DOCKET FILE

Docket No. 50-416 March 4, 1992

Mr. William T. Cottle
Vice President, Operations GGNS
Entergy Operations, Inc.
Post Office Box 756
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: CORRECTION TO AMENDMENT NO. 89 TO FACILITY OPERATING LICENSE NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M80701)

On February 20, 1992, the Commission issued Amendment No. 89 to Facility Operating License No. NPF-29 to Grand Gulf Nuclear Station, Unit 1. The amendment authorized a one-time extension of the required test interval for overall integrated leak rate tests (Type A tests) as specified in coupling the third Type A test to the plant shutdown for the 10-year Inservice Inspection outage.

Correction is being made to incorporate corrected Technical Specification (TS) overleaf page B 3/4 6-2 to TS B 3/4 6-1. The overleaf page inadvertently omitted changes made with Amendment No. 84. Please accept our apologies for any inconvenience this may have caused you.

Sincerely,
ORIGINAL SIGNED BY

Paul W. O'Connor, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
TS pages B 3/4 6-1
and B 3/4 6-2

cc w/enclosure:
See next page

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WASHINGTON, D. C. 20555

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Paul W.

Paul W. O'Connor
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Office of Nucl

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3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 PRIMARY CONTAINMENT INTEGRITY

PRIMARY CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 11.5 psig, P_a. As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to 0.75 L_a during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

Operating experience with the main steam line isolation valves has indicated that degradation has occasionally occurred in the leak tightness of the valves; therefore the special requirement for testing these valves.

The surveillance testing for measuring leakage rates is consistent with the requirements of Appendix J to 10 CFR 50 with the exception of exemptions granted for testing the airlocks after each opening, and uncoupling the third Type A test of each 10-year service period from the last outage of that period.

3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on PRIMARY CONTAINMENT INTEGRITY and the containment leakage rate given in Specifications 3.6.1.1 and 3.6.1.2. The specification makes allowances for the fact that there may be long periods of time when the air locks will be in a closed and secured position during reactor operation. Only one closed door in each air lock is required to maintain the integrity of the containment. Verification that each air lock door inflatable seal system is OPERABLE by the performance of a local leak-detection test for a period of less than 48 hours is permissible if it can be demonstrated that the leakage rate can be accurately determined for this shorter period. (This is in accordance with Sections 6.4 and 7.6 of ANSI N45.4-1972.)

3/4.6.1.4 MSIV LEAKAGE CONTROL SYSTEM

Calculated doses resulting from the maximum leakage allowance for the main steamline isolation valves in the postulated LOCA situations would be a small fraction of the 10 CFR 100 guidelines, provided the main steam line system from the isolation valves up to and including the turbine condenser remains intact. Operating experience has indicated that degradation has occasionally occurred in the leak tightness of the MSIV's such that the specified leakage requirements have not always been maintained continuously. The requirement for the leakage control system will reduce the untreated leakage from the MSIVs when isolation of the primary system and containment is required.