

December 22, 1987

Docket No. 50-298

Mr. George A. Trevors, Division  
Manager - Nuclear Support  
Nuclear Power Group  
Nebraska Public Power District  
Post Office Box 499  
Columbus, Nebraska 68601

Dear Mr. Trevors:

SUBJECT: COOPER NUCLEAR STATION, AMENDMENT NO. 114, TO FACILITY  
OPERATING LICENSE NO. DPR-46

The Commission has issued the enclosed Amendment No. 114 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. This amendment consists of changes to the Technical Specifications in response to your application dated December 21, 1987.

The amendment changes the Technical Specifications to extend the containment isolation logic functional test interval from six months to eighteen months.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

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William O. Long, Project Manager  
Project Directorate - IV  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 114 to License No. DPR-46
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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*by JWC*

Mr. George A. Trevors  
Nebraska Public Power District

Cooper Nuclear Station

cc:

Mr. G. D. Watson, General Counsel  
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Cooper Nuclear Station  
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Nemaha County Board of Commissioners  
Nemaha County Courthouse  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT  
DOCKET NO. 50-298  
COOPER NUCLEAR STATION  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114  
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nebraska Public Power District (the licensee) dated December 21, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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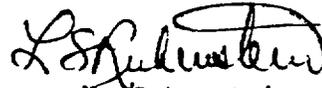
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

(2) Technical Specification

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Acting Assistant Director  
for Region IV Reactors and Special Projects  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 22, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 114

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages

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COOPER NUCLEAR STATION  
 TABLE 4.2.A (Page 2)  
 PRIMARY CONTAINMENT AND REACTOR VESSEL ISOLATION SYSTEM  
 TEST AND CALIBRATION FREQUENCIES

Item	Item I.D. No.	Function Test Freq.	Calibration Freq.	Instrument Check
<u>Logic Systems</u>				
1.	Main Steam Line Iso. Valves	Once/18 Months		
	Main Steam Line Drain Valves	Once/18 Months		
	Reactor Water Sample Valves	Once/18 Months		
2.	Drywell Vent Iso. Valves	Once/18 Months		
	Reactor Bldg. H & V	Once/18 Months		
	Standby Gas Treatment Start	Once/18 Months		
3.	Reactor Water Cleanup System	Once/18 Months		
	Iso. Reactor System Pump Trip	Once/18 Months		
4.	Drywell Floor Drain Iso. Valves	Once/18 Months		
	Drywell Equip. Drain Iso. Valves	Once/18 Months		
5.	RHR System Iso. Valves	Once/18 Months		
6.	Tip Withdrawal	Once/18 Months		

COOPER NUCLEAR STATION  
 TABLE 4.2.D  
 MINIMUM TEST AND CALIBRATION FREQUENCIES FOR RADIATION MONITORING SYSTEMS

System	Instrument I.D. No.	Functional Test Freq.	Calibration Freq.	Instrument Check
<u>Instrument Channels</u>				
Steam Jet Air Ejector Off-Gas System	RMP-RM-150 A & B	(12)	(12)	(12)
Reactor Building Isolation and Standby Gas Treatment Initiation	RMP-RM-452 A & B	(12)	(12)	(12)
Liquid Radwaste Discharge Isolation	RMP-RM-1	(11)	(11)	(11)
Main Control Room Ventilation Isolation	RMV-RM-1	Once/Month (1)	Once/3 Months	Once/Day
Mechanical Vacuum Pump Isolation	RMP-RM-251, A-D		See Tables 4.1.1 & 4.1.2	
<u>Logic Systems</u>				
SJAE Off-Gas Isolation		Once/Year		
Standby Gas Treatment Initiation		Once/18 Months		
Reactor Building Isolation		Once/18 Months		
Liquid Radwaste Disch. Isolation		Once/6 Months		
Main Control Room Vent Isolation		Once/6 Months		
Mechanical Vacuum Pump Isolation		Once/Operating Cycle		

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Amendment No. 114



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-46  
NEBRASKA PUBLIC POWER DISTRICT  
COOPER NUCLEAR STATION  
DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated December 21, 1987, the Nebraska Public Power District (the licensee) requested an amendment to Facility Operating License No. DPR-46 for the Cooper Nuclear Station (CNS). The proposed amendment would change the Technical Specifications to extend the surveillance interval for the Groups 2 and 6 containment isolation system logic functional tests from 6 months to 18 months.

2.0 DISCUSSION

The groups 2 and 6 containment isolation logics at CNS actuate isolation systems for RHR shutdown cooling, drywell floor and equipment drains, the TIP system, and secondary containment and, initiate the Standby Gas Treatment System. The secondary containment isolation includes isolation of the Reactor Recirculation Pump Motor-Generator Set Ventilation System (M-G Vent Sys). Isolation of the M-G Set Vent Sys would likely lead to a Recirculation Pump trip and subsequent reactor trip. Therefore it is undesirable to perform the secondary containment isolation logic functional test while operating. The licensee has requested an amendment to extend the test interval to 18 months. This would allow the test, which becomes due January 2, 1988, to be performed during the Spring 1988 refueling outage (March-April 1988) and during each subsequent refueling outage. This extended interval is consistent with the staff position as prescribed by NUREG-0123, "Standard Technical Specifications for Boiling Water Reactors" which provides guidance for Technical Specifications as referenced by Standard Review Plan Section 16. The proposed amendment is therefore acceptable.

3.0 EMERGENCY CIRCUMSTANCES

Without the proposed amendment, the facility would have to be shutdown to perform the test. This circumstance was not readily foreseen due to the fact that unplanned, mid-cycle outages have historically occurred permitting the test to be performed within the prescribed 6 month interval while shutdown.

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#### 4.0 NO SIGNIFICANT HAZARDS CONSIDERATION

The regulations in 10 CFR 50.92 state that the Commission may make a final determination that a licensee amendment involves no significant hazards consideration if the operation of the facility in accordance with the amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The evaluation in Section 2.0 shows that the extended surveillance interval would have no effect on the probability and no significant effect on the consequences of those accidents for which the isolation logic instrumentation is intended to mitigate. The proposed change does not create the possibility of a new or different accident and does not affect any margins of safety. Based on the evaluation, the staff has concluded that operation of the facility in the proposed manner would not involve a significant increase in the probability or consequences of an accident previously evaluated, would not create the possibility of a new or different kind of accident from any accident previously evaluated, and would not involve a significant reduction in margin of safety.

Accordingly, we conclude the amendment involves no significant hazards consideration.

#### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, consultation was held with the State of Nebraska by telephone. The State expressed no concern either from the standpoint of safety or no significant hazards consideration determination.

#### 6.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

7.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 22, 1987

Principal Contributor: William Long