



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 10 1986

Docket No. 50-416

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear
Mississippi Power & Light Company
P.O. Box 23054
Jackson, Mississippi 39205

Dear Mr. Kingsley:

SUBJECT: ISSUANCE OF AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE
NPF-29 GRAND GULF NUCLEAR STATION, UNIT NO. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 10 to Facility Operating License NPF-29 for the Grand Gulf Nuclear Station, Unit No. 1, located in Claiborne County, Mississippi. This amendment is in response to your letter dated November 14, 1985 as revised by your March 21, 1986 letter.

The amendment modifies License Condition 2.C.(28) "Advisor to Vice President" and changes the Technical Specifications related to the offsite organization to reflect changes in the licensee's nuclear operations staff. The amendment is effective as of its date of issuance. A copy of the related safety evaluation supporting Amendment No. 10 to Facility Operating License NPF-29 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Walter R. Butler".

Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosures:

1. Amendment No. 10
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. Oliver D. Kingsley, Jr.
Mississippi Power & Light Company

Grand Gulf Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MISSISSIPPI POWER & LIGHT COMPANY

MIDDLE SOUTH ENERGY, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

DOCKET NO. 50-416

GRAND GULF NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Grand Gulf Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-29 filed by the Mississippi Power & Light Company acting for itself, Middle South Energy, Inc., and South Mississippi Electric Power Association (the licensees), dated November 14, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this license amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraphs 2.C(2) and 2.C.(28) of Facility Operating License No. NPF-29 are hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through

Amendment No. 10 and the Environmental Protection Plan contained in Appendix B are hereby incorporated into this license. Mississippi Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(28) Advisor to Vice President

MP&L shall have on its nuclear operations staff, one or more corporate management officials or advisors (who may be either permanent employees or contracted consultants) who have substantial commercial nuclear power plant operating management experience and who will advise on all decisions affecting safe operation of the plant. This requirement shall be in effect until the plant has accumulated at least 6 months at power levels above 90% of full power.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosure:
Technical Specifications Changes

Date of Issuance: APR 10 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 10

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO.. 50-416

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended</u> <u>Page</u>	<u>Overleaf</u> <u>Page</u>
6-1	6-2
6-3	
6-6	6-5
6-9	
6-10	
6-11	
6-12	
6-13	
6-14	

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The GGNS General Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Superintendent or, during his absence from the Control Room, a designated individual shall be responsible for the Control Room command function. A management directive to this effect, signed by the Vice President, Nuclear Operations shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 OFFSITE

The offsite organization for unit management and technical support shall be as shown on Figure 6.2.1-1.

6.2.2 UNIT STAFF

The unit organization shall be as shown on Figure 6.2.2-1 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1.
- b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Reactor Operator shall be in the Control Room.
- c. A health physics technician* shall be onsite when fuel is in the reactor.
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. A site Fire Brigade of at least 5 members shall be maintained onsite at all times*. The Fire Brigade shall not include the Shift Superintendent, the STA, the two other members of the minimum shift crew necessary for safe shutdown of the unit, and any personnel required for other essential functions during a fire emergency. At least one AO shall be available to respond to non-fire-fighting commands from the Control Room.

*The number of health physics technicians and Fire Brigade personnel may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven-day period, all excluding shift turnover time.
3. A break of at least eight hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the GGNS General Manager or his designee, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the GGNS General Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The ISEG shall be composed of a multi-disciplined dedicated, onsite, group with a minimum assigned complement of five engineers or appropriate specialists.

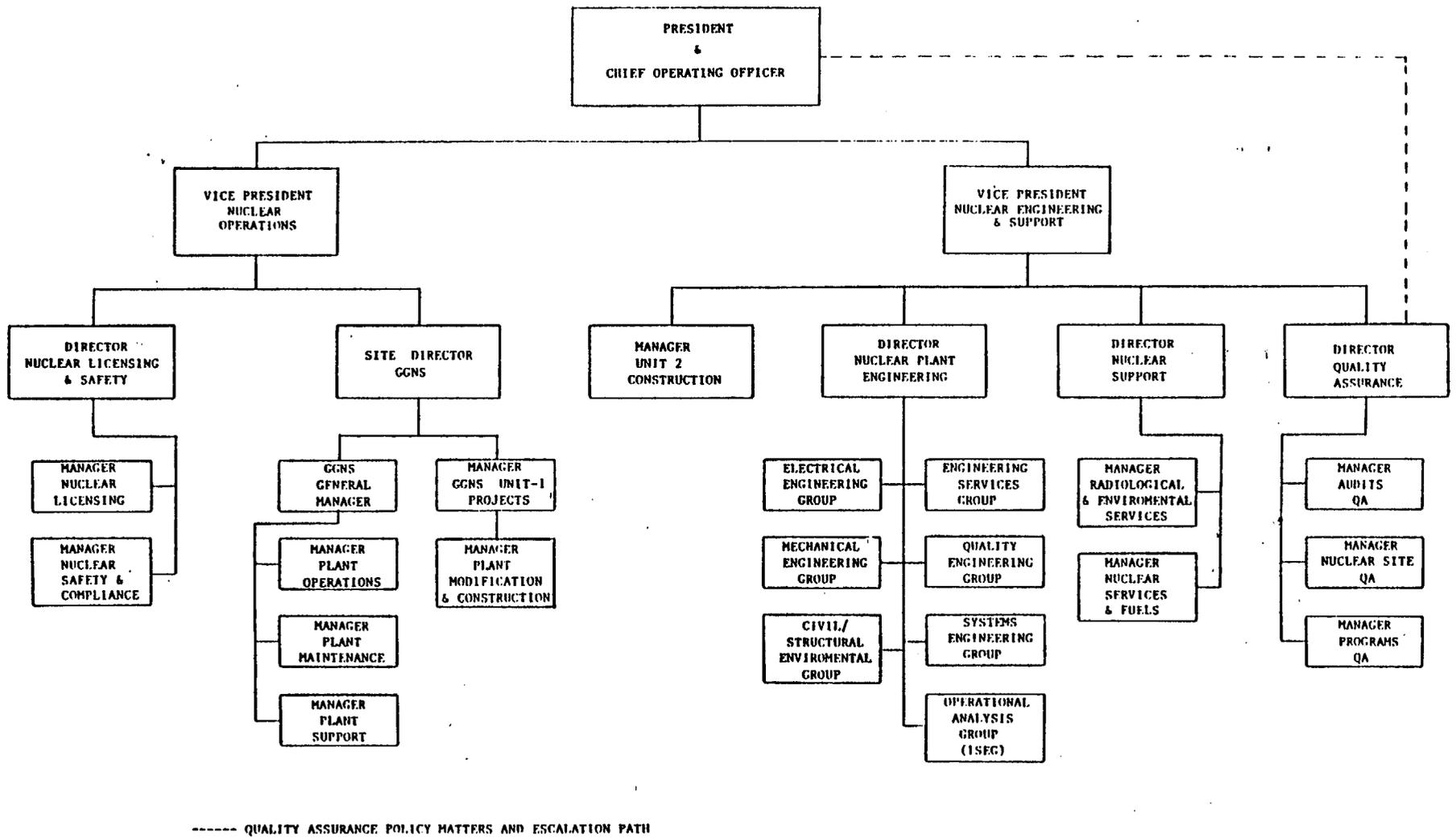


FIGURE 6.2.1-1 OFFSITE ORGANIZATION

TABLE 6.2.2-1

MINIMUM SHIFT CREW COMPOSITION

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	<u>CONDITIONS 1, 2, & 3</u>	<u>CONDITIONS 4 & 5</u>
SS	1	1
SRO	1	None
RO	2	1
AO	2	1
STA	1	None

TABLE NOTATION

- SS - Shift Superintendent with a Senior Reactor Operators License on Unit 1.
- SRO - Individual with a Senior Reactor Operators License on Unit 1.
- RO - Individual with a Reactor Operators License on Unit 1.
- AO - Auxiliary Operator.
- STA - Shift Technical Advisor.

Except for the Shift Superintendent, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Superintendent from the Control Room while the unit is in OPERATIONAL CONDITION 1, 2 or 3 an individual, other than the Shift Technical Advisor, with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Superintendent from the Control Room while the unit is in OPERATIONAL CONDITION 4 or 5, an individual with a valid SRO or RO license shall be designated to assume the Control Room command function.

ADMINISTRATIVE CONTROLS

INDEPENDENT SAFETY ENGINEERING GROUP (ISEG) (Continued)

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of unit activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

6.2.3.4 The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the Vice President, Nuclear Operations.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide technical support to the Shift Superintendent in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to safe operation of the unit.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter# to all licensees, except for the Chemistry/Radiation Control Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975; the Shift Technical Advisor who shall meet or exceed the qualifications referred to in Section 2.2.1.b of Enclosure I of the October 30, 1979 NRC letter to all operating nuclear power plants; and those members of the Independent Safety Engineering Group used for meeting the minimum complement specified in Section 6.2.3.2, each of whom shall have a Bachelor of Science degree or be registered as a Professional Engineer and shall have at least two years experience in their field, at least one year of which experience shall be in the nuclear field.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Superintendent, shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter# to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the GGNS General Manager on all matters related to nuclear safety.

* Not responsible for sign-off function.

Except that the experience and other training information provided in the licensee's letter to the NRC dated July 29, 1985 are acceptable for the individuals listed in that letter.

ADMINISTRATIVE CONTROLS

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The SRC shall be composed of the:

Chairman:	Vice President, Nuclear Operations
Member:	Vice President, Nuclear Engineering & Support
Member:	Director, Nuclear Plant Engineering
Member:	Site Director, GGNS
Member:	Director, Quality Assurance
Member:	Designated Representative, Middle South Services, Inc.
Member:	GGNS General Manager
Member:	Director, Nuclear Licensing and Safety
Member:	Manager, Radiological and Environmental Services
Member:	Principal Engineer, Operations Analysis

Two or more additional voting members shall be consultants to Mississippi Power and Light Company consistent with the recommendations of the Advisory Committee on Reactor Safeguards letter, Mark to Palladino dated October 20, 1981.

The SRC members shall hold a Bachelor's degree in an engineering or physical science field or equivalent experience and a minimum of five years of technical experience of which a minimum of three years shall be in one or more of the disciplines of 6.5.2.1a through h. In the aggregate, the membership of the committee shall provide specific practical experience in the majority of the disciplines of 6.5.2.1a through h.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

ADMINISTRATIVE CONTROLS

CONSULTANTS

6.5.2.4 Consultants, in addition to those required in Specification 6.5.2.2, shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC. No more than three consultants shall participate as voting members in SRC activities at any one time.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 The quorum of the SRC necessary for the performance of the SRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 7 SRC voting members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluations for (1) changes to procedures, equipment or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Appendix A Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. ALL REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the PSRC.
- j. Written reports from audits of the ALARA program.

ADMINISTRATIVE CONTROLS

AUDITS

- 6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRC. These audits shall encompass:
- a. The conformance of unit operation to provisions contained within the Appendix A Technical Specifications and applicable license conditions at least once per 12 months.
 - b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
 - c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
 - d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
 - e. The Emergency Plan and implementing procedures at least once per 12 months.
 - f. The Security Plan and implementing procedures at least once per 12 months.
 - g. Any other area of unit operation considered appropriate by the SRC or the Vice President, Nuclear Operations.
 - h. The Fire Protection Program and implementing procedures at least once per 24 months.
 - i. An independent fire protection and loss prevention inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
 - j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 36 months.
 - k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
 - l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
 - m. The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes at least once per 24 months.
 - n. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 4.15, February 1979, at least once per 12 months.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Vice President, Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Nuclear Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5.3 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Procedures required by Technical Specification 6.8 and other procedures which affect plant nuclear safety, and changes thereto, shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by an individual/group other than the individual/group which prepared the procedure or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Administrative Procedures shall be approved as delineated in writing by the GGNS General Manager. The GGNS General Manager shall approve administrative procedures, security implementing procedures and emergency plant implementing procedures. Temporary approval to procedures which clearly do not change the intent of the approved procedures may be made by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License. Temporary changes shall be reviewed by the reviewing authority within 14 days of being issued. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change.
- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Site Director, GGNS. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Implementation of proposed modifications to plant nuclear safety-related structures, systems and components shall be approved by the GGNS General Manager.

ADMINISTRATIVE CONTROLS

ACTIVITIES (Continued)

- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment.
- d. Events reportable pursuant to Section 50.73 to 10 CFR Part 50 shall be investigated and a report prepared which evaluates the event and which provides recommendations to prevent recurrence. Such report shall be approved by the GGNS General Manager.
- e. Individuals responsible for reviews performed in accordance with 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c and 6.5.3.1.d shall meet or exceed the qualification requirements of Section 4.4 of ANSI 18.1, 1971, as previously designated by the Site Director, GGNS or GGNS General Manager, as applicable. Each such review shall include a determination of whether or not additional, cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review shall include a determination of whether or not an unreviewed safety question is involved.
- g. Records of the above activities shall be provided to the GGNS General Manager, PSRC and/or as necessary for required reviews.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified pursuant to the requirements of Section 50.72 to 10 CFR Part 50, and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and submitted to the SRC and the Vice President, Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President, Nuclear Operations and the SRC shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems or structures, and (3) corrective action taken to prevent recurrence.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Vice President, Nuclear Operations within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 4.15, February 1979.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed as required by 6.5, above, prior to implementation and shall be reviewed periodically as set forth in administrative procedures.

6.8.3 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the:

1. RCIC system outside containment containing steam or water, except the drain line to the main condenser.
2. RHR system outside containment containing steam or water, except the line to the LRW system and headers that are isolated by manual valves.
3. HPCS system.
4. LPCS system.
5. Hydrogen analyzers of the combustible gas control system.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NPF-29

GRAND GULF NUCLEAR STATION, UNIT 1

MISSISSIPPI POWER & LIGHT COMPANY

MIDDLE SOUTH ENERGY, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

INTRODUCTION

By letter dated November 14, 1985, the licensee requested an amendment to the facility operating license which would: (1) delete License Condition 2.C.(28) which requires an Advisor to the Vice President in charge of nuclear operations with substantial commercial nuclear power plant operating management experience; and (2) change Technical Specification Figure 6.2.1-1 "Offsite Organization" and make other changes in Section 6 "Administrative Controls" to reflect proposed changes in the Nuclear Production Department (NPD) of the Grand Gulf Nuclear Station (GGNS). Specifically, the proposed Technical Specification (TS) changes consist of: (a) revisions to the Offsite Organization Chart; (b) changes in the membership of the Safety Review Committee (SRC) and an increase in the SRC quorum; and (c) changes in the authority to designate reviewers of modifications to safety related structures, systems and components.

By letter dated March 21, 1986, the licensee modified its November 14, 1985, submittal by withdrawing the request to delete License Condition 2.C.(28) and proposing instead to change the license condition to allow credit to be taken for the nuclear power plant operating management experience of the licensee's present management. The notice of consideration of issuance of this license amendment was published in the Federal Register before the licensee's March 21, 1986 submittal. The content of the March 21, 1986, submittal is not significantly different from the content of the initial submittal concerning this license condition in that the licensee justifies the change to License Condition 2.C.(28) (as it did the deletion of the license condition) on the basis that nuclear power plant operating experience possessed by new managers can be substituted for the experience of the advisor in the present license condition. The notice of consideration stated that the proposed change would terminate the requirement for an advisor based upon the experience of the new managers. Since the content of the March 21, 1986, submittal is not significantly different from the initial submittal and the notice adequately describes the license amendment request, as revised, the staff believes it is not necessary to renote this amendment request.

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2.0 EVALUATION

2.1 Change to License Condition 2.C.(28)

This license condition requires MP&L to provide an Advisor to the Vice President, Nuclear Operations until the plant has operated for at least 6 months at power levels above 90% of full power. This license condition was imposed in June 1982 due to the lack of commercial nuclear power experience in MP&L corporate management and states the following: "MP&L shall provide one or more additional staff members, reporting directly to a Vice President principally in charge of nuclear operations, who have substantial commercial nuclear power plant operating management experience and who will act as advisors to the Vice President on all decisions affecting safe operation of the plant. The additional staff members may be permanent employees or contracted consultants, but they shall be retained in this advisory position until the plant has operated for at least 6 months at power levels above 90% of full power."

MP&L has been fulfilling this license condition with contractors who have filled the position of Advisor to the Vice President, Nuclear Operations for over 3 years (approximately 75 days of that time while the plant was above 90% full power). However, since the license condition was imposed, MP&L has hired top management with commercial nuclear power experience. MP&L's current President and Chief Operating Officer, who has served in this capacity since 1984, and the Vice President, Nuclear Operations, who joined the staff in March 1985, have substantial commercial nuclear power plant operating management experience. Their resumes provided in MP&L's Updated Final Safety Analysis Report (UFSAR) indicate the following commercial nuclear power experience:

- ° William Cavanaugh III - President and Chief Operating Officer

Approximately 20 years nuclear power experience including over 10 years commercial nuclear power experience.

- ° Oliver D. Kingsley, Jr. - Vice President, Nuclear Operations

Over 15 years of commercial nuclear power plant experience including 1 year as a Plant Manager.

By letter dated March 21, 1986, the licensee proposed to change License Condition 2.C.(28) to state the following: "MP&L shall have on its nuclear operations staff, one or more corporate management officials or advisors (who may be either permanent employees or contracted consultants) who have substantial commercial nuclear power plant operating management experience and who will advise on all decisions affecting safe operation of the plant. This requirement shall be in effect until the plant has accumulated at least 6 months at power levels above 90% of full power."

Based on the above, the staff finds that the licensee's proposed change to License Condition 2.C.(28) is acceptable. The proposed license condition would replace the Advisor to the Vice President, Nuclear Operations with the President and Chief Operating Officer and the Vice President, Nuclear Operations who have substantial commercial nuclear power operating management experience.

2.2 Technical Specification Changes to Reflect Changes in the Nuclear Production Department

(a) Revisions to the Offsite Organization Chart

The revised chart (Technical Specification Figure 6.2.1-1) includes the following newly created positions: the President and Chief Operating Officer; the Vice President, Nuclear Engineering and Support; the Site Director, GGNS; the Manager, GGNS Unit 1-Projects; and the Manager, Plant Modification and Construction.

The President and Chief Operating Officer of MP&L has the ultimate responsibility for providing top level direction of all activities associated with the operation of GGNS through the Vice President, Nuclear Operations and the Vice President, Nuclear Engineering and Support. In addition, he will provide guidance on corporate quality assurance policy to the Director, Quality Assurance. The Director, Quality Assurance will maintain his independence by having a direct escalation path to the President and Chief Operating Officer.

The position Senior Vice President, Nuclear will be deleted since the President and Chief Operating Officer will assume the responsibility for providing top level direction of the activities associated with GGNS. All references in Section 6.0 of the Technical Specifications to Senior Vice President, Nuclear have been changed to Vice President, Nuclear Operations.

The new position of Vice President, Nuclear Engineering and Support and the positions reporting to him place more emphasis on the engineering and support functions in support of the operation of GGNS Unit 1 and less emphasis on the construction of GGNS Unit 2.

The new position of Site Director, GGNS will provide a higher level of management involvement at the site for problem solving and interfacing with other organizations.

The staff finds that the above proposed changes in the offsite organization should promote management control and effective lines of authority and communication between organizational units involved in management, operation and technical support. This meets the criteria in Sections 13.1.1 and 13.1.2 of NUREG-0800, the Standard Review Plan (SRP) and is in accordance with the guidance provided in NUREG-0731. Therefore, we find the organizational changes acceptable.

(b) Changes to Safety Review Committee Membership and Quorum

As proposed by the licensee, changes in the Safety Review Committee (Technical Specifications 6.5.2.2 and 6.5.2.6) involve adding the Site Director, GGNS and the Vice President, Nuclear Engineering and Support as Safety Review Committee members; deleting the Advisor to the Vice President, Nuclear Operations from the SRC; and increasing the SRC quorum from at least six voting members to seven. Adding these members to the SRC will provide broader management input ensuring that top management with valuable experience are involved in significant safety issues. These proposed changes meet the acceptance criteria in Section 13.4.2 of NUREG-0800, the Standard Review Plan and therefore, are acceptable.

(c) Changes in Authority to Designate Reviewers

The proposed change to Technical Specification (TS) 6.5.3.1.b allows the new position of Site Director, GGNS instead of the GGNS General Manager (who now reports to the Site Director) to designate who will conduct reviews of proposed changes or modifications to plant nuclear safety-related structures, systems and components. The staff notes that the Site Director, GGNS can choose the appropriate subordinate review group (plant staff or projects/modification and construction) thus ensuring effective reviews by groups specializing in the areas of concern. Hence, this change allows a higher level manager who has responsibility for those areas to select the subordinate review group. The staff finds that this is consistent with our approval of the proposed changes of the offsite organization chart (as discussed in Paragraph (a) above) and is, therefore, acceptable.

The proposed change to Technical Specification 6.5.3.1.e allows deletion of the requirement that individuals who conduct reviews, in accordance with TS 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c and 6.5.3.1.d, be members of the plant staff. These individuals are responsible for reviews of: (1) procedures required by Technical Specification 6.8 and other procedures which affect plant nuclear safety (TS 6.5.3.1.a); (2) proposed changes or modifications to plant nuclear safety-related structures, systems or components (TS 6.5.3.1.b); (3) proposed tests and experiments which affect plant nuclear safety and are not addressed in the FSAR (TS 6.5.3.1.c); and (4) events reportable pursuant to Section 50.73 to 10 CFR Part 50 (TS 6.5.3.1.d). The staff finds this proposed change to be acceptable since it requires that individuals who conduct the above cited reviews but who may not be members of the plant staff, meet or exceed the qualification requirements for professional technical review groups in Section 4.4 of ANSI N18.1-1971; it allows personnel who have appropriate knowledge to participate in these reviews, thereby providing review flexibility; and it is consistent with Section 13.4 of the SRP.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment relates to changes in administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria of categorical exclusion set forth in 10 CFR 51.22(c)(10) and pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (50 FR 49633) on December 3, 1985, and consulted with the state of Mississippi. No public comments were received, and the state of Mississippi did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: I. Schoenfeld, Facility Operations Branch, DBL

Dated: APR 10 1986

APR 10 1986

Docket No. 50-416

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear
Mississippi Power & Light Company
P.O. Box 23054
Jackson, Mississippi 39205

Dear Mr. Kingsley:

SUBJECT: ISSUANCE OF AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE
NPF-29 GRAND GULF NUCLEAR STATION, UNIT NO. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 10 to Facility Operating License NPF-29 for the Grand Gulf Nuclear Station, Unit No. 1, located in Claiborne County, Mississippi. This amendment is in response to your letter dated November 14, 1985 as revised by your March 21, 1986 letter.

The amendment modifies License Condition 2.C.(28) "Advisor to Vice President" and changes the Technical Specifications related to the offsite organization to reflect changes in the licensee's nuclear operations staff. The amendment is effective as of its date of issuance. A copy of the related safety evaluation supporting Amendment No. 10 to Facility Operating License NPF-29 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosures:

- 1. Amendment No. 10
- 2. Safety Evaluation

cc w/enclosures:
See next page

*Previously concurred: *JK*
 PD#4 PD#4/PM
 *Duncan ml *LKintner
 2/27/86 2/25/86
 3/28/86

WB
 PD#4/D
 *WButler
 2/28/86
 4/8/86.

Docket No. 50-416

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear
Mississippi Power & Light Company
P.O. Box 23054
Jackson, Mississippi 39205

Dear Mr. Kingsley:

SUBJECT: ISSUANCE OF AMENDMENT NO. TO FACILITY OPERATING LICENSE
NPF-29 GRAND GULF NUCLEAR STATION, UNIT NO. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. to Facility Operating License NPF-29 for the Grand Gulf Nuclear Station, Unit No. 1, located in Claiborne County, Mississippi. This amendment is in response to your letter dated November 14, 1985.

The amendment modifies the Technical Specifications related to the offsite organization to reflect changes in the Nuclear Production Department of the Grand Gulf Nuclear Station. The amendment is effective as of its date of issuance. A copy of the related safety evaluation supporting Amendment No. to Facility Operating License NPF-29 is enclosed.

The staff has also reviewed your proposal to fulfill the requirement in License Condition 2.C.(28) (for staff members with substantial commercial nuclear power plant operating management experience) through the Vice President, Nuclear Operations and the President and Chief Operating Officer, instead of with an Advisor to the Vice President, Nuclear Operations. The staff's evaluation of this proposal is included in the enclosed safety evaluation. Because the current Vice President, Nuclear Operations and the President have substantial commercial nuclear power plant operating management experience, the staff finds the proposal to be acceptable.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosures:

1. Amendment No.
2. Safety Evaluation

cc w/enclosures:
See next page

PD#4
Duncan ml
2/27/86

PD#4/PM
LKintner
2/25/86

PD#4/D
WButler
2/28/86

WB

Amendment No.10 and the Environmental Protection Plan contained in Appendix B are hereby incorporated into this license. Mississippi Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(28) Advisor to Vice President

MP&L shall have on its nuclear operations staff, one or more corporate management officials or advisors (who may be either permanent employees or contracted consultants) who have substantial commercial nuclear power plant operating management experience and who will advise on all decisions affecting safe operation of the plant. This requirement shall be in effect until the plant has accumulated at least 6 months at power levels above 90% of full power.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosure:
Technical Specifications Changes

Date of Issuance: APR 10 1986

*Previously concurred:

PD/LA

PD#4/PM

*MDuncan

*LKintner

02/27/86

02/25/86

OELD

M Young
03/31/86

PD#4/D

*WButler

02/28/86

JPC
WBY
WB
4/8/86

3/31/86

Amendment No. and the Environmental Protection Plan contained in Appendix B are hereby incorporated into this license. Mississippi Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosure:
Technical Specifications Changes

Date of Issuance:

PD/LA
MDuncan
02/27/86

PKC
PD#4/PM
LKintner
02/25/86

PD#4/D
WButler
02/28/86

WB

RE

Amendment No. 10 to Facility Operating License No. NPF-29-Grand Gulf, Unit 1

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