

NOV 18 1978

Docket No. 50-368

Mr. William Cavanaugh III
Executive Director of Generation
and Construction
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

SUBJECT: ISSUANCE OF AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE
NO. NPF-6 FOR ARKANSAS NUCLEAR ONE, UNIT 2

By letter dated November 13, 1978 you requested one-time relief from Technical Specification 3.8.1.1.b to allow operation in Modes 3 and 4 with one diesel generator and two offsite power sources operable. You indicated that a mechanical failure of one of the two emergency diesel generators occurred on November 9, 1978 which will prevent you from returning to a pre-critical heat up condition (Modes 3 and 4) from your present cold shutdown operating condition (Mode 5) pursuant to Technical Specification 3.8.1.1.b.

By letters dated November 15 & 17, 1978 you also submitted proposed Technical Specification changes needed in the limiting condition for operation (Section 3.8.1.1) and surveillance requirements (Section 4.8.1.1.2) in order to operate as you have requested.

Based on our review and evaluation, Facility Operating License No. NPF-6 is amended, effective immediately, by making the changes to the Technical Specification described in the attachment to License Amendment No. 6. As noted therein one-time relief has been granted from the Technical Specification 3.8.1.1.b, subject to the changes listed for Technical Specification Section 3.8.1.1 and 4.8.1.1.2 and other restrictions related to making changes in reactivity. The relief will not apply to plant operations in Modes 1 or 2 and, accordingly, both diesel generators will be required to be in operable status before the plant operation can proceed beyond the presently authorized operational Mode 3.

We have determined that Amendment No. 6 does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact appraisal need not be prepared in connection

781208003

Const. 11
P 50

OFFICE	with the issuance of this amendment.				
SURNAME					
DATE					

Mr. William Cavanaugh III

- 2 -

NOV 18 1978

Copies of the license amendment, FEDERAL REGISTER Notice, and Safety Evaluation for this Technical Specification change are enclosed.

Sincerely,



John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Enclosures:

1. Amendment No. 6 to Facility
Operating License No. NPF-6
2. FEDERAL REGISTER Notice
3. Safety Evaluation Supporting
Amendment No. 6

ccs w/enclosures:

See page 3

OFFICE	LWR 1	DOR	DSS	OELD	LWR 1
SURNAME	Edgerton/red	DBrinkman	FRosa	CWoodhead	JStolz
DATE	11/17/78	11/17/78	11/18/78	11/17/78	11/18/78

Mr. William Cavanaugh III

NOV 18 1978

cc: Mr. Daniel H. Williams
Manager, Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Philip K. Lyon, Esq.
House, Holms & Jewell
1550 Tower Building
Little Rock, Arkansas 72203

Mr. C. W. Reed, Project Engineer
Bechtel Power Corporation
San Francisco, California 94119

Mr. Fred Sernatinger, Project Manager
Combustion Engineering, Inc.
1000 Prospect Hill Road
Windsor, Connecticut 06095

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
C-E Power Systems
Combustion Engineering, Inc.
4853 Cordell Avenue, Suite A-1
Bethesda, Maryland 20014

Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Director, Bureau of Environmental
Health Services
4815 West Markham Street
Little Rock, Arkansas 72201

Attorney General
Justice Building
Little Rock, Arkansas 72201

Mr. Bruce Blanchard
Environmental Projects
Review
Department of the Interior
Room 4256
18th and C Street; N. W.
Washington, D. C. 20240

U. S. Environmental Protection
Agency
ATTN: Ms. F. Munter
Office of Federal Activities
Room W-535, Waterside Mall
401 M Street, S. W.
Washington, D. C. 20460

OFFICE >

SURNAME >

DATE >

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

FACILITY OPERATING LICENSE

Amendment No. 6
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) having found that:
 - A. The issuance of this license amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the amended Facility Operating License NO. NPF-6 is hereby amended by changing the Technical Specifications as indicated in the attachment to this license amendment.

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A & B as revised through Amendment No. 6, are hereby incorporated in license NPF-6. Arkansas Power and Light Company shall operate the facility in accordance with the Technical Specifications except for the following specific exemptions.

7812080023

OFFICE >						
SURNAME >						
DATE >						

The licensee shall be exempted from compliance with the following Appendix A Technical Specification related to the steam generator low water level trip setpoint while conducting the steam generator feedwater system waterhammer testing during the initial startup and power ascension testing program. The value of the steam generator low water level trip setpoint in Item 8(b) of Technical Specification Table 3.3-4 may be reduced, during this testing only, from a value of greater than or equal to 49.4 percent to greater than or equal to 10.0 percent. The licensee shall be exempted from compliance with Appendix A Technical Specification 3.3.3.6 for the Containment Radiation Monitors during Mode 3 operations.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance: 18 Nov 1978

OFFICE	LWR 1	DOR	DSS J.R.	OELD E.R.	LWR J
SURNAME	E.H. / red	DBrinkman	FRosa	Woodhead	JStolz
DATE	11/17/78	11/17/78	11/18/78	11/ /78	11/18/78

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

The requirements of Technical Specification 3.8.1.1.b and 4.8.1.1.2 related to alternating current (a-c) power source operation are temporarily changed on a one-time basis to the requirements described below. These revised Technical Specification requirements expire when the Commission has made a written determination that both diesel generators are operational, but not later than three weeks from the date of issuance of this license amendment.

LIMITING CONDITION FOR OPERATION

3.8.1.1

b. One independent diesel generator with:

1. A day fuel tank containing a minimum volume of 280 gallons of fuel (equivalent to 50% of indicated tank volume),
2. A separate fuel storage system containing a minimum volume of 22,500 gallons of fuel (equivalent to 100% of indicated tank level), and
3. A separate fuel transfer pump.

APPLICABILITY: MODES 3 and 4.

a. Delete

- b. With one offsite circuit of the above required a-c electrical power sources inoperable, demonstrate the OPERABILITY of the remaining a-c source by performing Surveillance Requirements 4.8.1.1.1.a and 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter, restore the inoperable sources to OPERABLE status within 12 hours or be in COLD SHUTDOWN within the following 30 hours.
- c. With two of the above required offsite a-c circuits inoperable, demonstrate the OPERABILITY of the diesel generator by performing Surveillance Requirement 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter, unless the diesel generator is already operating; restore at least one of the inoperable offsite sources to OPERABLE status within 4 hours and restore at least two offsite circuits to OPERABLE status within 12 hours from time of initial loss or be in COLD SHUTDOWN within the following 30 hours.

OFFICE: ARKANSAS - UNIT 2

SURNAME >

DATE >

ELECTRICAL POWER SYSTEMS

ACTION (Continued)

- d. With the above required diesel generator inoperable, demonstrate the OPERABILITY of two offsite a-c circuits by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; restore the inoperable diesel generator to OPERABLE status within 2 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.2 The diesel generator shall be demonstrated Operable ~~plant operational modes~~ *sfh*

- a. On a frequency of once every 3 days by:

In addition, the relief from Technical Specification 3.8.1.1.b is subject to the following provisions that the licensee shall:

- (1) Verify at least once per 24 hours that the reactor coolant system boron concentration is equal to or greater than 1731 parts per million.
- (2) Secure all deboration paths. In addition during the time of relief verify the lineups at least once per eight hours.
- (3) Verify that all control element assemblies not being actively tested are fully inserted and deenergized. In addition during the time of relief the control element assemblies not being actively tested shall be verified to be fully inserted and deenergized at least once per eight hours.

OFFICE						
SURNAME						
DATE						



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

FACILITY OPERATING LICENSE

Amendment No. 6
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) having found that:
 - A. The issuance of this license amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the amended Facility Operating License NO. NPF-6 is hereby amended by changing the Technical Specifications as indicated in the attachment to this license amendment.

2.C.(2) Technical Specifications

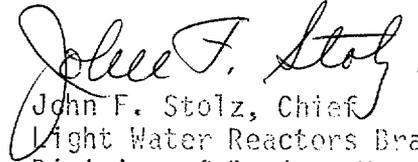
The Technical Specifications contained in Appendices A & B as revised through Amendment No. 6, are hereby incorporated in license NPF-6. Arkansas Power and Light Company shall operate the facility in accordance with the Technical Specifications except for the following specific exemptions.

7812080023

The licensee shall be exempted from compliance with the following Appendix A Technical Specification related to the steam generator low water level trip setpoint while conducting the steam generator feedwater system waterhammer testing during the initial startup and power ascension testing program. The value of the steam generator low water level trip setpoint in Item 8(b) of Technical Specification Table 3.3-4 may be reduced, during this testing only, from a value of greater than or equal to 49.4 percent to greater than or equal to 10.0 percent. The licensee shall be exempted from compliance with Appendix A Technical Specification 3.3.3.6 for the Containment Radiation Monitors during Mode 3 operations.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance: *18 Nov 1978*

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

The requirements of Technical Specification 3.8.1.1.b and 4.8.1.1.2 related to alternating current (a-c) power source operation are temporarily changed on a one-time basis to the requirements described below. These revised Technical Specification requirements expire when the Commission has made a written determination that both diesel generators are operational, but not later than three weeks from the date of issuance of this license amendment.

LIMITING CONDITION FOR OPERATION

3.8.1.1

- b. One independent diesel generator with:
 - 1. A day fuel tank containing a minimum volume of 280 gallons of fuel (equivalent to 50% of indicated tank volume),
 - 2. A separate fuel storage system containing a minimum volume of 22,500 gallons of fuel (equivalent to 100% of indicated tank level), and
 - 3. A separate fuel transfer pump.

APPLICABILITY: MODES 3 and 4.

- a. Delete
- b. With one offsite circuit of the above required a-c electrical power sources inoperable, demonstrate the OPERABILITY of the remaining a-c source by performing Surveillance Requirements 4.8.1.1.1.a and 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter, restore the inoperable sources to OPERABLE status within 12 hours or be in COLD SHUTDOWN within the following 30 hours.
- c. With two of the above required offsite a-c circuits inoperable, demonstrate the OPERABILITY of the diesel generator by performing Surveillance Requirement 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter, unless the diesel generator is already operating; restore at least one of the inoperable offsite sources to OPERABLE status within 4 hours and restore at least two offsite circuits to OPERABLE status within 12 hours from time of initial loss or be in COLD SHUTDOWN within the following 30 hours.

ELECTRICAL POWER SYSTEMS

ACTION (Continued)

- d. With the above required diesel generator inoperable, demonstrate the OPERABILITY of two offsite a-c circuits by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; restore the inoperable diesel generator to OPERABLE status within 2 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.2 The diesel generator shall be demonstrated Operable: plant operational Modes 3 and 4.

- a. On a frequency of once every 3 days by:

In addition, the relief from Technical Specification 3.8.1.1.b is subject to the following provisions that the licensee shall:

- (1) Verify at least once per 24 hours that the reactor coolant system boron concentration is equal to or greater than 1731 parts per million.
- (2) Secure all deboration paths. In addition during the time of relief verify the lineups at least once per eight hours.
- (3) Verify that all control element assemblies not being actively tested are fully inserted and deenergized. In addition during the time of relief the control element assemblies not being actively tested shall be verified to be fully inserted and deenergized at least once per eight hours.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-368

ARKANSAS POWER AND LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT 2

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. NPF-6 to Arkansas Power and Light Company for operation of Arkansas Nuclear One, Unit 2 (the facility) located at the licensee's site in Pope County, Arkansas. The amendment is effective as of the date of its issuance.

The amendment provides one-time relief from Technical Specification 3.8.1.1.b to allow non-nuclear heat up operation with only one diesel generator and two offsite power systems available before repairs are completed on the second diesel generator which experienced mechanical failure on November 9, 1978. The basis of granting relief is that the plant has not yet gone critical and so there are no fission products requiring decay heat removal from the reactor core. The relief expires when the Commission has made a written determination that both diesel generators are operable but not later than three weeks from the date of issuance of this license amendment.

The amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this

amendment was not required since the amendment does not involve a

OFFICE >

SURNAME >

DATE >

significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action see (1) the licensee's application for amendment dated November 13, 1978 and submittal of additional information in a letter dated November 15, 1978, (2) Amendment No. 6 to License No. NPF-6, and (3) the Commission's related Safety Evaluation supporting Amendment No. 6 to License No. NPF-6. All of these items are available for public inspection at the Commission's Public Document Room at 1717 H Street, N. W., Washington, D. C. 20555 and the Arkansas Polytechnic College, Russellville, Arkansas 72801.

Dated at Bethesda, Maryland, this *18th* day of November 1978.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

OFFICE	LWR 1	DOR	DSS	OELD	LWR 1
SURNAME	EHilton/red	DBrinkman	FRosa	CWoodhead	JStolz
DATE	11/17/78	11/17/78	11/ /78	11/ /78	11/18/78

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-368

ARKANSAS POWER AND LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT 2

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. NPF-6 to Arkansas Power and Light Company for operation of Arkansas Nuclear One, Unit 2 (the facility) located at the licensee's site in Pope County, Arkansas. The amendment is effective as of the date of its issuance.

The amendment provides one-time relief from Technical Specification 3.8.1.1.b to allow non-nuclear heat up operation with only one diesel generator and two offsite power systems available before repairs are completed on the second diesel generator which experienced mechanical failure on November 9, 1978. The basis of granting relief is that the plant has not yet gone critical and so there are no fission products requiring decay heat removal from the reactor core. The relief expires when the Commission has made a written determination that both diesel generators are operable but not later than three weeks from the date of issuance of this license amendment.

The amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action see (1) the licensee's application for amendment dated November 13, 1978 and submittal of additional information in a letter dated November 15, 1978, (2) Amendment No. 6 to License No. NPF-6, and (3) the Commission's related Safety Evaluation supporting Amendment No. 6 to License No. NPF-6. All of these items are available for public inspection at the Commission's Public Document Room at 1717 H Street, N. W., Washington, D. C. 20555 and the Arkansas Polytechnic College, Russellville, Arkansas 72801.

Dated at Bethesda, Maryland, this 18th day of November 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 6 TO NPF-6

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-368

INTRODUCTION

By letter dated November 13, 1978 the licensee requested one-time relief from Technical Specification (T.S.) 3.8.1.1.b for Arkansas Nuclear One, Unit 2 (ANO-2) to allow operation in Modes 3 and 4 with one diesel generator and two offsite power sources operable before repairs are completed on the second diesel generator which experienced mechanical failure on November 9, 1978. In addition, the licensee submitted in a letter dated November 15 & 17, 1978, proposed Technical Specification changes needed in the limiting conditions of operation (Section 3.8.1.1) and surveillance requirements (Section 4.8.1.1.2) in order to operate as requested.

DISCUSSION AND EVALUATION

The licensee states that the relief from the Technical Specification 3.8.1.1.b as noted above will have no effect on the health and safety of the public since the ANO-2 reactor has not yet been critical. Therefore, there is no decay heat to remove from the core and no safety significance when returning to a heated up condition in the presently authorized Modes 3 and 4 from a cold shutdown condition.

We have reviewed and evaluated the licensee's requests and evaluation. We agree with the licensee that the plant can operate safely with only one diesel generator and two offsite power sources until the repairs have been completed on the second diesel generator so that both diesel generators are operational subject to interim changes to the Technical Specifications noted below. The revised Technical Specification requirements expire when the Commission has made written determination that both diesel generators are operational, but not later than three weeks from the date of issuance of this license amendment. Our conclusion is based on:

1. The ANO-2 reactor has not yet been critical and therefore contains no fission product sources requiring decay heat removal from the reactor core.
2. Interim changes to the Technical Specifications during the relief period shall require that the licensee shall:
 - a. Verify at least once per 24 hours that the reactor coolant system boron concentration is equal to or greater than 1731 parts per million.

1812080025

OFFICE						
SURNAME						
DATE						

- b. Secure all deboration paths. In addition during the time of repairs verify the lineups at least once per eight hours.
- c. Verify that all control element assemblies not being actively tested are fully inserted and deenergized. In addition during the time of relief the control element assemblies not being actively tested shall be verified to be fully inserted and deenergized at least once per eight hours.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact and/or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Dated: 18 November 1978

SEE PREVIOUS YELLOW FOR PREVIOUS CONCURRENCES

OFFICE	LWR 1 EH/ton/red	DOR DBrinkman*	DSS FRosa	OELD CWoodhead	LWR 1 JStolz	
SURNAME	ton/red	Brinkman	Rosa	Woodhead	Stolz	
DATE	11/ 78	11/ 78	11/ 78	11/ 78	11/ 78	

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 6 TO NPF-6

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-368

INTRODUCTION

By letter dated November 13, 1978 the licensee requested one-time relief from Technical Specification (T.S.) 3.8.1.1.b for Arkansas Nuclear One, Unit 2 (ANO-2) to allow operation in Modes 3 and 4 with one diesel generator and two offsite power sources operable before repairs are completed on the second diesel generator which experienced mechanical failure on November 9, 1978. In addition, the licensee submitted in 4 letters dated November 15, 1978, proposed Technical Specification changes needed in the limiting conditions of operation (Section 3.8.1.1) and surveillance requirements (Section 4.8.1.1.2) in order to operate as requested.

DISCUSSION AND EVALUATION

The licensee states that the relief from the Technical Specification 3.8.1.1.b as noted above will have no effect on the health and safety of the public since the ANO-2 reactor has not yet been critical. Therefore, there is no decay heat to remove from the core and no safety significance when returning to a heated up condition in the presently authorized Modes 3 and 4 from a cold shutdown condition. In addition the licensee is proposing to continuously operate the remaining operable diesel generator while in operational Modes 3 and 4 until such time that both diesel generators are proved to be operable.

We have reviewed and evaluated the licensee's requests and evaluation. We agree with the licensee that the plant can operate safely with only one diesel generator and two offsite power sources until the repairs have been completed on the second diesel generator so that both diesel generators are operational subject to interim changes to the Technical Specifications noted below. Our conclusion is based on:

Commission has made written determination that both diesel generators are operational, but not later than three weeks from the date of issuance of this license amendment.

1. The ANO-2 reactor has not yet been critical and therefore contains no fission products requiring decay heat removal from the reactor core.
2. Interim changes to the Technical Specifications during the relief period shall require that the licensee shall:

Operate the second diesel generator continuously during plant operational Modes 3 and 4.

Verify at least once per 24 hours that the reactor coolant system boron concentration is equal to or greater than 1731 parts per million.

OFFICE →					
SURNAME →					
DATE →					

7812080029

- c. Secure all deboration paths. In addition during the time of repairs verify the lineups at least once per eight hours.
- d. Verify that all control element assemblies not being actively tested are fully inserted and deenergized. In addition during the time of relief the control element assemblies not being actively tested shall be verified to be fully inserted and deenergized at least once per eight hours.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact and/or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Dated:

subject to minor corrections

OFFICE	LWR 1	DOR	DSS	OELD	LWR 1
SURNAME	EPY ton/red	DBrinkman	FRosa	Woodhead	JStolz
DATE	11/17/78	11/17/78	11/19/78	11/17/78	11/18/78



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555
November 30, 1978

DISTRIBUTION:
Docket
LWR #1 Rdg
EGHylton
LEngle

Docket No. 50-368

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: NOTICE OF ISSUANCE OF AMENDMENT NO. 6 TO OPERATING LICENSE NO.
NPF-6 FOR ARKANSAS NUCLEAR ONE, UNIT 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (15) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: _____

of the package

P.S. An extra copy/is enclosed for the NRC PDR.

Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	LWR #1					
SURNAME →	EGHylton					
DATE →	11/30/78					

November 18, 1978

ARKANSAS NUCLEAR ONE, UNIT 2 OPERATING LICENSE NO. NPF-6, AMENDMENT NO. 6

Distribution

Docket File 
NRC PDR
Local PDR
LWR #1 File
Attorney, ELD
R. S. Boyd
D. B. Vassallo
J. F. Stolz
L. Engle
E. Hylton (5)
F. J. Williams
D. Crutchfield
Lana Cobb
IE (5)
N. Dube, MPA
M. Jinks, OA (4)
R. Diggs, ADM
V. A. Moore, DSE
R. C. DeYoung, DSE
R. H. Vollmer, DSE
M. L. Ernst, DSE
R. P. Denise, DSE
R. J. Mattson, SS
J. P. Knight, SS
D. F. Ross, SS
R. L. Tedesco, SS
B. Scharf, ADM (15 copies)
D. Skovholt
I. Dinitz
A. Toalston, AIG
H. Bristow, NMSS
V. Stello, OR
B. Grimes, OR
J. McGough, OR
D. Eisenhut, OR
W. Pasciak, OR
F. S. Echols, EP
M. Slater, EP
F. Rosa
D. Brinkman
R. Ingram
DOR Project Manager

bcc: J. R. Buchanan, NSIC
T. B. Abernathy, TIC
A. Rosenthal, ASLAB
J. Yore, ASLBP
ACRS (16)

P