

April 30, 1991

Docket No. 50-298

Mr. Guy R. Horn
Nuclear Power Group Manager
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68602-0499

Dear Mr. Horn:

SUBJECT: CORRECTION TO AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE
NO. DPR-46 (TAC NO. 75840)

On April 12, 1990, the Commission issued Amendment No. 133 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. This amendment consisted of changes to the Technical Specifications (TSs) in response to your application dated February 12, 1990. The amendment revised the TSs to reflect the Cycle 14 reload and included a change to Section 1.1.A (page 6) associated with MCPR Safety Limits.

A typographical error associated with Section 1.1.C was inadvertently included in the amendment. Enclosed is a corrected page with the correction of Section 1.1.C.

Sincerely,

Paul W. O'Connor
Signed By:

Paul W. O'Connor, Project Manager
Project Directorate IV-1
Division of Reactor Projects III, IV, and V
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "Paul W. O'Connor".

Paul W. O'Connor, Project Manager
Project Directorate IV-1
Division of Reactor Projects III, IV, and V
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

Mr. Guy R. Horn
Nuclear Power Group Manager

Cooper Nuclear Station

cc:

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1.1 FUEL CLADDING INTEGRITYApplicability

The Safety Limits established to preserve the fuel cladding integrity apply to those variables which monitor the fuel thermal behavior.

Objective

The objective of the Safety Limits is to establish limits below which the integrity of the fuel cladding is preserved.

Action

If a Safety Limit is exceeded, the reactor shall be in at least hot shutdown within 2 hours.

Specifications

- A. Reactor Pressure >800 psia and Core Flow >10% of Rated

The existence of a minimum critical power ratio (MCPR) less than 1.06 for two recirculation loop operation (1.07 for single-loop operation) shall constitute violation of the fuel cladding integrity safety.

- B. Core Thermal Power Limit (Reactor Pressure <800 psia and/or Core Flow <10%)

When the reactor pressure is <800 psia or core flow is less than 10% of rated, the core thermal power shall not exceed 25% of rated thermal power.

- C. Power Transient

To ensure that the Safety Limit established in Specification 1.1.A and 1.1.B is not exceeded, each required scram shall be initiated by its expected scram signal. The Safety Limit shall be assumed to be exceeded when scram is accomplished by a means other than the expected scram signal.

2.1 FUEL CLADDING INTEGRITYApplicability

The Limiting Safety System Settings apply to trip settings of the instruments and devices which are provided to prevent the fuel cladding integrity Safety Limits from being exceeded.

Objective

The objective of the Limiting Safety System Settings is to define the level of the process variables at which automatic protective action is initiated to prevent the fuel cladding integrity Safety Limits from being exceeded.

Specifications

- A. Trip Settings

The limiting safety system trip settings shall be as specified below:

1. Neutron Flux Trip Settings
 - a. APRM Flux Scram Trip Setting (Run Mode)

When the Mode Switch is in the RUN position, the APRM flux scram trip setting shall be:

$$S \leq 0.66 W + 54\% - .66 \Delta W$$

where:

- S = Setting in percent of rated thermal power (2381 MWt)
- W = Two-loop recirculation flow rate in percent of rated (rated loop recirculation flow rate is that recirculation flow rate which provides 100% coreflow at 100% power)
- ΔW = Difference between two-loop and single-loop effective drive flow at the same core flow.

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