

May 22, 1991

Docket No. 50-298

Mr. Guy R. Horn  
Nuclear Power Group Manager  
Nebraska Public Power District  
Post Office Box 499  
Columbus, Nebraska 68602-0499

Dear Mr. Trevors:

SUBJECT: COOPER NUCLEAR STATION - AMENDMENT NO. 140 TO FACILITY  
OPERATING LICENSE NO. DPR-46 (TAC NO. 79959)

The Commission has issued the enclosed Amendment No. 140 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The amendment consists of changes to the Technical Specifications in response to your application dated March 11, 1991.

The amendment changes the Technical Specifications to extend the surveillance frequency for the residual heat removal (RHR) logic system functional test from once/6 months to once/18 months in order to avoid undesirable plant configurations during power operation.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by

Paul W. O'Connor, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III, IV, and V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.140 to  
License No. DPR-46
2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION:

Docket File	NRC/Local PDR	PD4-1 Reading	P. O'Connor(2)
C. Grimes(MS13E4)	T. Quay	P. Noonan	ACRS(10)(MSP315)
OGC(MS15B18)	D. Hagan(MS3206)	G. Hill(4)(P1-37)	
Wanda Jones(MS7103)	J. Calvo(MS11F22)	PD4-1 Plant File	
GPA/PA(MS2G5)	ARM/LFMB(MS4503)	P. Harrell, RIV	

OFC	:PD4-1/LA	:PD4-1/PE	:PD4-1/PM	:OTSB	:OGC	:PD4-1/D	:
NAME	:PNoonan	:WReckley	:PO'Connor	:JCalvo	:TQuay	:	:
DATE	:4/14/91	:4/3/91	:4/4/91	:4/10/91	:4/18/91	:5/21/91	:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, reading "Paul W. O'Connor".

Paul W. O'Connor, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III, IV, and V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 140 to  
License No. DPR-46
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. Guy R. Horn  
Nuclear Power Group Manager

Cooper Nuclear Station

cc:

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Columbus, Nebraska 68602-0499

Cooper Nuclear Station  
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Lincoln, Nebraska 68509-8922

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Brownville, Nebraska 68321

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U.S. Nuclear Regulatory Commission  
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Lincoln, Nebraska 68509-5007



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140  
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nebraska Public Power District (the licensee) dated March 11, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

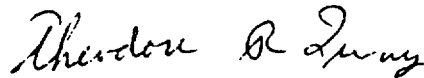
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director  
Project Directorate IV-1  
Division of Reactor Projects III, IV, and V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 22, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 140

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE PAGE

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INSERT PAGE

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COOPER NUCLEAR STATION  
TABLE 4.2.B (Page 3)  
RHR SYSTEM TEST & CALIBRATION FREQUENCIES

Item	Item I.D. No.	Functional Test Freq.	Calibration Freq.	Instrument Check
<u>Logic (4) (6)</u>				
1. Logic Bus Power Monitor		Once/6 Months	N.A.	N.A.
2. RHR Initiation		Once/18 Months	N.A.	N.A.
3. RHR Pump & Valve Control		Once/18 Months	N.A.	N.A.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE NO. DPR-46  
NEBRASKA PUBLIC POWER DISTRICT  
COOPER NUCLEAR STATION  
DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated March 11, 1991, Nebraska Public Power District (the licensee) submitted a request for changes to the Cooper Nuclear Station Technical Specifications (TS). The requested changes would change the surveillance frequency for the residual heat removal (RHR) logic system functional test from once/6 months to once/18 months in order to avoid undesirable plant configurations during power operation.

2.0 EVALUATION

The existing Cooper TS required logic system functional testing of the RHR system is performed every six months and involves placing both trains in a configuration which precludes rapid restoration of the design function if needed to respond to a design basis event. In order to prevent rendering both trains of RHR inoperable simultaneously during power operation, the licensee proposed to change the required surveillance interval to eighteen months so that the testing would be performed only during plant shutdown conditions. The change is required because the licensee's review of the surveillance procedures could not identify a practical procedural change to prevent disabling both trains during power operation.

The logic system functional tests and the simulated automatic actuation test (TS 4.5.A.3) which is performed once per operating cycle are comparable to the system functional tests that the Standard Technical Specifications (STS) require for the Emergency Core Cooling Systems (ECCS) at least once per eighteen months. The staff agrees that testing which renders the ECCS function of both RHR trains inoperable during power operation is undesirable. In fact, such testing would not be allowed by the Technical Specifications of most plants.

Based on its review, the staff agrees with the licensee that the surveillance interval for the RHR logic system functional testing should be changed to eighteen months. Testing for individual RHR system components will continue to be performed on monthly and quarterly frequencies and, when combined with the integrated tests performed each cycle, provide a high degree of confidence in the reliable performance of the system. In addition, this change would be consistent with STS and prevent the periodic loss of both RHR trains due to surveillance activities. Therefore, the staff finds the licensee's proposed TS change is acceptable.



### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Nebraska State official was notified of the proposed issuance of the amendment. The State official had no comment.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (56 FR 15643). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Reckley

Date: May 22, 1991