

January 3, 1985

Docket No. 50-298

Mr. J. M. Pilant, Technical
Staff Manager
Nuclear Power Group
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68601

Dear Mr. Pilant:

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. This amendment consists of changes to the Technical Specifications in response to your application dated February 29, 1984 and supplemented by your July 18, 1984 submittal.

The amendment revises the Technical Specifications to (1) incorporate changes proposed in response to TMI Action Plan Items set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements" and requested by the staff's Generic Letter 83-36, (2) add four additional fire detectors located near the service water pumps to the list of fire detection instruments in Table 3.14 and correct the identification numbers of two fire detectors listed in that table, and (3) clarify a setpoint on Table 3.2.B for the Reactor Core Isolation Cooling System. The changes proposed in response to Generic Letter 83-36 pertain to the following TMI Action Plan Items:

- II.B.3 - Post-Accident Sampling
- II.F.1.1 - Noble Gas Monitor
- II.F.1.2 - Iodine Particulate Sampling
- II.F.1.3 - Containment High Range (Radiation) Monitor
- II.F.1.4 - Containment Pressure Monitor
- II.F.1.5 - Containment Water Level Monitor

A copy of the Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Ernest D. Sylvester, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures and cc:
See next page

Mr. J. M. Pilant

- 2 -

Enclosures:

1. Amendment No. 90 to
License No. DPR-46
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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Mr. J. M. Pilant
Nebraska Public Power District
Cooper Nuclear Station

cc:

Mr. G. D. Watson, General Counsel
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68601

Mr. Arthur C. Gehr, Attorney
Snell & Wilmer
3100 Valley Center
Phoenix, Arizona 85073

Cooper Nuclear Station
ATTN: Mr. Paul Thomason, Division
Manager of Nuclear Operations
Post Office Box 98
Brownville, Nebraska 68321

Director
Nebraska Dept. of Environmental Control
Post Office Box 94877
State House Station
Lincoln, Nebraska 68509

Mr. William Siebert, Commissioner
Nemaha County Board of Commissioners
Nemaha County Courthouse
Auburn, Nebraska 68305

Mr. Dennis DuBois
U. S. Nuclear Regulatory Commission
Resident Inspector
Post Office Box 218
Brownville, Nebraska 68321

John T. Collins
Regional Administrator
Region IV Office
U. S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

H. Ellis Simmons, Director
Division of Radiological Health
Department of Health
301 Centennial Mall, South
Post Office Box 95007
Lincoln, Nebraska 68509



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District dated February 29, 1984, as supplemented July 18, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the licensee is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

(2) Technical Specification

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Revise the Appendix A Technical Specifications as follows. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
i	i
iv	iv
49	49
58	58
65	65
66	66
-	67a
-	67b
80	80
-	82a
87a	87a
216k	216k
227	227

RADIOLOGICAL TECHNICAL SPECIFICATIONS

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LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.2.D (cont'd.)	4.2.D (cont'd.)
5. Mechanical Vacuum Pump Isolation	5. Mechanical Vacuum Pump Isolation
a. The mechanical vacuum pump shall be capable of being automatically isolated and secured by a signal of high radiation in the main steam line tunnel whenever the main steam isolation valves are open.	The instrument surveillance requirements are given on Tables 4.1.1, 4.1.2, and 4.2.D.
b. If the limits of (3.2.D.5.a) are not met, the vacuum pump shall be isolated.	
E. <u>Drywell Leak Detection</u>	E. <u>Drywell Leak Detection</u>
The limiting conditions of operation for the instrumentation that monitors drywell leak detection are given in Table 3.2.E.	Instrumentation shall be calibrated and checked as indicated in Table 4.2.E.
F. <u>Primary Containment Surveillance Information</u>	F. <u>Primary Containment Surveillance Information</u>
The limiting conditions of operation for the instrumentation that provides surveillance information readouts are given in Table 3.2.F.	Instrumentation shall be calibrated and checked as indicated in Table 4.2.F.
G. <u>Recirculation Pump Trip</u>	G. <u>Recirculation Pump Trip</u>
The limiting conditions for operation for the instrumentation that trips the recirculation pumps as a means of limiting the consequences of a failure to scram during an anticipated transient are given in Table 3.2.G.	Instrumentation shall be functionally tested and calibrated as indicated on Table 4.2.G.
H. <u>Post-Accident Monitoring</u>	H. <u>Post-Accident Monitoring</u>
The limiting conditions for operation for the instrumentation that monitors post-accident conditions are given in Table 3.2.H.	Instrumentation shall be functionally tested and calibrated as indicated on Table 4.2.H.

COOPER NUCLEAR STATION
TABLE 3.2.B (PAGE 6)
REACTOR CORE ISOLATION COOLING SYSTEM (RCIC) CIRCUITRY REQUIREMENTS

Instrument	Instrument I.D. No.	Setting Limit	Minimum Number of Operable Components Per Trip System (1)	Action Required When Component Operability Is Not Assured
RCIC High Turbine Exhaust Press.	RCIC-PS-72, A & B	≤ 25 psig	1(2)	A
RCIC Low Pump Suction Press.	RCIC-PS-67-1	≤ 15 " Hg Vacuum	1(2)	
RCIC Steam Line Space Excess Temp.	RCIC-TS-79, A,B,C,&D RCIC-TS-80, A,B,C,&D RCIC-TS-81, A,B,C,&D RCIC-TS-82, A,B,C,&D	$\leq 200^{\circ}\text{F}$	2(4)	A
RCIC Steam Line High ΔP	RCIC-dPIS-83 & 84	$370" \leq S \leq 620" \text{ H}_2\text{O}$	1	A
RCIC Steam Supply Press. Low	RCIC-PS-87, A,B,C,&D	≥ 50 psig	2(2)	A
RCIC Low Pump Disch. Flow	RCIC-FIS-57	≥ 40 gpm	1(2)	A
Pump Discharge Line Low Pressure	CM-PS-269	≥ 10 psig	(3)	D
RCIC Turbine Condition- al Supervisory Alarm Timer	RCIC-TDR-K9	$13.5 \leq T \leq 16.5$	(3)	E
Reactor Low Water Level	10A-K80, A & B 10A-K79, A & B (NBI-LIS-72, A,B,C, & D)	≥ -37 " Indicated Level	2(2)	A
Reactor High Water Level	NBI-LIS-101, A & C #2	$\leq +58.5$ Indicated Level	2(2)	A
RCIC Steamline High ΔP Actuation Timer	RCIC-TDR-K12 RCIC-TDR-K32	$2.7 \leq T \leq 3.3$ sec	1	A

COOPER NUCLEAR STATION
TABLE 3.2.F
PRIMARY CONTAINMENT SURVEILLANCE INSTRUMENTATION

Instrument	Instrument I.D. No.	Range	Minimum Number of Operable Instrument Channels	Action Required When Minimum Condition Not Satisfied (1)
Reactor Water Level	NBI-LI-85A NBI-LI-85B	-150" to +60" -150" to +60"	2	A,B,C
Reactor Pressure	RFC-PI-90A RFC-PI-90B	0 - 1200 psig 0 - 1200 psig	2	A,B,C
Drywell Pressure	PC-PI-512A PC-PR-512B PC-PR-1A PC-PR-1B	0 - 80 psia 0 - 80 psia 0 - 250 psig 0 - 250 psig	2 2	A,B,C F
Drywell Temperature	PC-TR-503 PC-TI-505	50 - 170°F 50 - 350°F	2	A,B,C
Suppression Chamber/Torus Air Temperature	PC-TR-21A PC-TR-23, Ch 1 & 2	0 - 300°F 0 - 400°F	2	A,B,C
Suppression Chamber/Torus Water Temperature	PC-TR-24, Ch 1 to 16	0 - 250°F	4	A,B,C
Suppression Chamber/Torus Water Level	PC-LI-10 PC-LR-11 PC-LI-12 PC-LI-13 PC-LR-1A PC-LR-1B	(-4' to +6') (-4' to +6') -10" to +10" -10" to +10" 0 - 30' 0 - 30'	2 2 2	A,B,C A,B,C,E F
Suppression Chamber/Torus Pressure	PC-PR-20	0 - 2 psig	1	B,C
Control Rod Position	N.A.	Indicating Lights	1	A,B,C,D
Neutron Monitoring	N.A.	S.R.M., I.R.M., LPRM 0 - 100% power	1	A,B,C,D
Torus to Drywell Differential Pressure	PC-dPR-20	0 - 2 psid	1	A,B,C,E
Suppression Chamber/ Drywell Pressure (ΔP)	PC-PR-20/513 (2)	0 - 2 psig	1	

Amendment No. 98, 99

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NOTES FOR TABLE 3.2.F

1. The following actions will be taken if the minimum number of operable instrument channels as required are not available.
 - A. From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.
 - B. From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.
 - C. If the requirements of A and B above cannot be met, an orderly shutdown shall be initiated within 24 hours.
 - D. These surveillance instruments are considered to be redundant to each other.
 - E. In the event that both channels are inoperable and indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in Hot Shutdown in six (6) hours and in a Cold Shutdown condition in the following eighteen (18) hours.
 - F. From and after the date that one of these parameters is reduced to one indication, either restore the inoperable component(s) to operable status within 30 days of the event, or prepare and submit a Special Report to the Commission outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status. In the event that both channels are inoperable and indication cannot be restored in fourteen (14) days, an orderly shutdown shall be initiated.
2. These instrument channels are utilized as input for the performance of a manual calculation to provide the second Torus to Drywell differential pressure indication.

COOPER NUCLEAR STATION
TABLE 3.2.H
POST-ACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS*

Instrument	Instrument ID Number	Range	Minimum Number of Operable Instrument Channels	Action Required When Component Operability Is Not Assured
Elevated Release Point (ERP) Monitor (High Range Noble Gas)	RMP-RM-3B	1.00E-2 to 1.00E+5 $\mu\text{c/cc}$ (Xe-133 Equivalent)	1	A
Turbine Building Ventilation Exhaust Monitor (High Range Noble Gas)	RMV-RM-20B	1.00E-2 to 1.00E+5 $\mu\text{c/cc}$ (Xe-133 Equivalent)	1	A
Radwaste/Augmented Radwaste Exhaust Monitor (High Range Noble Gas)	RMV-RM-30B	1.00E-2 to 1.00E+5 $\mu\text{c/cc}$ (Xe-133 Equivalent)	1	A
Primary Containment Gross Radiation Monitor	RMA-RM-40A RMA-RM-40B	1.0-1.0E+7 R/Hr. 1.0-1.0E+7 R/Hr.	2	A

*Note: Other Post-Accident Monitoring Instrumentation is located in Table 3.2.F-
Drywell Pressure, PC-PR-1A and 1B, Suppression Chamber/Torus Water Level PC-LR-1A and 1B

NOTES FOR TABLE 3.2.H

Action:

- A. With the number of operable components less than required by the minimum component operable requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
- 1) either restore the inoperable component(s) to operable status within 7 days of the event, or
 - 2) prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

COOPER NUCLEAR STATION
TABLE 4.2.F
PRIMARY CONTAINMENT SURVEILLANCE INSTRUMENTATION
TEST AND CALIBRATION FREQUENCIES

Instrument	Instrument I.D. No.	Calibration Frequency	Instrument Check
Reactor Water Level	NBI-LI-85A	Once/6 Months	Each Shift
	NBI-LI-85B	Once/6 Months	Each Shift
Reactor Pressure	RFC-PI-90A	Once/6 Months	Each Shift
	RFC-PI-90B	Once/6 Months	Each Shift
Drywell Pressure	PC-PR-512A	Once/6 Months	Each Shift
	PC-PI-512B	Once/6 Months	Each Shift
	PC-PR-1A	Once/6 Months	Each Shift
	PC-PR-1B	Once/6 Months	Each Shift
Drywell Temperature	PC-TR-503	Once/6 Months	Each Shift
	PC-TI-505	Once/6 Months	Each Shift
Suppression Chamber/Torus Air Temperature	PC-TR-21A	Once/6 Months	Each Shift
	PC-TR-23, Ch. 1 & 2	Once/6 Months	Each Shift
Suppression Chamber/Torus Water Temperature	PC-TR-24, Ch 1 to 16	Once/6 Months	Each Shift
Suppression Chamber/Torus Water Level	PC-LI-10	Once/6 Months	Each Shift
	PC-LR-11	Once/6 Months	Each Shift
	PC-LI-12	Once/6 Months	Each Shift
	PC-LI-13	Once/6 Months	Each Shift
	PC-LR-1A	Once/6 Months	Each Shift
	PC-LR-1B	Once/6 Months	Each Shift
Suppression Chamber/Torus Pressure	PC-PR-20	Once/6 Months	Each Shift
Control Rod Position	N.A.	N.A.	Each Shift
Neutron Monitoring (APRM)	N.A.	Once/Week	Each Shift
Torus to Drywell Differential Pressure	PC-dPR-20	Once/6 Months	Each Shift
Suppression Chamber/ Drywell Pressure (AP)	PC-PR-20/513 (2)	Once/6 Months	Each Shift

COOPER NUCLEAR STATION
TABLE 4.2.H
CALIBRATION FREQUENCY FOR POST-ACCIDENT MONITORING INSTRUMENTATION*

<u>Item</u>	<u>Item ID Number</u>	<u>Function Test Frequency</u>	<u>Calibration Frequency</u>
Elevated Release Point (ERP) Monitor (High Range Noble Gas)	RMP-RM-3B	Once/Month	Once/Cycle
Turbine Building Ventilation Exhaust Monitor (High Range Noble Gas)	RMV-RM-20B	Once/Month	Once/Cycle
Radwaste/Augmented Radwaste Exhaust Monitor (High Range Noble Gas)	RMV-RM-30B	Once/Month	Once/Cycle
Primary Containment Gross Radiation Monitors**	RMA-RM-40A,B	Once/Month	Once/Cycle

*Note: Other Post-Accident Monitoring Instrumentation calibration requirements are in Table 4.2.F-
Drywell Pressure, PC-PR-1A and 1B, Suppression Chamber/Torus Water Level PC-LR-1A and 1B.

**CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the
detector, for range decades above 10 R/hr and a one point calibration check of the detector below
10 R/hr with an installed or portable gamma source.

3.2 BASES (Cont'd)

F. Primary Containment Surveillance Information

For each parameter monitored, as listed in Table 3.2.F, there are two (2) channels of instrumentation. By comparing readings between the two (2) channels, a near continuous surveillance of instrument performance is available. Any deviation in readings will initiate an early recalibration, thereby maintaining the quality of the instrument readings.

The operability of the reactor water level instrumentation in Tables 3/4.2.F ensures that sufficient information is available to monitor and assess accident situations.

G. Recirculation Pump Trip

The recirculation pump trip has been added as a means of limiting the consequences of the unlikely occurrence of a failure to scram during an anticipated transient. The response of the plant to this postulated event falls within the envelope of study events given in General Electric Company Topical Report, NEDO-10349, dated March, 1971.

H. Post-Accident Monitoring

The post-accident monitoring instrumentation supplements existing instrumentation that was designed to monitor primarily the normal operational ranges of these parameters. Post-accident monitoring instrumentation provides information for the ranges that may exist during the extreme conditions postulated to occur during and after some accidents.

REFERENCES

1. "Low-Low Set Relief Logic System and Lower MSIV Water Level Trip for Cooper Nuclear Station", NEDE 22197, December 1982, General Electric Company.

INSTRUMENT LOCATION

INSTRUMENT ID NO.

2	<u>Control Room</u>	FP-SD-17-1 FP-SD-17-2 FP-SD-17-3
3	<u>Cable Spreading Room</u>	FP-SD-16-1 FP-SD-16-2 FP-SD-16-3 FP-SD-16-4 FP-SD-16-5 FP-SD-16-6
	<u>Cable Expansion Room</u>	FP-SD-16-7 FP-SD-16-8
4	<u>Switchgear Rooms</u>	
	DC Switchgear Rooms	FP-SD-15-2 FP-SD-15-3
	Critical Switchgear Room	FP-SD-22-1 FP-SD-22-2
5	<u>Station Battery Rooms</u>	FP-SD-15-1 FP-SD-15-4 FP-SD-15-1A FP-SD-15-4A
6	<u>Diesel Generator Rooms</u>	FP-SD-10-1 FP-SD-10-2 FP-SD-10-3 FP-SD-10-4 CO2-SD-DG-1A CO2-SD-DG-1B CO2-SD-DG-1C CO2-SD-DG-1D CO2-SD-DG-2A CO2-SD-DG-2B CO2-SD-DG-2C CO2-SD-DG-2D
7	<u>Diesel Fuel Storage Rooms</u>	CO2-TD-DG-1A CO2-TD-DG-1B
8	<u>Safety Related Equipment not in Reactor Building</u>	
	RHR Service Water Booster Pumps	FP-SD-14-3
	Emergency Condensate Storage Tanks	FP-SD-14-1
	Service Water Pumps	FP-SD-32-1 FP-SD-32-2 FP-SD-32-3 FP-SD-32-4 FP-FD-32-5 FP-FD-32-6
9	<u>Auxiliary Relay Room & Reactor Protection System Rooms</u>	
	Auxiliary Relay Room	FP-SD-15-9
	Reactor Protection System Room 1A	FP-SD-15-7
	Reactor Protection System Room 1B	FP-SD-15-8

Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-46 dated October 24, 1980.

- B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

D. Post-Accident Sampling System (PASS)

A program shall be established to ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. This program shall include training of personnel, procedures for sampling and analysis and provisions for operability of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-46
NEBRASKA PUBLIC POWER DISTRICT
COOPER NUCLEAR STATION
DOCKET NO. 50-298

1.0 INTRODUCTION

By letter dated February 29, 1984, Nebraska Public Power District (NPPD, the licensee) proposed changes to the Cooper Nuclear Station Technical Specifications which would (1) respond to TMI Action Plan items, (2) update the list of fire detection instruments, and (3) clarify a reactor core isolation cooling (RCIC) system instrument setpoint. The licensee submitted a revised proposal for the TMI Technical Specification changes by letter dated July 18, 1984. This amendment revises the Cooper Technical Specifications to incorporate the changes proposed by the licensee submittal of February 29, 1984, as revised by the July 18, 1984 submittal.

2.0 EVALUATION

A. NUREG-0737 ACTION ITEMS

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-36. Generic Letter 83-36 was issued to all Boiling Water Reactors (BWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letters dated February 29, 1984 and July 18, 1984, NPPD responded to Generic Letter 83-36 by submitting Technical Specification change requests for Cooper Nuclear Station for the following TMI Action Plan items discussed below.

8501160540 850103
PDR ADDCK 05000298
P PDR

1. Post-Accident Sampling (II.B.3)

The guidance provided by Generic Letter 83-36 requested that an administrative program should be established, implemented and maintained to ensure that the licensee has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The Post-Accident Sampling System is not required to be operable at all times. Administrative procedures are to be established for returning inoperable instruments to operable status as soon as practicable.

The licensee has provided a proposed revision to the Technical Specifications which is consistent with the guidelines provided in our Generic Letter 83-36. We conclude that the licensee has an acceptable Technical Specification for Post-Accident Sampling System.

2. Noble Gas Effluent Monitors (II.F.1.1)

The licensee has supplemented the existing normal range monitors to provide noble gas monitoring in accordance with Item II.F.1.1. Proposed Technical Specifications were submitted that are consistent with the guidelines provided in our Generic Letter 83-36. We conclude that the Technical Specifications for Item II.F.1.1 are acceptable.

3. Sampling and Analysis of Plant Effluents (II.F.1.2)

The guidance provided by Generic Letter 83-36 requested that an administrative program should be established, implemented and maintained to ensure the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident. The licensee has proposed Technical Specifications that are consistent with our guidance. We conclude that the Technical Specifications for sampling and analysis of plant effluents are acceptable.

4. Containment High-Range Radiation Monitor (II.F.1.3)

The licensee has installed two in-containment monitors in Cooper Nuclear Station that are consistent with the guidance of TMI Action Plan item II.F.1.3. Generic Letter 83-36 provided guidance for limiting conditions of operation and surveillance requirements for these monitors. The licensee proposed Technical Specifications that are consistent with the guidance provided in our Generic Letter 83-36. We conclude that the proposed Technical Specifications for item II.F.1.3 are acceptable.

5. Containment Pressure Monitor (II.F.1.4)

Cooper Nuclear Station has been provided with two supplementary channels for monitoring containment pressure following an accident. The licensee

has proposed Technical Specifications that are consistent with the guidelines contained in Generic Letter 83-36 concerning the requirement for the number of operable channels and surveillance on containment pressure monitors. However, the licensee has proposed an alternative Action Statement by letter dated July 18, 1984 which will require the licensee to submit a special report if one of the two channels is inoperable for more than 30 days, and it will require plant hot shutdown if both channels are inoperable for more than 14 days.

We have reviewed the instrumentation for monitoring containment pressure at Cooper Nuclear Station. The Cooper Station has two safety grade channels which cover the pressure range up to 80 psia, in addition to two wide range channels provided by TMI Action Plan item II.F.1. Considering the frequency of using these monitors and the design of the Cooper Station, we conclude that the proposed Action Statement by the licensee is acceptable. We conclude that the proposed Technical Specifications for containment pressure monitors meet the intent of Generic Letter 83-36 and are acceptable.

6. Containment Water Level Monitor (II.F.1.5)

The wide range suppression chamber/torus water level monitors provide the capability required by TMI Action Plan item II.F.1.5. The licensee has proposed Technical Specifications that are consistent with the guidelines contained in Generic Letter 83-36 concerning the requirements for the number of operable channels and surveillance on wide range suppression chamber water level monitors. However, the licensee has proposed an alternate Action Statement by letter dated July 18, 1984 which will require the licensee to submit a special report if one of the two channels is inoperable for more than 30 days, and it will require plant hot shutdown if both channels are inoperable for more than 14 days.

We have reviewed the instrumentation for monitoring suppression chamber water level at Cooper Station. The Cooper Station has two safety grade channels for monitoring water level for design basis accident, in addition to two channels required by TMI Action Plan item II.F.1.5. Considering the frequency of using these monitors and the design of the Cooper Station, we conclude that the proposed Action Statement is acceptable. We conclude that the proposed Technical Specifications for suppression chamber water level monitors meets the intent of Generic Letter 83-36 and are acceptable.

Requirements and references to drywell level instrumentation, erroneously proposed in the February 29, 1984 application in response to TMI Action Plan item II.F.1.5, were deleted by letter dated July 18, 1984. Because TMI Action Plan item II.F.1.5 only addresses suppression chamber/torus water level instrumentation, there was no need to include drywell water level instrumentation requirements in the Technical Specifications.

7. Suppression Chamber/Torus Water Temperature Monitor

By letter dated July 18, 1984 the licensee proposed a Technical Specification change to correct the identification number of the suppression chamber/torus water temperature monitor on Table 3.2.F. The proposed change would make the Technical Specifications consistent with the as-installed plant equipment. The need for this change was noted during the licensee's review of the Technical Specifications as requested by Generic Letter 83-36. This change is an administrative change to up-date the Technical Specification and is, therefore, acceptable. In addition, this change has been superseded by Amendment No. 88.

Based on our review we have determined that the Technical Specifications proposed in response to Generic Letter 83-36 by the licensee's February 29, 1984 submittal, as revised by the July 18, 1984 submittal, are acceptable.

B. FIRE DETECTION INSTRUMENTS

By letter dated February 29, 1984 the licensee proposed to revise the Technical Specification Table 3.14, "Fire Detection Instruments" to include four additional fire detectors in the number presently located in the service water pump area. These fire detectors were installed to meet the requirements of Appendix R to 10 CFR Part 50. This change to Table 3.14 constitutes an additional restriction by requiring operability of these additional fire detectors and is, therefore, acceptable.

The licensee has also proposed to correct the identification of the two existing fire detectors listed in Table 3.14 for the service water pump area. This Technical Specification change would make Table 3.14 consistent with as-installed plant equipment. This change is an administrative change to correct the Technical Specifications and is, therefore, acceptable.

Based on our review we have determined that the revisions to the fire detection instrument Technical Specifications proposed by the licensee's February 29, 1984 submittal are acceptable.

C. RCIC INSTRUMENTATION

By letter dated February 29, 1984 the licensee proposed a revision to Technical Specification Table 3.2.B, "Reactor Core Isolation Cooling System (RCIC) Circuitry Requirements" to clarify the meaning of a setting limit number. The RCIC low pump suction pressure setting limit is currently stated as "less than or equal to -15" Hg." This proposed revision would change the setting limit to "less than or equal to 15" Hg Vacuum." Since "less than or equal to -15" Hg" means the same as "less than or equal to 15" Hg Vacuum," this is a purely administrative change and is, therefore, acceptable.

Based on our review we have determined that the revisions to the RCIC instrumentation Technical Specifications proposed by the licensee's February 29, 1984 submittal are acceptable.

4.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Patel, E. Sylvester

Dated: January 3, 1985