

March 31, 1987

Docket No.: 50-416

Mr. Oliver D. Kingsley, Jr.  
Vice President, Nuclear Operations  
System Energy Resources, Inc.  
Post Office Box 23054  
Jackson, Mississippi 39205

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Dear Mr. Kingsley:

SUBJECT: CHANGES TO TECHNICAL SPECIFICATIONS REGARDING SAFETY RELIEF VALVES,  
THERMAL OVERLOAD PROTECTION DEVICES AND SECONDARY CONTAINMENT  
ISOLATION VALVES

RE: GRAND GULF NUCLEAR STATION, UNIT 1

The Commission has issued the enclosed Amendment No. 29 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 17, 1986, Items 1, 3 and 4. Item 2 regarding the addition of smoke detectors in the control rod drive repair room is not addressed in this amendment.

This amendment would change the Technical Specifications (TSs) by: (1) deleting the two-minute time limit for closing a stuck open safety relief valve (SRV) and changing the temperature limit of the suppression pool water from 105° to 110°F; (2) adding a requirement to bypass the thermal overload protection device for the motor operated valve in the reactor core isolation cooling (RCIC) turbine bypass line; and (3) changing the nomenclature of a secondary containment isolation valve in the residual heat removal (RHR) discharge line to the liquid radwaste system.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed by

Lester L. Kintner, Project Manager  
BWR Project Directorate No. 4  
Division of BWR Licensing

Enclosures:

- Amendment No. 29 to License No. NPF-29
- Safety Evaluation

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cc w/enclosures:  
See next page

PD#4/PM  
MO'Brien  
3/31/87

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3/31/87

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OGC	PD#4/D
Young	WButler
3/23/87	3/31/87



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

March 31, 1987

Docket No.: 50-416

Mr. Oliver D. Kingsley, Jr.  
Vice President, Nuclear Operations  
System Energy Resources, Inc.  
Post Office Box 23054  
Jackson, Mississippi 39205

Dear Mr. Kingsley:

SUBJECT: CHANGES TO TECHNICAL SPECIFICATIONS REGARDING SAFETY RELIEF VALVES,  
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A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "L L Kintner".

Lester L. Kintner, Project Manager  
BWR Project Directorate No. 4  
Division of BWR Licensing

Enclosures:

1. Amendment No. 29 to License No. NPF-29
2. Safety Evaluation

cc w/enclosures:

See next page

Mr. Oliver D. Kingsley, Jr.  
System Energy Resources, Inc.

Grand Gulf Nuclear Station (GGNS)

cc:

Mr. Ted H. Cloninger  
Vice President, Nuclear Engineering  
and Support  
System Energy Resources, Inc.  
Post Office Box 23054  
Jackson, Mississippi 39205

Mr. C. R. Hutchinson  
GGNS General Manager  
System Energy Resources, Inc.  
Post Office Box 756  
Port Gibson, Mississippi 39150

Robert B. McGehee, Esquire  
Wise, Carter, Child, Steen and Caraway  
P.O. Box 651  
Jackson, Mississippi 39205

The Honorable William J. Guste, Jr.  
Attorney General  
Department of Justice  
State of Louisiana  
Baton Rouge, Louisiana 70804

Nicholas S. Reynolds, Esquire  
Bishop, Liberman, Cook, Purcell  
and Reynolds  
1200 17th Street, N.W.  
Washington, D. C. 20036

Office of the Governor  
State of Mississippi  
Jackson, Mississippi 39201

Mr. Ralph T. Lally  
Manager of Quality Assurance  
Middle South Utilities System  
Services, Inc.  
P.O. Box 61000  
New Orleans, Louisiana 70161

Attorney General  
Gartin Building  
Jackson, Mississippi 39205

Mr. John G. Cesare  
Director, Nuclear Licensing and Safety  
System Energy Resources, Inc.  
P.O. Box 23054  
Jackson, Mississippi 39205

Mr. Jack McMillan, Director  
Division of Solid Waste Management  
Mississippi Department of Natural  
Resources  
Bureau of Pollution Control  
Post Office Box 10385  
Jackson, Mississippi 39209

Mr. R. W. Jackson, Project Engineer  
Bechtel Power Corporation  
15740 Shady Grove Road  
Gaithersburg, Maryland 20877-1454

Alton B. Cobb, M.D.  
State Health Officer  
State Board of Health  
P.O. Box 1700  
Jackson, Mississippi 39205

Mr. Ross C. Butcher  
Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
Route 2, Box 399  
Port Gibson, Mississippi 39150

President  
Claiborne County Board of Supervisors  
Port Gibson, Mississippi 39150

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta Street, N.W., Suite 2900  
Atlanta, Georgia 30323

Mr. James E. Cross  
GGNS Site Director  
System Energy Resources, Inc.  
P.O. Box 756  
Port Gibson, Mississippi 39150



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

MISSISSIPPI POWER & LIGHT COMPANY  
SYSTEM ENERGY RESOURCES, INC.  
SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION  
DOCKET NO. 50-416  
GRAND GULF NUCLEAR STATION, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29  
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that
  - A. The application for amendment by Mississippi Power & Light Company, System Energy Resources, Inc. (formerly Middle South Energy, Inc.) and South Mississippi Electric Power Association, (the licensees) dated October 17, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-29 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 29, are hereby incorporated into this license. System Energy Resources, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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PDR ADOCK 05000416  
P PDR

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

**Original Signed by**

Walter R. Butler, Director  
BWR Project Directorate No. 4  
Division of BWR Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 31, 1987

PD#4/LA  
MO/Butler  
3/30/87

*JK*  
PD#4/PM  
LKintner:ca  
3/20/87

*Up to date copy of SEC license amendment*  
OGC  
*M. Young*  
3/13/87

PD#4/D  
WButler  
3/31/87 *WB*

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Walter R. Butler*  
Walter R. Butler, Director  
BWR Project Directorate No. 4  
Division of BWR Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 31, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 29

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf page(s) provided to maintain document completeness.\*

Remove

3/4 4-5  
3/4 4-6

3/4 6-53  
3/4 6-54

3/4 8-47  
3/4 8-48

Insert

3/4 4-5  
3/4 4-6\*

3/4 6-53\*  
3/4 6-54

3/4 8-47  
3/4 8-48\*

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY VALVES

SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2.1 For the following safety/relief valves:

- a. The safety valve function of at least 7 valves and the relief valve function of at least 6 valves other than those satisfying the safety valve function requirement shall be OPERABLE with the specified lift settings, and
- b. The safety/relief tail-pipe pressure switches for each safety/relief valve shall be OPERABLE.

<u>Number of Valves</u>	<u>Function</u>	<u>Setpoint* (psig)</u>
8	Safety	1165 ± 11.6 psi
6	Safety	1180 ± 11.8 psi
6	Safety	1190 ± 11.9 psi
1	Relief	1103 ± 15 psi
10 <sup>#</sup>	Relief	1113 ± 15 psi
9	Relief	1123 ± 15 psi

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With the safety and/or relief valve function of one or more of the above required safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With one or more safety/relief valves stuck open, provided that suppression pool average water temperature is less than 110°F, take action to close the stuck open relief valve(s); if suppression pool average water temperature is 110°F or greater, place the reactor mode switch in the Shutdown position.
- c. With one or more safety/relief tail-pipe pressure switches inoperable, restore the inoperable switch(es) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With either relief valve function pressure actuation trip system "A" or "B" inoperable, restore the inoperable trip system to OPERABLE status within 7 days; otherwise be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2.1.1 The tail-pipe pressure switch for each safety/relief valve shall be demonstrated OPERABLE with the setpoint verified to be 30 ± 5 psig by performance of a:

\*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.

#Initial opening of 1B21-F051B is 1103 ± 15 psig due to low-low set function.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

---

- a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and a
- b. CHANNEL CALIBRATION at least once per 18 months.\*

4.4.2.1.2 The relief valve function pressure actuation instrumentation shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL FUNCTIONAL TEST, including calibration of the trip unit, at least once per 31 days.
- b. CHANNEL CALIBRATION, LOGIC SYSTEM FUNCTIONAL TEST and simulated automatic operation of the entire system at least once per 18 months.

---

\*The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.

TABLE 3.6.6.2-1 (Continued)

SECONDARY CONTAINMENT VENTILATION SYSTEM AUTOMATIC ISOLATION DAMPERS/VALVES

<u>VALVE FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
Valves (Continued)	
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F064)-(A)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F066)-(A)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F047)-(A)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F063)-(B)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F065)-(B)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F067)-(B)	4
Cond. & Refuel Water Transfer Aux. Bldg. Isol. Valve (P11-F061)-(B)	4
Floor and Equipment Drains System Aux. Bldg. Isol. Valve (P45-F158)-(A)	9
Floor and Equipment Drains System Aux. Bldg. Isol. Valve (P45-F160)-(A)	9
Floor and Equipment Drains System Aux. Bldg. Isol. Valve (P45-F163)-(A&B)	9
Floor and Equipment Drains System Aux. Bldg. Isol. Valve (P45-F159)-(B)	9
Floor and Equipment Drains System Aux. Bldg. Isol. Valve (P45-F161)-(B)	9
Makeup Water Treatment Sys. Aux. Bldg. Isol. Valve (P21-F024)-(A)	30
Domestic Water System Aux. Bldg. Isol. Valve (P66-F029A)-(A&B)	4
PSW Aux. Bldg. Isol. Valve (P44-F121)-(A)	100

TABLE 3.6.6.2-1 (Continued)

SECONDARY CONTAINMENT VENTILATION SYSTEM AUTOMATIC ISOLATION DAMPERS/VALVES

<u>VALVE FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
Valves (Continued)	
PSW Aux. Bldg. Isol. Valve (P44-F122)-(A)	100
PSW Aux. Bldg. Isol. Valve (P44-F117)-(A)	100
PSW Aux. Bldg. Isol. Valve (P44-F118)-(A)	100
PSW Aux. Bldg. Isol. Valve (P44-F120)-(B)	100
PSW Aux. Bldg. Isol. Valve (P44-F123)-(B)	100
PSW Aux. Bldg. Isol. Valve (P44-F116)-(B)	100
PSW Aux. Bldg. Isol. Valve (P44-F119)-(B)	100
RHR Discharge To Liquid Radwaste Valve (E12-F203)-(A&B)	30

TABLE 3.8.4.2-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>BYPASS DEVICE (CONTINUOUS) (ACCIDENT CONDITIONS) (NO)</u>	<u>SYSTEM(S) AFFECTED</u>
Q1E51F010	Continuous	RCIC System
Q1E51F013	Continuous	RCIC System
Q1E51F019	Continuous	RCIC System
Q1E51F022	Continuous	RCIC System
Q1E51F031	Continuous	RCIC System
Q1E51F045	Continuous	RCIC System
Q1E51F046	Continuous	RCIC System
Q1E51F059	Continuous	RCIC System
Q1E51F068	Continuous	RCIC System
RCIC Trip and Throttle Valve on Turbine Q1E51C002	Continuous	RCIC System
Q1E51F095	Continuous	RCIC System
Q1B21F065A	No	Reactor Coolant System
Q1B21F065B	No	Reactor Coolant System
Q1B21F098A	No	Reactor Coolant System
Q1B21F098B	No	Reactor Coolant System
Q1B21F098C	No	Reactor Coolant System
Q1B21F098D	No	Reactor Coolant System
Q1B21F019	Continuous	Reactor Coolant System
Q1B21F067A	Continuous	Reactor Coolant System
Q1B21F067B	Continuous	Reactor Coolant System
Q1B21F067C	Continuous	Reactor Coolant System
Q1B21F067D	Continuous	Reactor Coolant System
Q1B21F016	Continuous	Reactor Coolant System
Q1B21F147A	Continuous	MSL Drain Post LOCA Leakage Control
Q1B21F147B	Continuous	MSL Drain Post LOCA Leakage Control
Q1B33F019	Continuous	Recirculation System
Q1B33F020	Continuous	Recirculation System
Q1B33F125	Continuous	Recirculation System
Q1B33F126	Continuous	Recirculation System
Q1B33F127	Continuous	Recirculation System
Q1B33F128	Continuous	Recirculation System
Q1D23F591	*	Drywell Monitoring System
Q1D23F592	*	Drywell Monitoring System
Q1D23F593	*	Drywell Monitoring System
Q1D23F594	*	Drywell Monitoring System
Q1E12F040	Continuous	RHR System
Q1E12F023	Continuous	RHR System
Q1E12F006A	Continuous	RHR System
Q1E12F052A	Continuous	RHR System
Q1E12F008	Continuous	RHR System
Q1E12F394	Continuous	RHR System

TABLE 3.8.4.2-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>BYPASS DEVICE (CONTINUOUS) (ACCIDENT CONDITIONS) (NO)</u>	<u>SYSTEM(S) AFFECTED</u>
Q1E12F074A	Continuous	RHR System
Q1E12F026A	Continuous	RHR System
Q1E12F082A	No	RHR System
Q1E12F082B	No	RHR System
Q1E12F290A	Continuous	RHR System
Q1E12F047A	Continuous	RHR System
Q1E12F027A	Continuous	RHR System
Q1E12F073A	Continuous	RHR System
Q1E12F346	Continuous	RHR System
Q1E12F024A	Continuous	RHR System
Q1E12F087A	Continuous	RHR System
Q1E12F048A	Continuous	RHR System
Q1E12F042A	Continuous	RHR System
Q1E12F004A	Continuous	RHR System
Q1E12F003A	Continuous	RHR System
Q1E12F011A	Continuous	RHR System
Q1E12F053A	Continuous	RHR System
Q1E12F037A	Continuous	RHR System
Q1E12F028A	Continuous	RHR System
Q1E12F064A	Continuous	RHR System
Q1E12F290B	Continuous	RHR System
Q1E12F004C	Continuous	RHR System
Q1E12F021	Continuous	RHR System
Q1E12F064C	Continuous	RHR System
Q1E12F042C	Continuous	RHR System
Q1E12F048B	Continuous	RHR System
Q1E12F049	Continuous	RHR System
Q1E12F037B	Continuous	RHR System
Q1E12F053B	Continuous	RHR System
Q1E12F074B	Continuous	RHR System
Q1E12F042B	Continuous	RHR System
Q1E12F064B	Continuous	RHR System
Q1E12F096	Continuous	RHR System
Q1E12F094	Continuous	RHR System
Q1E12F006B	Continuous	RHR System
Q1E12F011B	Continuous	RHR System
Q1E12F052B	Continuous	RHR System
Q1E12F047B	Continuous	RHR System
Q1E12F027B	Continuous	RHR System
Q1E12F004B	Continuous	RHR System
Q1E12F087B	Continuous	RHR System
Q1E12F003B	Continuous	RHR System
Q1E12F026B	Continuous	RHR System
Q1E12F024B	Continuous	RHR System
Q1E12F028B	Continuous	RHR System
Q1E12F009	Continuous	RHR System
Q1E12F073B	Continuous	RHR System



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-29

MISSISSIPPI POWER & LIGHT COMPANY

SYSTEM ENERGY RESOURCES, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

GRAND GULF NUCLEAR STATION, UNIT 1

DOCKET NO. 50-416

1.0 INTRODUCTION

By letter dated October 17, 1986, Mississippi Power & Light Company, (the licensee)\* requested an amendment to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1 (GGNS-1) for four items. Items 1, 3 and 4 are addressed in this amendment. The proposed amendment would change the Technical Specifications (TSs) by: (1) deleting the two-minute time limit for closing a stuck open safety relief valve (SRV) and changing the temperature limit of the suppression pool water from 105°F to 110°F; (2) adding a requirement to bypass the thermal overload protection device for the motor operated valve in the reactor core isolation cooling (RCIC) turbine bypass line; and (3) changing the nomenclature of a secondary containment isolation valve in the residual heat removal (RHR) discharge line to the liquid radwaste system. These three changes are considered separately in the evaluation below.

2.0 EVALUATION

2.1 Limiting conditions for operation with stuck-open safety relief valve

Action Statement b in TS 3.4.2.1 requires that with one or more stuck-open SRV(s), the reactor mode switch must be placed in the shutdown position, if the suppression pool temperature is 105°F or greater, or if the stuck-open SRVs are not closed within two minutes. The 105°F suppression pool temperature limit would be changed to 110°F in order to be consistent with Action Statement b(2) in TS 3.6.3.1 which requires reactor shutdown when the suppression pool average water temperature exceeds 110°F. The licensee proposed to delete the two-minute closure requirement for stuck-open SRV(s) because the present two-minute limit does not allow sufficient time for operator action based on a stuck-open

\*On December 20, 1986, the Commission issued License Amendment No. 27 which authorized the transfer of control and performance of licensed activities from Mississippi Power & Light Company to System Energy Resources, Inc. (SERI). "The licensee" refers to Mississippi Power & Light Company before December 20, 1986 and to SERI on or after December 20, 1986.

SRV event in GGNS-1 (LER 86-011-00, dated May 7, 1986). The GGNS Final Safety Analysis Report, (FSAR) Section 15.1.4, "Inadvertent Safety/Relief Valve Opening" takes no credit for a reactor shutdown within two minutes following the determination that an SRV is stuck open. FSAR Section 15.1.4.5 states that the radiological consequences of a stuck-open SRV are less than those associated with MSIV closure. The event is, therefore, bounded by the analysis performed for MSIV closure as described in FSAR Section 15.2.4.5. Adequate capability of the suppression pool to perform its steam suppression function is obtained in TS 3.6.3.1 by specifying minimum pool water level and maximum pool water temperature. The proposed deletion of the two-minute closure time limit while retaining the 110°F pool temperature limit in TS 3.4.2.1 for stuckopen SRVs would avert unnecessary reactor shutdowns while maintaining adequate capability of the suppression pool to perform its intended post accident steam suppression function. Suppression pool temperatures exceeding the proposed 110°F suppression pool temperature limit would require reactor shutdown for stuck-open SRV(s).

For reasons cited above, the NRC staff concludes that the proposed deletion of the two-minute closure time limit for stuck-open SRV(s) and the change of the suppression pool temperature limit from 105°F to 110°F, are acceptable.

## 2.2 Bypass of the thermal overload protection device for the motor operated valve in the RCIC turbine bypass line

The RCIC turbine bypass valve is a motor operated valve, which allows a small amount of steam to be supplied to the RCIC turbine prior to the opening of the main steam supply valve. This initial supply of steam allows the RCIC turbine to operate at lower speeds during the starting transients and thereby establish the necessary hydraulic oil pressure needed for the operation of the turbine governor valve. By allowing a smaller amount of steam to enter the RCIC turbine initially via this bypass valve, potential overspeed trip conditions are avoided.

Thermal overload relays are designed primarily to protect continuous-duty motors while they are running rather than during starting. Use of these overload devices to protect intermittent-duty motors may result in undesired actuation of the devices and subsequent motor trip if the cumulative effect of heating caused by successive starts at short intervals is not taken into account in determining the overload trip setting.

The licensee proposed to bypass the thermal overload protection device for the motor operator of the RCIC turbine bypass valve and to add the bypass in TS Table 3.8.4.2-1. By continuously bypassing the thermal overload protection the availability of the RCIC turbine bypass valve to perform its function will be enhanced because the valve will utilize all available design margins in abnormal situations. However, the thermal overload protection would not be bypassed during testing and maintenance periods,

for maximum equipment protection. The licensee has stated that this bypassing does not result in jeopardizing the completion of the intended design function for RCIC, or in degrading other safety systems because of any sustained abnormal motor circuit currents that may be present. Should the RCIC turbine bypass valve fail the RCIC turbine control valve can be manually-operated to start the RCIC.

We have compared this proposal with Regulatory Guide 1.106, "Thermal Overload Protection for Electric Motor-Operated Valves" and found it to be consistent with the Regulatory Guide. Since the licensee's proposal may increase the availability of the RCIC system, will not degrade other safety systems' function, and is consistent with Regulatory Guide 1.106, the NRC staff finds the change to the TS to add the bypass of the thermal overload protection device for the motor operated valve in the RCIC turbine bypass to be acceptable.

### 2.3 Nomenclature of a secondary containment isolation valve

TS Table 3.6.6.2-1, "Secondary Containment Ventilation System Automatic Isolation Dampers/Valves" includes a valve labeled: RHR "A" Loop Discharge to Liquid Radwaste Valve (E12-F203)-(A). The proposed new designation for the valve would be: RHR Discharge to Liquid Radwaste Valve (E12-F203)-(A&B). The deletion of "A Loop" from the designation reflects the as-built system since flushing lines from both RHR Loops A and B connect to the RHR discharge header which discharges to the liquid radwaste system through valve E12-F203. The term "(A and B)" at the end of the proposed designations indicates that the two solenoid valves which supply air to the air-operated RHR valve are powered from separate ESF Power Dimensions "A" and "B". Because the proposed change is administrative and does not change requirements in the Technical Specifications, the NPC staff concludes that the change in valve nomenclature is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 5862) on February 26, 1987, and consulted with the state of Mississippi. No public comments were received, and the state of Mississippi did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and the security nor to the health and safety of the public.

**Principal Contributors:**

D. D. Yue, Reactor Systems Branch, DBL  
F. J. Witt, Plant Systems Branch, DBL

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