DOCKET FILE

Docket No. 50-416 March 10, 1992

Mr. William T. Cottle
Vice President, Operations GGNS
Entergy Operations, Inc.
Post Office Box 756
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: CORRECTION TO AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M81916)

On February 25, 1992, the Commission issued Amendment No. 91 to Facility Operating License No. NPF-29 to Grand Gulf Nuclear Station, Unit 1. The amendment deleted a Technical Specification (TS) requirement to perform a daily surveillance verifying the measured recirculation system drive flow to be less than or equal to the established drive flow for a given flow control valve position.

Correction is being made to delete footnote "h" only from TS page 3/4 3-8. The corresponding overleaf page is also provided to maintain document completeness. Please accept our apologies for any inconvenience this may have caused you.

Sincerely,

ORIGINAL SIGNED BY

Paul W. O'Connor, Senior Project Manager
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Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
TS pages 3/4 3-8

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Office of Nuclear Reactor Regulation

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TABLE 4.3.1.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION(a)	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
1. Intermediate Range Monitors:				
a. Neutron Flux - High	S/U,S,(b) S	S/U, W W	R R	2 3, 4, 5
b. Inoperative	NA	W	NA	2, 3, 4, 5
2. Average Power Range Monitor:(f)				
a. Neutron Flux - High, Setdown	S/U,S,(b) S	S/U, W W	SA SA	2 3, 5
b. Flow Biased Simulated Thermal Power - High	S	Q	W(d)(e), SA, R(i)	1
c. Neutron Flux - High	S	Q	W(d), SA	1
d. Inoperative	NA	Q	NA	1, 2, 3, 5
3. Reactor Vessel Steam Dome Pressure - High	S	Q	R(g)	1, 2(j)
4. Reactor Vessel Water Level - Low, Level 3	S	Q	R(g)	1, 2
5. Reactor Vessel Water Level - High, Level 8	S	Q	R(g)	1
6. Main Steam Line Isolation Valve - Closure	NA	Q	R	1
7. Main Steam Line Radiation - High	S	Q	R	1, 2(j)
8. Drywell Pressure - High	S	Q	R(g)	1, 2(k)

TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
9. Scram Discharge Volume Water Level - High				
a. Transmitter/Trip Unit	S	Q	R(g)	1, 2, 5(l)
b. Float Switch	NA	Q	R	1, 2, 5(l)
10. Turbine Stop Valve - Closure	S	Q	R(g)	1
11. Turbine Control Valve Fast Closure Valve Trip System Oil Pressure - Low	S	Q	R(g)	1
12. Reactor Mode Switch Shutdown Position	NA	R	NA	1, 2, 3, 4, 5
13. Manual Scram	NA	W	NA	1, 2, 3, 4, 5

(a) Neutron detectors may be excluded from CHANNEL CALIBRATION.

(b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decade during each startup after entering OPERATIONAL CONDITION 2 and the IRM and APRM channels shall be determined to overlap for at least 1/2 decade during each controlled shutdown, if not performed within the previous 7 days.

(c) [DELETED]

(d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER > 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER.

(e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.

(f) The LPRMs shall be calibrated at least once per 1000 MWD/T using the TIP system.

(g) Calibrate trip unit at least once per 92 days.

(h) Deleted.

(i) This calibration shall consist of verifying the 6 ± 1 second simulated thermal power time constant.

(j) Not applicable when the reactor pressure vessel head is unbolted or removed per Specification 3.10.1.

(k) Not applicable when DRYWELL INTEGRITY is not required.

(l) Applicable with any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.