June 13, 1988

Docket No. 50-298

Mr. George A. Trevors, Division
Manager - Nuclear Support
Nuclear Power Group
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68601

Dear Mr. Trevors:

SUBJECT: COOPER NUCLEAR STATION - AMENDMENT NO. 121TO FACILITY OPERATING LICENSE NO. DPR-46 (TAC NO. 67164)

The Commission has issued the enclosed Amendment No.121 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The amendment consist of changes to the Technical Specifications in response to your application dated February 2, 1988 (Change Number 50).

The amendment changes the Technical Specifications to revise instrument identification numbers and depict extended reactor vessel water level instrument range.

A copy of our related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next Bi-weekly Federal Register notice.

Sincerely,

/s/
William O. Long, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 121to
License No. DPR-46
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION:

Docket File	JPartlow	NRC PDR	TBarnhart (4)
Local PDR	Wanda Jones	PD4 Reading	EButcher
PNoonan (3)	ACRS (10)	W. Long	GPA/PA
JCalvo	ARM/LFMB	OGC-Rockville	DHagan
S. Newberry	EJordan	Plant File	

DOCUMENT NAME: COOPER AMENDMENT CHANGE 50

PD4/LA	PD4/PM	SICB/BC	OGC-Rockville	PD4/D
PNoonan	WLong:sr	SNewberry	RBachmann	JCalvo
04/3/88	04/13/88	06/3/88	06/7/88	06/13/88

subject to noted changes to SER. JC

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P PDR

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suggested by note Wong to JER. 6/14

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 13, 1988

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Manager - Nuclear Support
Nuclear Power Group
Nebraska Public Power District
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See next page

cc:
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Nebraska Public Power District
P. O. Box 499
Columbus, Nebraska 68601

Cooper Nuclear Station
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Lincoln, Nebraska 68509-8922

Mr. Larry Bohlken, Chairman
Nemaha County Board of Commissioners
Nemaha County Courthouse
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Auburn, Nebraska 68305

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Arlington, Texas 76011

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Division of Radiological Health
Department of Health
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P. O. Box 95007
Lincoln, Nebraska 68509-5007

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Nebraska Public Power District (the licensee) dated February 2, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

José A. Calvo

Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications

Date of Issuance: June 13, 1988

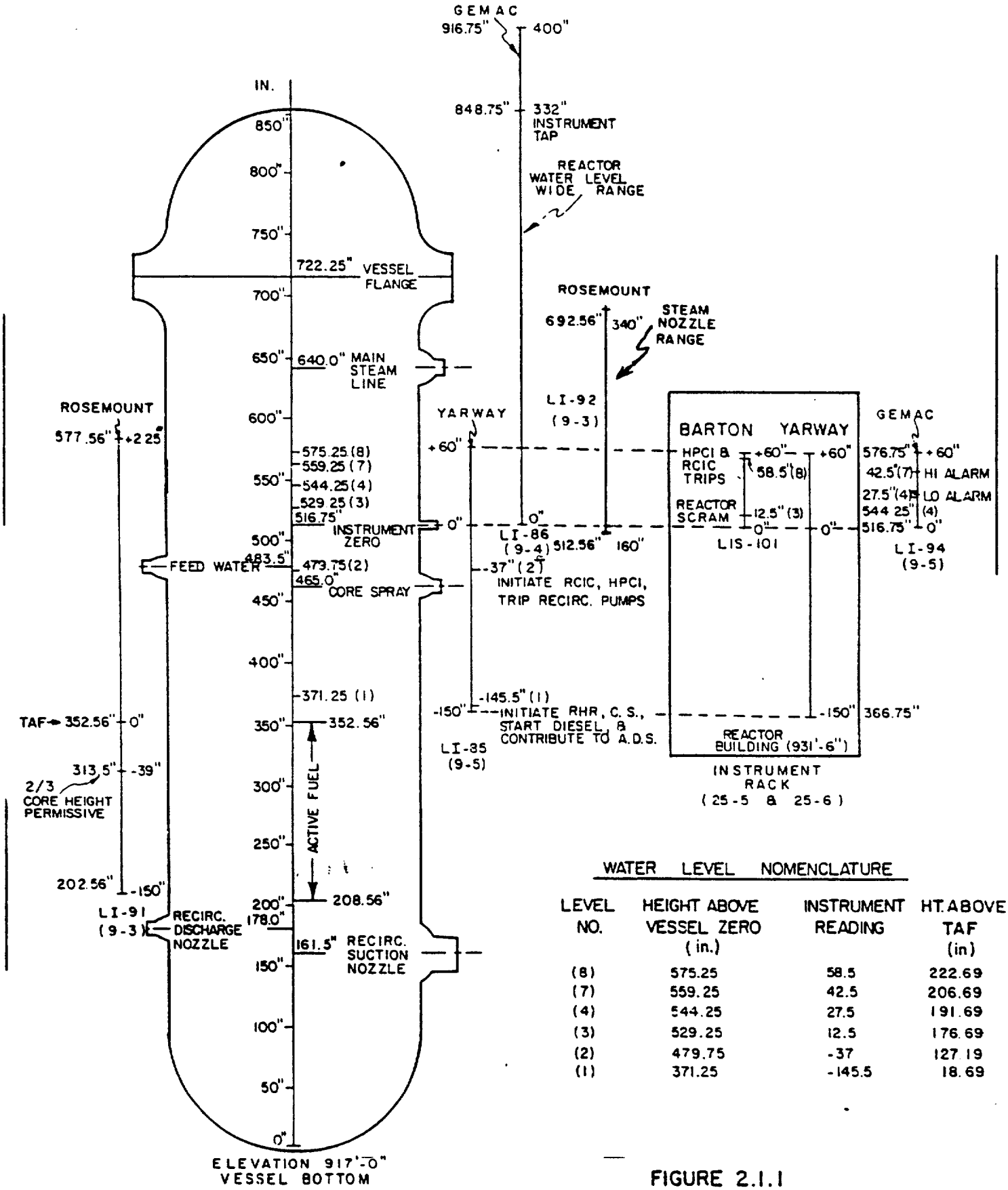
ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages
10
53
54
55
56
70
71
73



WATER LEVEL NOMENCLATURE

LEVEL NO.	HEIGHT ABOVE VESSEL ZERO (in.)	INSTRUMENT READING	HT. ABOVE TAF (in)
(8)	575.25	58.5	222.69
(7)	559.25	42.5	206.69
(4)	544.25	27.5	191.69
(3)	529.25	12.5	176.69
(2)	479.75	-37	127.19
(1)	371.25	-145.5	18.69

FIGURE 2.1.1

REACTOR WATER LEVEL INDICATION CORRELATION

COOPER NUCLEAR STATION
TABLE 3.2.B (PAGE 1)
CIRCUITRY REQUIREMENTS CORE SPRAY SYSTEM

Instrument	Instrument I.D. No.	Setting Limit	Minimum Number of Operable Components Per Trip System	Action Required When Component Operability Is Not Assured (1)
Reactor Low Water Level	NBI-LIS-72 A, B, C, & D	$\geq$-145.5 of Indicated Level	2	A
Reactor Low Pressure	NBI-PS-52 A1, A2, C1, & C2 NBI-PIS-52 B & D	$\leq$450 psig	2	A
Drywell High Pressure	PC-PS-101, A, B, C & D	$\leq$2 psig	2	A
Core Spray Pump Disch. Pressure	CS-PS-44, A & B CS-PS-37, A & B	100 $\leq$ P $\leq$ 165 psig	2	A
Core Spray Pump Time Delay	CS-TDR-K16 A & B	9$\leq$T$\leq$11 seconds	1	B
Low Voltage Relay Emerg. Bus	27X1 - 1F & 1G 27X2 - 1F & 1G	Loss of Voltage	1	B
Aux. Bus Low Voltage Relay	27X3 - 1A & 1B	Loss of Voltage	1	B
Pump Discharge Line Low Pressure	CM-PS-73, A & B	$\geq$10 psig	(3)	D

COOPER NUCLEAR STATION
TABLE 3.2.B (PAGE 2)
RESIDUAL HEAT REMOVAL SYSTEM (LPCI MODE) CIRCUITRY REQUIREMENTS

Instrument	Instrument I.D. No.	Setting Limit	Minimum Number of Operable Components Per Trip System (1)	Action Required When Component Operability Is Not Assured
Drywell High Pressure	PC-PS-101 A,B,C, & D	$\leq$2 psig	2	A
Reactor Low Water Level	NBI-LIS-72, A,B,C, & D #1	$\geq$-145.5" Indicated Level	2	A
Reactor Vessel Shroud Level Below Low Level Trip	NBI-LITS-73, A & B #1	$\geq$-39 Indicated Level	1	B
Reactor Low Pressure	RR-PS-128, A & B	$\leq$75 psig	1	B
Reactor Low Pressure (Injection Valve Permissive)	NBI-PS-52A1,A2,C1&C2 NBI-PIS-52B & D	$\leq$450 psig	1	A
Drywell Pressure Containment Spray	PC-PS-119, A,B,C, &D	$\leq$2 psig	2	A
RHR Pump Discharge	RHR-PS-120, A,B,C,&D RHR-PS-105, A,B,C,&D	100 $\leq$ P $\leq$ 165 psig 100 $\leq$ P $\leq$ 165 psig	2 2	A A
Reactor Low Pressure (Recirc. Discharge Permissive)	NBI-PS-52A & C NBI-PIS-52B & D	185 $\leq$ P $\leq$ 235 psig	1	A

COOPER NUCLEAR STATION
TABLE 3.2.B (PAGE 3)
RESIDUAL HEAT REMOVAL SYSTEM (LPCI MODE) CIRCUITRY REQUIREMENTS

Instrument	Instrument I.D. No.	Setting Limit	Minimum Number of Operable Components Per Trip System (1)	Action Required When Component Operability Is Not Assured
RHR Pump Low Flow	RHR-dPIS-125 A & B	$\geq$2500 gpm	1	A
Time Delays	RHR-TDR-K45, 1A&1B	4.25$\leq$T$\leq$5.75 min.	1	A
RHR Pump Start Time Delay	RHR-TDR-K75A & K70B	4.5$\leq$T$\leq$5.5 Sec.	1	A
	RHR-TDR-K75B & K70A	$\leq$.5 sec.	1	A
RHR Heat Exchanger Bypass T.D.	RHR-TDR-K93, A & B	1.8$\leq$T$\leq$2.2 min.	1	B
RHR Crosstie Valve Position	RHR-LMS-8	Valve Not closed	(3)	E
Bus 1A Low Volt. Aux. Relay	27 X 3/1A	Loss of Voltage	1	B
Bus 1B Low Volt. Aux. Relay	27 X 3/1B	Loss of Voltage	1	B
Bus 1F Low Volt. Aux. Relays	27 X 1/1F	Loss of Voltage	1	B
	27 X 2/1F	Loss of Voltage	1	B
Bus 1G Low Volt. Aux. Relays	27 X 1/1G	Loss of Voltage	1	B
	27 X 2/1G	Loss of Voltage		
Pump Discharge Line	CM-PS-266	$\geq$5 psig	(3)	D
	CM-PS-270	$\geq$15 psig	(3)	D
Emergency Buses Undervoltage Relays (degraded voltage)	27/1F-2, 27/1FA-2	3600 $\pm$5%	2	B
	27/1G-2, 27/1GB-2	8 second $\pm$2 sec.	2	B
		time delay	1	B
Emergency Buses Loss of Voltage Relays	27/1F-1, 27/1FA-1, 27/1G-1, 27/1GB-1, 27/ET-1, 27/ET-2	2900V $\pm$5% 5 second $\pm$1 sec. delay	1	B
Emergency Buses Under-Voltage Relays Timers	27X7/1F, 27X7/1G,	10 second $\pm$2 sec.	1	B

COOPER NUCLEAR STATION
TABLE 3.2.B (PAGE 4)
HPCI SYSTEM CIRCUITRY REQUIREMENTS

Instrument	Instrument I.D. No.	Setting Limit	Minimum Number of Operable Components Per Trip System (1)	Action Required When Component Operability Is Not Assured
Reactor Low Water Level	NBI-LIS-72, A,B,C, & D #3	$\geq$37" Indicated Level	2	A
Reactor High Water Level	NBI-LIS-101, B & D #2	$\leq$+58.5" Indicated Level	2(2)	A
High Drywell Press.	PC-PS-101 A,B,C, & D	$\leq$2 psig	2(2)	A
HPCI Turbine High Exhaust Pressure	HPCI-PS-97, A & B	$\leq$150 psig	1(2)	A
HPCI Pump Low Suction Press.	HPCI-PS-84-1	$\leq$15" Hg Vacuum	1(2)	A
HPCI Pump Low Discharge Flow	HPCI-FS-78	$\geq$400 gpm	1(2)	A
HPCI Low Steam Supply Pressure	HPCI-PS-68, A,B,C & D	$\geq$100 psig	2(2)	A
HPCI Steam Line High ΔP	HPCI-dPIS-76 HPCI-dPIS-77	130 $\leq$S$\leq$210" H_2O -130 $\geq$S$\geq$-210" H_2O	1	A
HPCI Steam Line Space Hi Temp.	HPCI-TS-101, A,B,C & D -102, 103, 104, HPCI-TS-125,126,127,128 RHR-TS-150,151,152,153 154,155,156,157,158,159 160,161	$\leq$200°F	2(4)	A
Emerg. Cond. Storage Tank Low Level	HPCI-LS-74 A & B HPCI-LS-75 A & B	$\geq$0" H_2O (10,000 gal. usable remaining)	1(2)	A

COOPER NUCLEAR STATION
TABLE 4.2.B (Page 1)
CORE SPRAY SYSTEM TEST & CALIBRATION FREQUENCIES

Item	Item I.D. No.	Functional Test Freq.	Calibration Freq.	Instrument Check
Instrument				
1. Reactor Low Water Level	NBI-LIS-72, A,B,C, & D	Once/Month (1)	Once/3 Months	Once/Day
2. Reactor Low Pressure	NBI-PS-52, A1,A2,C1, & C2 NBI-PIS-52, B & D	Once/Month (1)	Once/3 Months	None
3. Drywell High Pressure	PC-PS-101, A,B,C, & D	Once/Month (1)	Once/3 Months	None
4. Core Spray Pump Disch. Press.	CS-PS-44, A & B CS-PS-37, A & B	Once/Month (1) Once/Month (1)	Once/3 Months Once/3 Months	None None
5. Core Spray Pump Time Delay	CS-TDR - K16, A & B	Once/Month (1)	Once/Oper. Cycle (4)	None
6. Emergency Bus Low Volt Relay	27X1 - 1F & 1G 27X2 - 1F & 1G	Once/Oper. Cycle Once/Oper. Cycle	Once/5 Years Once/5 Years	None None
7. Aux. Bus Low Voltage Relay	27X3 - 1A & 1B	Once/Oper. Cycle	Once/5 Years	None
8. Pump Disch. Line Low Press.	CM-PS-73, A & B	Once/3 Months	Once/3 Months	None
Logic (4) (6)				
1. Logic Power Monitor		Once/6 Months	N.A.	N.A.
2. Core Spray Initiation		Once/6 Months	N.A.	N.A.
3. Pump & Valve (Signal Override) Control		Once/6 Months	N.A.	N.A.

COOPER NUCLEAR STATION
TABLE 4.2.B (Page 2)
RHR SYSTEM TEST & CALIBRATION FREQUENCIES

Item	Item I.D. No.	Functional Test Freq.	Calibration Freq.	Instrument Check
Instrumentation				
1. Drywell High Pressure	PC-PS-101, A, B, C & D	Once/Month (1)	Once/3 Months	None
2. Reactor Vessel Shroud Level	NBI-LITS-73, A & B #1	Once/Month (1)	Once/3 Months	Once/Day
3. Reactor Low Pressure	RR-PS-128 A & B	Once/Month (1)	Once/3 Months	None
4. Reactor Low Pressure	NBI-PS-52 A1,A2,C1, & C2 NBI-PIS-52 B & D	Once/Month (1)	Once/3 Months	None
5. Drywell Press.-Containment Spray	PC-PS-119, A,B,C & D	Once/Month (1)	Once/3 Months	None
6. RHR Pump Discharge Press.	RHR-PS-120, A,B,C & D	Once/Month (1)	Once/3 Months	None
7. RHR Pump Discharge Press.	RHR-PS-105, A,B,C & D	Once/Month (1)	Once/3 Months	None
8. RHR Pump Low Flow Switch	RHR-dPIS-125 A & B	Once/Month (1)	Once 3 Months	None
9. RHR Pump Start Time Delay	RHR-TDR-K70, A & B	Once/Month (1)	Once/Oper. Cycle	None
10. RHR Injection Valve Close T.D.	RHR-TDR-K45 1A & 1B	Once/Month (1)	Once/Oper. Cycle	None
11. RHR Pump Start Time Delay	RHR-TDR-K75, A & B	Once/Month (1)	Once/Oper. Cycle	None
12. RHR Heat Exchanger Bypass T.D.	RHR-TDR-K93, A & B	Once/Month (1)	Once/Oper. Cycle	None
13. RHR Cross Tie Valve Position	RHR-LMS-8	Once/Month (1)	N.A.	
14. Low Voltage Relays	27 X 3/1A	(7)		None
15. Low Voltage Relays	27 X 3/1B	(7)		None
16. Low Voltage Relays	27 x 2/1F, 27 X 2/1G	(7)		None
17. Low Voltage Relays	27 X 1/1F, 27 X 1/1G	(7)		None
18. Pump Disch. Line Press. Low	CM-PS-266, CM-PS-270	Once/3 Months	Once/3 Months	None
19. Emergency buses Undervoltage Relays (Degraded Voltage)	27/1F-2, 27/1FA-2, 27/1G-2, 27/1GB-2	Once/Month	Once/18 Months	Once/12 hrs.
20. Emergency Buses Loss of Voltage Relays	27/1F-1, 27/1FA-1, 27/1G-1, 27/1GB-1, 27/ET-1, 27/ET-2	Once/Month	Once/18 Months	Once/12 hrs.
21. Emergency Buses Undervoltage Relays Timers	27X7/1F, 27X7/1G	Once/Month	Once/18 Months	None

COOPER NUCLEAR STATION
TABLE 4.2.B (Page 4)
HPCI TEST & CALIBRATION FREQUENCIES

Item		Item I.D. No.	Functional Test Freq.	Calibration Freq.	Instrument Check
1.	Reactor Low Water Level	NBI-LIS-72, A,B,C, & D, #3	Once/Month (1)	Once/3 Months	Once/Day
2.	Reactor High Water Level	NBI-LIS-101, (B & D #2)	Once/Month (1)	Once/3 Months	Once/Day
3.	High Drywell Pressure	PC-PS-101 A, B, C, & D	(7)	(7)	None
4.	HPCI Turbine High Exhaust Press.	HPCI-PS-97 A & B	Once/Month (1)	Once/3 Months	None
5.	HPCI Pump Low Suction Press.	HPCI-PS-84-1	Once/Month (1)	Once/3 Months	None
6.	HPCI Pump Low Discharge Flow	HPCI-FS-78	Once/Month (1)	Once/3 Months	None
7.	HPCI Low Steam Supply Press.	HPCI-PS-68, A,B,C, & D	Once/Month (1)	Once/3 Months	None
8.	HPCI Steam Line High ΔP	HPCI-dPIS-76	Once/Month (1)	Once/3 Months	None
		HPCI-dPIS-77	Once/Month (1)	Once/3 Months	None
9.	HPCI Steam Line Space High Temp.	HPCI-TS-101, A,B,C, & D 102, 103, 104, HPCI-TS-125, 126, 127, 128 RHR-TS-150,151,152,153,154, 155,156,157,158,159,160,161	Once/Month (1)	Once/Oper. Cycle	None
10.	Emergency Cond. Stg. Tk. Low Level	HPCI-LS-74 A & B	Once/Month (1)	Once/3 Months	None
		HPCI-LS-75 A & B	Once/Month (1)	Once/3 Months	None
11.	Suppression Chamber High Water Level	HPCI-LS-91 A & B	Once/Month (1)	Once/3 Months	None
12.	HPCI Gland Seal Cond. Hotwell Level	HPCI-LS-356 B	Once/Month (1)	Once/3 Months	None
		HPCI-LS-356 A	Once/Month (1)	Once/3 Months	None
13.	HPCI Control Oil Pressure Low	HPCI-PS-2787-H	Once/Month (1)	Once/3 Months	None
		HPCI-PS-2787-L	Once/Month (1)	Once/3 Months	None
14.	Turbine Condition Supr. Alarm Actuation Timer	HPCI-TDR-K14	Once/Month (1)	Once/Oper. Cycle	None
15.	Pump Disch. Line Low Press.	CM-PS-268	Once/3 Months	Once/3 Months	None
16.	HPCI Turbine Stop Valve Mon.	HPCI-LMS-4	Once/Month	N.A.	None
17.	Sup. Chamber HPCI Suction Vlv.	HPCI-LMS-2	Once/Month	N.A.	None
18.	HPCI Steam Line High ΔP Actuation Timer	HPCI-TDR-K33,	Once/Month	Once/Oper. Cycle	None
		HPCI-TDR-K43	Once/Month	Once/Oper. Cycle	None
Logic (4)(6)					
1.	Logic Bus Power Monitor		Once/6 Months	N.A.	
2.	HPCI Initiation		Once/6 Months	N.A.	
3.	HPCI Turbine Trip		Once/6 Months	N.A.	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated February 2, 1988 the Nebraska Public Power District (the licensee) requested an amendment to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The proposed amendment would revise the Technical Specifications to (1) change the instrument identification numbers (Tables 3.2.B and 4.2.B) for reactor pressure switches which provide closing signals to the reactor recirculation system discharge valves and permissive input signals to the core spray and low pressure coolant injection systems, (2) change the instrument identification number (Table 3.2.B) for the residual heat removal system crosstie valve position indicator switch, (3) change the instrument identification numbers (Tables 3.2.B and 4.2.B) for drywell pressure instruments associated with the high pressure coolant injection (HPCI) system, and (4) modify the "Reactor Water Level Indication Correlation" drawing (Figure 2.1.1) to reflect modifications extending the range of reactor vessel water level instrumentation.

2.0 DISCUSSION & EVALUATION

Pressure Switch Identification Numbers: This change would reflect a plant modification in which two Barksdale duplex pressure switches (NBI-PS-52A, and 52C) for which spares are no longer readily available would be replaced by four Static-O-Ring single element pressure switches (NBI-PS-52A1, A2, C1, and C2). The replacement switches will be seismically and environmentally qualified and will serve the same function as those being replaced. The proposed change would have no effect on safety and is acceptable.

Instrument Identification Number for RHR Crosstie Valve Position Indicator Switch: The crosstie valve position indicator switch is identified as "RHR-LMS-8" in the facility design drawings. The proposed amendment would revise the Technical Specifications identification number to be consistent with the drawings. No change would be made to the actual instrument. This change corrects a simple error, has no effect on safety, and is acceptable.

HPCI Instrument Identification Numbers: The instrument identification numbers for the drywell pressure instrument channels serving the HPCI initiation instrumentation would be changed to be consistent with the general policy of identifying instrument channels by use of the sensor instruments ID numbers instead of the associated logic relay ID numbers. This change is thus being made for consistency only, no changes being made to the actual instrument channels. These changes would have no effect on safety and are acceptable.

Reactor Water Level Indication Correlation Drawing: Figure 2.1.1 would be revised to reflect modifications to be made to extend the range of water level instruments in accordance with the licensee's Regulatory Guide 1.97 accident monitoring instrumentation commitments to provide improved capability to monitor reactor vessel water level. The modifications extend the range of water level monitoring down to 6 inches below the bottom of active fuel and up to the top of the steam separator. Based on conformance to previously approved Regulatory Guide 1.97 plans (Ref: Letter from W. Long to J. Pilant dated October 27, 1986) this change is acceptable. It is noted that this change does not add, delete or modify any limiting conditions for operation or surveillance requirements but serves only to update descriptive information contained in the Technical Specifications.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. The amendment also involves changes to administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sections 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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