

Mr. William T. Cottle
Vice President, Operations GGNS
Entergy Operations, Inc.
Post Office Box 756
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: ISSUANCE OF AMENDMENT NO.104 TO FACILITY OPERATING LICENSE
NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. 84323)

The Nuclear Regulatory Commission has issued the enclosed Amendment No.104 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment revises the Technical Specifications (TS) in response to your application dated August 19, 1992 as supplemented December 3, 1992.

The amendment modifies TS 4.0.5, Applicability, and TS 3/4.4.3, Reactor Coolant System Leakage. The amendment modifies the TS regarding the licensee's inservice inspection (ISI) program to include a statement of compliance with the NRC staff's position on schedule, methods, personnel and sample expansion given in GL 88-01 and revises leakage monitoring requirements.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Paul W. O'Connor, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No.104 to NPF-29
- 2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

December 23, 1992

Docket No. 50-416

Mr. William T. Cottle
Vice President, Operations GGNS
Entergy Operations, Inc.
Post Office Box 756
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: ISSUANCE OF AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE
NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M84323)

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A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Paul W. O'Connor".

Paul W. O'Connor, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 104 to NPF-29
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. W. T. Cottle
Grand Gulf Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

ENERGY OPERATIONS, INC.

SYSTEM ENERGY RESOURCES, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

MISSISSIPPI POWER AND LIGHT COMPANY

DOCKET NO. 50-416

GRAND GULF NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated August 19, 1992, as supplemented December 3, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-29 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 104, are hereby incorporated into this license. Entergy Operations, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John T. Larkins, Director
Project Directorate IV-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the
Technical Specifications

Date of Issuance: December 23, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 104

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES

3/4 0-3
3/4 4-9
3/4 4-10

INSERT PAGES

3/4 0-3
3/4 4-9
3/4 4-10

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

<u>ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities</u>	<u>Required frequencies for performing inservice inspection and testing activities</u>
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE.
- c. 30 gpm total leakage.
- d. 1 gpm leakage at reactor coolant system pressure of 1050 ± 10 psig from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.
- e. 2 gpm increase UNIDENTIFIED LEAKAGE within any 24-hour period or less (applicable in OPERATIONAL CONDITION 1 only).

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than the limits in b and/or c, above, reduce the leakage rate to within the limits 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN with the following 24 hours.
- c. With any reactor coolant system pressure isolation valve leakage greater than the above limit, isolate high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two close manual or deactivated automatic valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With or more high/low pressure interface valve pressure monitors and/or interlocks inoperable, restore the inoperable monitor(s) and/or interlock(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- e. With any reactor coolant system UNIDENTIFIED LEAKAGE increase greater than 2 gpm within 24-hour period or less, identify the source of leakage increase as not service sensitive Type 304 or 316 austenitic stainless steel within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the drywell atmospheric particulate and gaseous radioactivity at least once per 12 hours,
- b. Monitoring the drywell floor and equipment drain sump level and flow rate at least once per 12 hours,
- c. Monitoring the drywell air coolers condensate flow rate at least once per 12 hours, and
- d. Monitoring the reactor vessel head flange leak detection system at least once per 24 hours.

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit:

- a. At least once per 18 months, and
- b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3

4.4.3.2.3 The high/low pressure interface valves leakage pressure monitors shall be demonstrated OPERABLE with alarm and interlock setpoints per Table 3.4.3.2-2 and Table 3.4.3.2-3 by performance a:

- a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
- b. CHANNEL CALIBRATION at least once per 18 months.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NO. NPF-29
ENERGY OPERATIONS, INC., ET AL.
GRAND GULF NUCLEAR STATION, UNIT 1
DOCKET NO. 50-416

1.0 INTRODUCTION

By letters dated August 8, 1988, July 14, 1989, and April 23, 1990, the licensee (Energy Operations, Inc.) submitted its response to Generic Letter (GL) 88-01, "NRC Position on Intergranular Stress-Corrosion Cracking (IGSCC) in BWR Austenitic Stainless Steel Piping." The NRC staff provided its Safety Evaluation (SE) of the licensee's submittals by letter dated October 1, 1990. In its SE, the staff found the licensee's responses acceptable except for the responses related to the following Technical Specifications (TS) changes requested in GL 88-01:

- (1) Inclusion of a statement in the TS that commits to the NRC staff positions regarding GL 88-01;
- (2) Modification to TS 3.4.3.2.e to specify that reactor coolant system leakage shall be limited to a 2-gpm increase in unidentified leakage in any 24-hour period; and
- (3) Modification to the surveillance requirements in TS 4.4.3.2.1 to specify that primary containment sump flow rate shall be monitored at least once per 12 hours.

By letter dated December 7, 1990, the licensee declined to submit these TS changes. Following the issuance of Supplement 1 to GL 88-01, dated February 4, 1992, and discussions with the NRC staff, the licensee submitted the requested TS changes by letter dated August 19, 1992, as supplemented December 3, 1992. The December 3, 1992, letter provided clarifying information that did not change the initial proposed no significant hazards determination.

2.0 EVALUATION

The licensee has proposed to add the following statement as TS 4.0.5.f:

- f. The inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.

The wording of this proposed statement corresponds to that specified in the NRC staff's Safety Evaluation (SE) of the licensee's submittals, and is therefore acceptable.

The licensee has proposed to modify TS 3.4.3.2 as follows:

Limiting Condition for Operation 3.4.3.2.e

Reactor coolant system leakage shall be limited to 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24-hour period or less (applicable in OPERATIONAL CONDITION 1 only).

Action 3.4.3.2.e

With any reactor coolant system UNIDENTIFIED LEAKAGE increase greater than 2 gpm within any 24-hour period or less, identify the source of leakage increase as not service sensitive Type 304 or 316 austenitic stainless steel within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

The parenthetical note in Limiting Condition for Operation 3.4.3.2.e was added by letter dated December 4, 1992. The proposed limit is more stringent than the existing TS limit. The staff has reviewed the proposed modification and finds it to be consistent with the guidance provided in GL 88-01. It is therefore acceptable.

The licensee has proposed to change the surveillance requirement in TS 4.4.3.2.1.a, b, and c to specify that leakage measurements shall be monitored at least once per twelve (12) hours. The proposed wording for these TS is:

Surveillance Requirement 4.4.3.2.1

The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the drywell atmospheric particulate and gaseous radioactivity at least once per 12 hours,
- b. Monitoring the drywell floor and equipment drain sump level and flow rate at least once per 12 hours,
- c. Monitoring the drywell air coolers condensate flow rate at least once per 12 hours, and ...

This proposed change is consistent with Supplement 1 to GL 88-01 and is therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Mississippi State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 40212). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Rathbun

Date: December 23, 1992