November 10, 1986

Docket No. 50-298

Mr. J. M. Pilant, Technical
Staff Manager
Nuclear Power Group
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68601

Dear Mr. Pilant:

The Commission has issued the enclosed Amendment No. 103 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. This amendment consists of changes to the Technical Specifications in response to your application dated September 5, 1986 (Change Number 24).

The amendment change the Technical Specifications to reflect removal of the reactor vessel head spray piping.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

William O. Long, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:
1. Amendment No. 103 to
License No. DPR-46
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Nebraska Public Power District (the licensee) dated September 5, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the licensee is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

(2) Technical Specification

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.103 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 10, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 103

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indiciated by marginal lines.

Pages

52a
168
173

NOTES FOR TABLE 3.2.A (cont'd.)

Group 2

Isolation Signals:

1. Reactor Low Water Level (+12.5 in.)
2. High Dry Well Pressure (≤ 2 psig)

Isolations:

1. RHR Shutdown Cooling System
2. Drywell floor and equipment drain sump discharge lines.
3. TIP ball valves
4. Group 6 isolation relays

Group 3

Isolation Signals:

1. Reactor Low Water Level (+12.5 in.)
2. Reactor Water Cleanup System High Flow (200% of system flow)
3. Reactor Water Cleanup System High Area Temperature (≤ 200°F)
4. Reactor Water Cleanup System High System Temperature (≤ 140°F)

Isolations:

1. Reactor Water Cleanup System

Group 4

Isolation Signals:

Provided by instruments on Table 3.2.B (HPCI)

Isolations:

Isolates the HPCI steam line

Group 5

Isolation Signals:

Provided by instruments on Table 3.2.B (RCIC)

Isolations:

Isolates the RCIC steam line.

Group 6

Isolation Signals:

1. Group 2 Isolation Signal
2. Reactor Building H&V Exhaust Plenum High Radiation (<100 mr/hr)

Amendment No. 103

COOPER NUCLEAR STATION
TABLE 3.7.1 (Page 1)
PRIMARY CONTAINMENT ISOLATION VALVES

Valve & Steam	Number of Power Operated Valves Inboard	Outboard	Maximum Operating Time (Sec) (1)	Normal Position (2)	Action On Initiating Signal (3)
Main Steam Isolation Valves MS-AO-80- A,B,C, & D MS-AO-86- A,B,C, & D	4	4	3 < T < 5 3 < T < 5	0 0	GC GC
Drywell Floor Drain Iso. Valves RW-AO-82, RW-AC-83	1	1	15	0	GC
Drywell Equipment Drain Iso. Valves RW-AO-94, RW-AO-95	1	1	15	0	GC
Main Steam Line Drain Valves MS-MO-74, MS-MO-77	1	1	30	C	SC
Reactor Water Sample Valves RRV-740AV, RRV-741AV	1	1	15	0	GC
Reactor Water Cleanup System Iso. Valves RWCU-MO-15, RWCU-MO-18	1	1	60	0	GC
RHR Suction Cooling Iso. Valve RHR-MO-17, RHR-MO-18	1	1	40	C	SC
RHR Discharge to Radwaste Iso. Valves RHR-MO-57, RHR-MO-67	1	1	20	C	SC
Suppression Chamber Purge & Vent PC-245AV, PC-230MV		2	15	C	SC
Suppression Chamber N_2 Supply PC-237AV, PC-233MV		2	15	C	SC

TABLE 3.7.4
PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

PEN. NO.	VALVE NUMBERS	TEST MEDIA
X-7A	MS-AO-80A and MS-AO-86A, Main Steam Isolation Valves	Air
X-7B	MS-AO-80B and MS-AO-86B, Main Steam Isolation Valves	Air
X-7C	MS-AO-80C and MS-AO-86C, Main Steam Isolation Valves	Air
X-7D	MS-AO-80D and MS-AO-86D, Main Steam Isolation valves	Air
X-8	MS-MO-74 and MS-MO-77, Main Steam Line Drain	Air
X-9A	RF-15CV and RF-16CV, Feedwater Check Valves	Air
X-9A	RCIC-AO-22, RCIC-MO-17, and RWCU-15CV, RCIC/RWCU Connection to Feedwater	Air
X-9B	RF-13CV and RF-14CV, Feedwater Check Valves	Air
X-9B	HPCI-AO-18 and HPCI-MO-57, HPCI Connection to Feedwater	Air
X-10	RCIC-MO-15 and RCIC-MO-16, RCIC Steam Line	Air
X-11	HPCI-MO-15 and HPCI-MO-16, HPCI Steam Line	Air
X-12	RHR-MO-17 and RHR-MO-18, RHR Suction Cooling	Air
X-13A	RHR-MO-25A and RHR-MO-27A, RHR Supply to RPV	Air
X-13B	RHR-MO-25B and RHR-MO-27B, RHR Supply to RPV	Air
X-14	RWCU-MO-15 and RWCU-MO-18, Inlet to RWCU System	Air
X-16A	CS-MO-11A and CS-MO-12A, Core Spray to RPV	Air
X-16B	CS-MO-11B and CS-MO-12B, Core Spray to RPV	Air
X-18	RW-732AV and RW-733AV, Drywell Equipment Sump Discharge	Air
X-19	RW-765AV and RW-766AV, Drywell Floor Drain Sump Discharge	Air
X-25	PC-232MV and PC-238AV, Purge and Vent Supply to Drywell	Air
X-25	ACAD-1305MV and ACAD-1306MV, Supply to Drywell	Air
X-26	PC-231MV, PC-246AV, and PC-306 MV Purge and Vent Exhaust from Drywell	Air
X-26	ACAD-1310MV, Bleed from Drywell	Air

Amendment No. ~~80~~, ~~82~~, ~~88~~, 103

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 103 TO FACILITY OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated September 5, 1986, the Nebraska Public Power District (the licensee) requested an amendment to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The proposed amendment would change the Technical Specifications to reflect the removal of the reactor vessel head spray piping.

2.0 EVALUATION

The amendment removes that portion of the reactor vessel head spray piping from the outboard containment isolation valve to the flange on the vessel head, and the spray nozzle within the vessel. The head spray piping is a subsystem of the residual heat removal (RHR) system. It may be placed in operation during a normal shutdown and cooldown. When the reactor is cooling down, part of the flow from the RHR system may be diverted to the head spray nozzle to condense steam in the reactor upper head. This action will facilitate flooding of the vessel. This operational mode is not used, as operational experience has shown that the head spray is unnecessary. The head spray has no safety function and is not used in any accident sequence. No credit is given for use of the head spray to safely shut down the plant during an accident. Head spray operability is not a Technical Specifications requirement for any mode of operation.

The proposed changes to the Technical Specifications would (1) delete the operability and surveillance requirements for containment isolation valves RHR-MOV-32 and RHR-MOV-33, (2) delete the aforementioned valves from Table 3.7.1 which identifies the primary containment isolation valves, (3) delete the aforementioned valves from Table 3.7.4 which identifies the testable containment penetrations associated with the isolation valves, and (4) delete the head spray function from the listing of containment isolation groups in Table 3.2.A. The head spray line containment penetration would remain installed as a spare and would continue to be subjected to periodic local leak rate testing as required by 10 CFR 50, Appendix J.

The staff has reviewed the proposed modification and associated Technical Specifications changes and concludes that they are acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Long

Dated: November 10, 1986