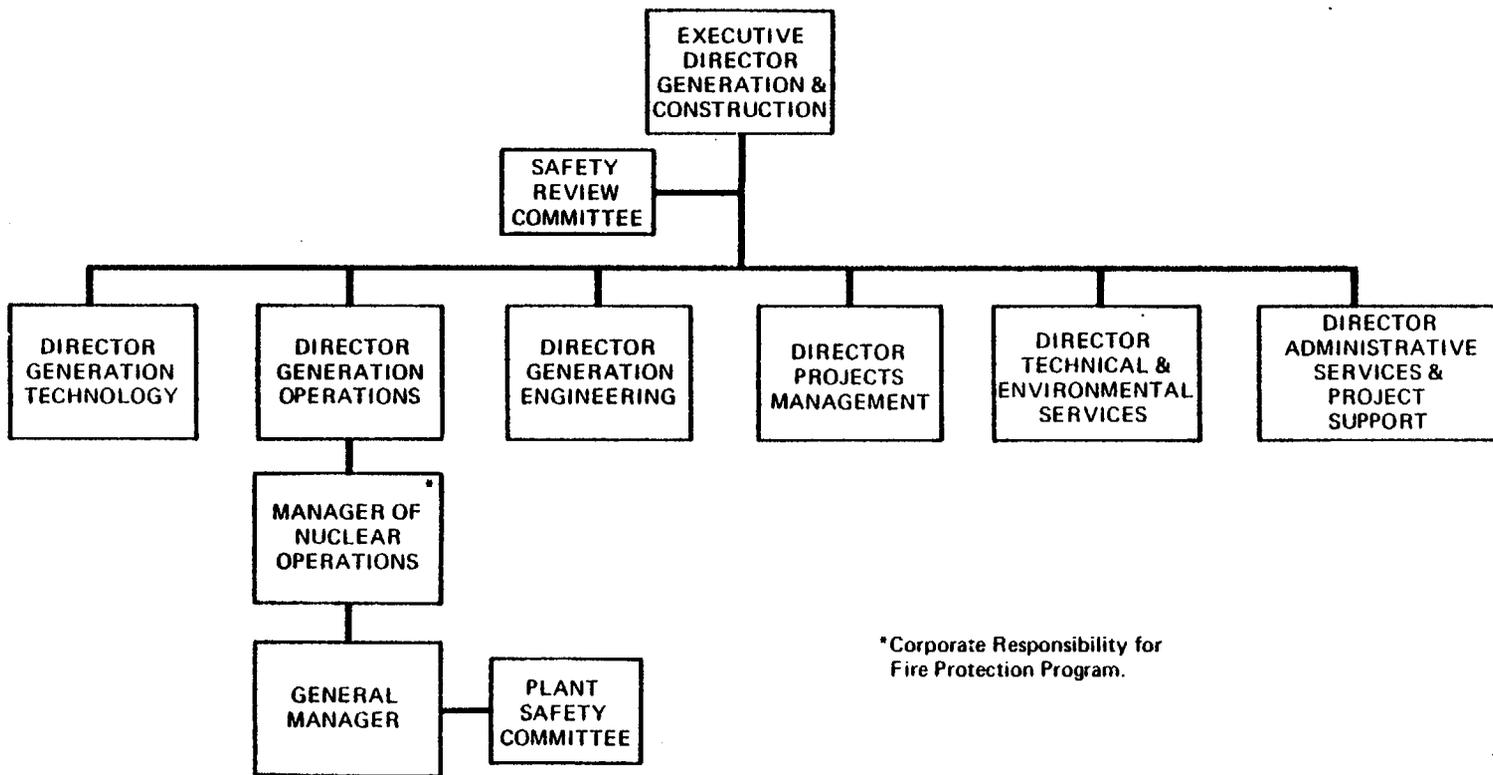


Figure 6.2-1 Management Organization Chart

ARKANSAS POWER & LIGHT COMPANY
ARKANSAS NUCLEAR ONE - UNIT 2

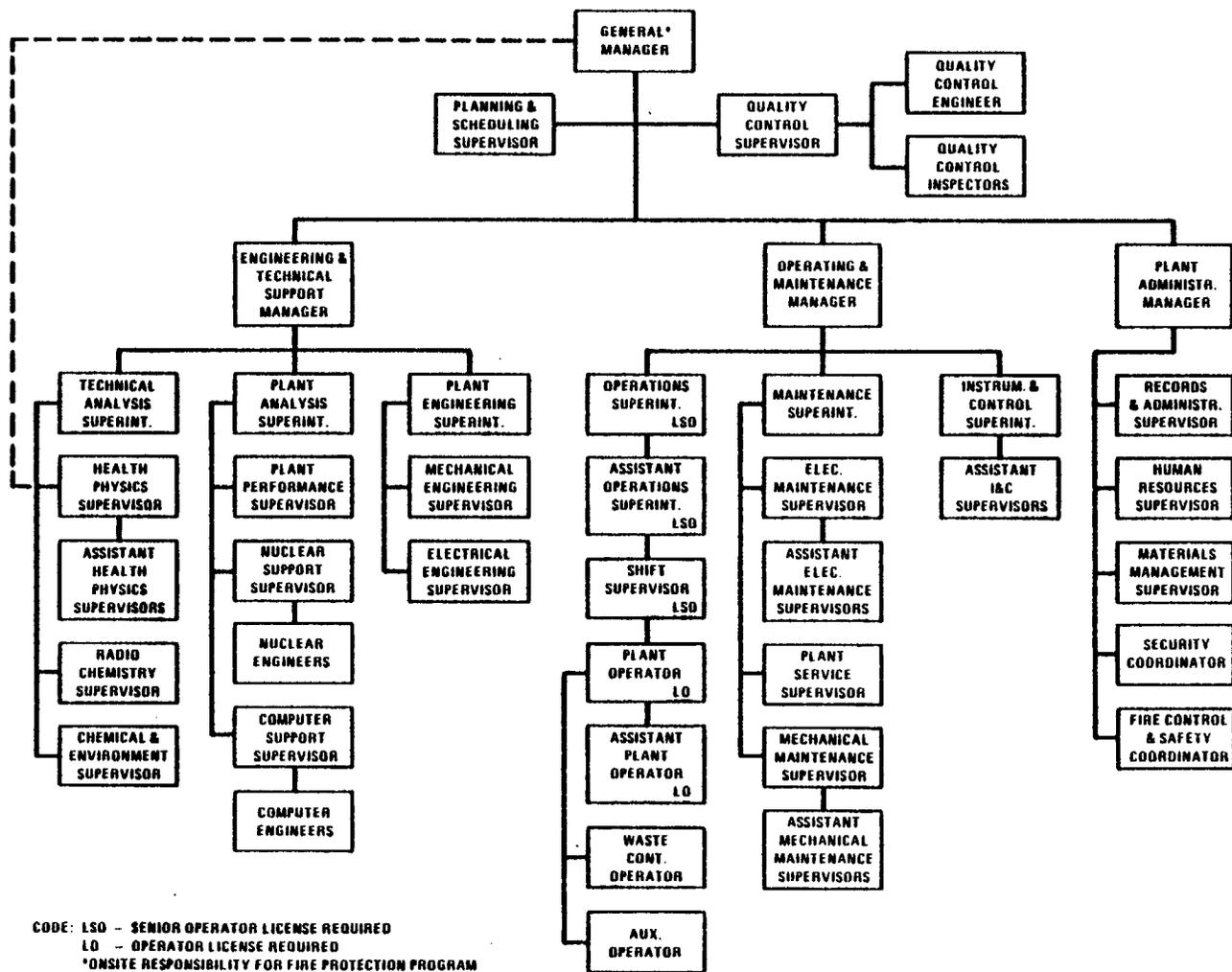


Figure 6.2-2 Functional Organization for Plant Operation

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	2	1

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the General Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code - 1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY COMMITTEE (PSC)

FUNCTION

6.5.1.1 The Plant Safety Committee shall function to advise the General Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the:

Chairman:	Operations and Maintenance Manager
Member:	Operations Superintendent
Member:	Technical Analysis Superintendent
Member:	Maintenance Superintendent
Member:	Instrumentation & Controls Superintendent
Member:	Plant Analysis Superintendent
Member:	Health Physics Supervisor

The General Manager shall appoint in writing an acting chairman in the absence of the Operations and Maintenance Manager.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PCS necessary for the performance of the PSC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and three members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the General Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager and to the Chairman of the Safety Review Committee.

ADMINISTRATIVE CONTROLS

- f. Review of events requiring 24 hour written notification to the Commission.
- g. * Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the General Manager or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Safety Review Committee.

AUTHORITY

6.5.1.7 The Plant Safety Committee shall:

- a. Recommend in writing to the General Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Director, Generation Operations and the Safety Review Committee of disagreement between the PSC and the General Manager; however, the General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the General Manager and Chairman of the Safety Review Committee.

ADMINISTRATIVE CONTROLS

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The SRC shall be composed of the:

Chairman:	Director, Generation Operations
Member:	Director, Generation Technology
Member:	Director, Technical and Environmental Services
Member:	Manager of Safety
Member:	Arkansas Nuclear One General Manager
Member:	Manager, Technical Analysis
Member:	Arkansas Nuclear One Plant Analysis Superintendent
Member:	Director, Generation Engineering
Member:	Radiation and Health Physics Consultant*
Member:	Nuclear Safety Consultant*

*The Radiation and Health Physics Consultant and the Nuclear Safety Consultant shall have an academic degree in engineering or physical science field; and in addition, each shall have a minimum of five years technical experience, of which a minimum of three years shall be in their respective field of expertise.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Operations and to the SRC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Manager, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PSC and approved by the General Manager within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ADMINISTRATIVE CONTROLS

ANNUAL REPORTS^{1/}

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis for the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,^{2/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).

^{1/} A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

^{2/} This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management and Program Analysis, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office of Inspection and Enforcement, no later than the 15th of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 14 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

5.0 ADMINISTRATIVE CONTROLS

5.1 Responsibility

Corporate responsibility for implementation of the Environmental Technical Specifications, and for assuring that station operations are controlled to provide protection for the environment has been assigned to the Executive Director of Generation and Construction.

The ANO General Manager, through the Engineering and Technical Support Manager, and Technical Analysis Superintendent shall be responsible for compliance with the Environmental Technical Specifications at the plant level.

The Manager of Technical Analysis shall be responsible for radiological analysis of environmental samples.

5.2 Organization

Figure 5-1 shows the organization chart at both plant and corporate levels relative to environmental matters.

5.3 Review

5.3.1 Plant Safety Committee

The Plant Safety Committee (PSC) shall be responsible for review of the following:

- a. Proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- b. Proposed written procedures, as described in Specification 5.5, and proposed changes thereto which affect the plant's environmental impact.

- c. Proposed changes or modifications to plant systems or equipment which would affect the plant's environmental impact.
- d. Results of the Environmental Monitoring Programs.
- e. Investigation of all reported instances of violations of Environmental Technical Specifications. Where investigation warrants, instances shall be evaluated and recommendations formulated to prevent recurrence.

5.3.2 Safety Review Committee

The Safety Review Committee (SRC) shall be responsible for review of the following:

- a. The environmental evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under Section 5.7.3, to verify that such actions did not constitute an unreviewed environmental question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed environmental question as defined in Section 5.7.3.B.
- c. Proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- d. Results of the Environmental Monitoring Programs.
- e. Investigation of all reported instances of violations of Environmental Technical Specifications.

- a. Records and drawing changes reflecting plant design modifications made to systems and equipment as described in Specification 5.6.3.
- b. Records of environmental surveillance data.
- c. Records to demonstrate compliance with the limiting conditions for operation in Section 2.

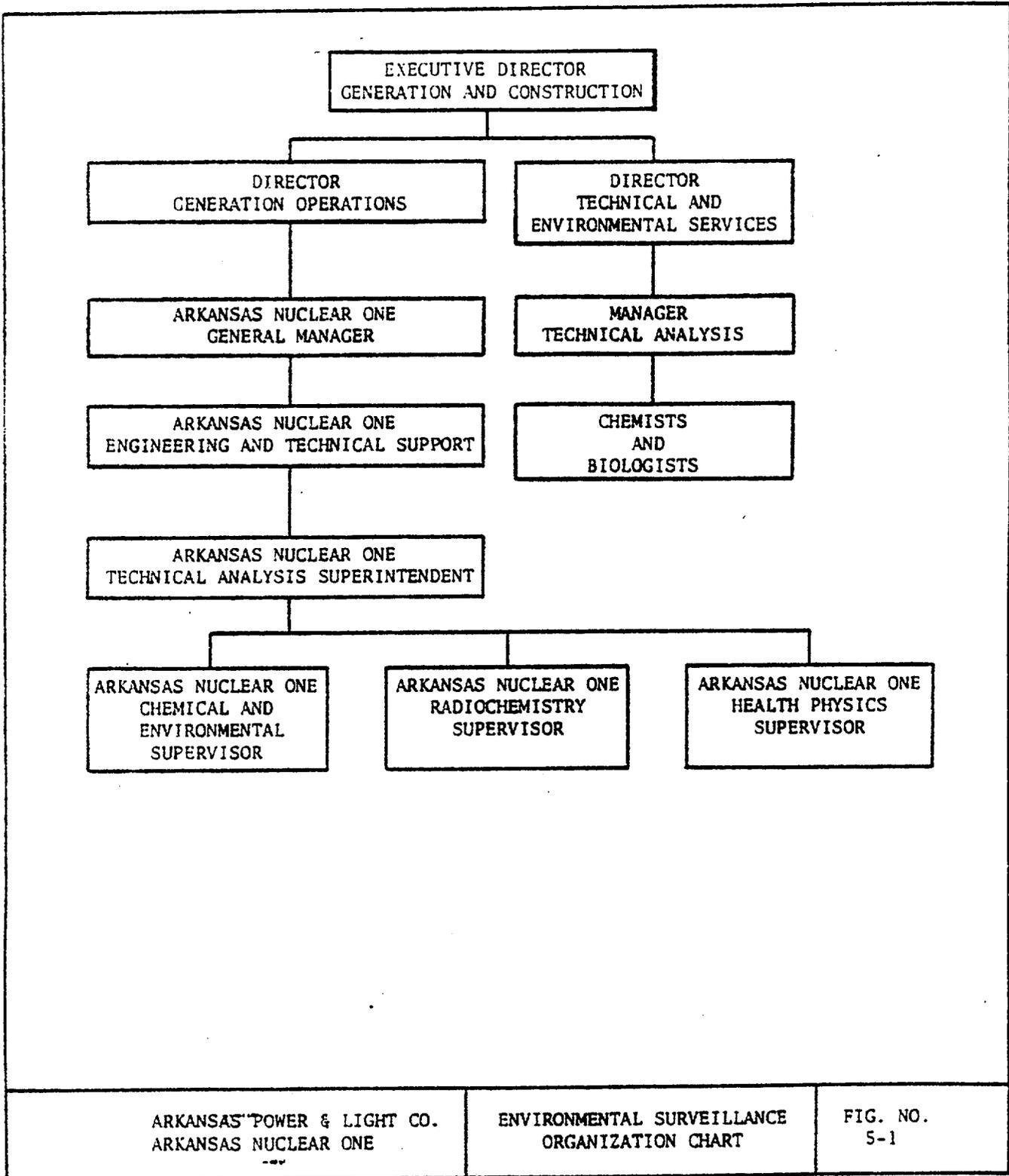
5.8.2 All other records and logs relating to the environmental technical specifications shall be retained for five years.

5.9 Special Requirements

The ANO-2 transmission line rights-of-way have low growing species of cedar, sumac, oak and shrubs as a screen and to assist with erosion control.

Planting of grass and clover shall be carried out to further prevent erosion. Further plantings of game food and clover shall be made in cooperation with landowners and the Arkansas Game and Fish Commission. No herbicides shall be used for land management on transmission line right-of-way.

The grounds in the immediate vicinity of the plant building shall be landscaped. Remaining portions of the plant site shall be allowed to remain in their present wild state with the exception of the area on which the visitors center will be located. This area is located approximately 0.7 miles north east of the Containment on a hill overlooking the plant.



ARKANSAS POWER & LIGHT CO.
ARKANSAS NUCLEAR ONE

ENVIRONMENTAL SURVEILLANCE
ORGANIZATION CHART

FIG. NO.
5-1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 37 AND 5 TO
FACILITY OPERATING LICENSES NOS. DPR-51 AND NPF-6

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNITS NOS. 1&2

DOCKETS NOS. 50-313 & 50-368

Introduction

By letter dated September 22, 1978, supplemented by letter dated October 17, 1978, Arkansas Power and Light Company (the licensee or AP&L) requested amendment of the Technical Specifications (TS), Appendices A and B, appended to Facility Operating Licenses Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1&2 (ANO-1&2). The changes would reflect a change in the licensee's plant staff organization structure and also would make the ANO-1 Administrative Controls Section of the Environmental Technical Specifications (ETS) (Appendix B) consistent with ANO-2. Changes to the TS were also proposed which would modify the implementation schedule of future proposed changes and the responsibilities of the Plant Safety Committee.

Evaluation

We have reviewed the proposed changes to the ANO-1&2 Administrative Controls Sections of the TS. The reorganized plant staff would combine the functions of operations and maintenance under the new position of Operations and Maintenance Manager, and the technical support activities under the new position of Engineering and Technical Support Manager. The Operations and Maintenance Manager, Engineering and Technical Support Manager, and Plant Administration Manager would all report to the General Manager (currently Plant Manager). The Quality Control Supervisor would report directly to the General Manager. New positions of Plant Analysis Superintendent, Plant Engineering Superintendent, and Fire Control and Safety Coordinator would be established. These changes would not reduce the technical resources of the plant staff and should provide better supervision and management of the plant staff.

We conclude that the revised plant staff organization meets the NRC staff's position described in Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," does not decrease the margin of safety, and is acceptable.

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We also find that the proposed change to the ANO-1 Administrative Controls of the ETS makes the plant ETS current and consistent between units and therefore conclude that it is acceptable.

We have reviewed the proposed changes to the TS which would modify the implementation of future proposed changes and the responsibilities of the Plant Safety Committee. We have discussed these proposed changes with the licensee and the licensee has requested that these changes be considered separately in another amendment. We have accepted this request, and the licensee has agreed to provide additional bases to support the proposed changes.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 13, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-313 AND 50-368ARKANSAS POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 37 and 5 to Facility Operating Licenses Nos. DPR-51 and NPF-6, issued to Arkansas Power & Light Company (the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Units Nos. 1 and 2 (ANO-1&2) located in Pope County, Arkansas. The amendments become effective no later than 90 days after the date of issuance.

The amendments modify the ANO-1&2 Technical Specifications dealing with the plant organization structure, and revise the ANO-1 Administrative Controls of the Environmental Technical Specifications to make them current and consistent with ANO-2.

The application for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

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- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the licensee's filing dated September 22, 1978, as supplemented October 17, 1978, (2) Amendment No. 37 to License No. DPR-51 and Amendment No. 5 to License No. NPF-6, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Arkansas Polytechnic College, Russellville, Arkansas. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 13th day of November 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors