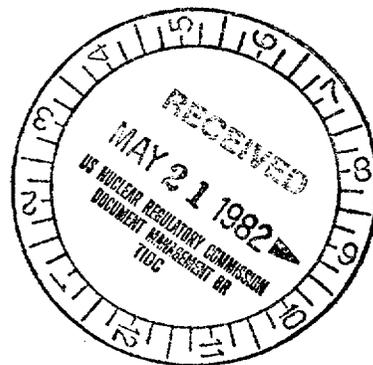


May 10, 1982

Docket No. 50-298



Mr. J. M. Pilant, Director
Licensing & Quality Assurance
Nebraska Public Power District
P.O. Box 499
Columbus, Nebraska 68601

Dear Mr. Pilant:

The Commission has issued the enclosed Amendment No. 79 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station in response to your telecopied request of March 23, 1982 and confirmed by your application dated March 23, 1982 and confirmed by Commission letter of March 23, 1982. The amendment was issued orally by telephone on March 23, 1982.

The amendment modifies the Technical Specifications to permit operation of the facility during startup for a period not to exceed 48 hours between March 22 and March 25, 1982 with the containment deinterred and without drywell suppression chamber differential pressure.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Byron L. Siegel, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No. 79
- 2. Safety Evaluation
- 3. Notice

cc: w/enclosures
See next page

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- Extra-5

*FR NOTICE
AMENDMENT
no level inspection subject to normal standards
has been reviewed by Siegel
4/15/82
approved*

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|---------|---------|----------|-----------|---------|---------|
| OFFICE | ORB#2 | ORB#2 | ORB#2 | AD. OR | OELD |
| SURNAME | SNorris | BSVegete | DVassallo | TNovak | HA 5522 |
| DATE | 4/19/82 | 4/19/82 | 4/19/82 | 4/23/82 | 4/30/82 |

Mr. J. M. Pilant
Nebraska Public Power District

cc:

Mr. G. D. Watson, General Counsel
Nebraska Public Power District
P. O. Box 499
Columbus, Nebraska 68601

Mr. Arthur C. Gehr, Attorney
Snell & Wilmer
3100 Valley Center
Phoenix, Arizona 85073

Cooper Nuclear Station
ATTN: Mr. L. Lessor
Station Superintendent
P. O. Box 98
Brownville, Nebraska 68321

Auburn Public Library
118 - 15th Street
Auburn, Nebraska 68305

Director
Nebraska Dept. of Environmental Control
P. O. Box 94877, State House Station
Lincoln, Nebraska 68509

Mr. William Siebert, Commissioner
Nemaha County Board of Commissioners
Nemaha County Courthouse
Auburn, Nebraska 68305

Mr. Dennis Dubois
USNRC
Resident Inspector
P. O. Box 218
Brownville, NE 68321

U. S. Environmental Protection Agency
Region VII Office
Regional Radiation Representative
324 East 11th Street
Kansas City, MO 64106

John T. Collins
Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District dated March 23, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of March 23, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "D. B. Vassallo". The signature is written in a cursive, flowing style.

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 10, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

164

167a

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7.A (cont'd.)

4.7.A (cont'd.)

- d. If specifications 3.7.A.4.a,b or c, cannot be met, the situation shall be corrected within 24 hours or the reactor will be placed in a cold shutdown condition within the subsequent 24 hours.

- c. Once each operating cycle, each vacuum breaker valve shall be visually inspected to insure proper maintenance and operation of the position indicator switch. The differential pressure set-point shall be verified.

5. Oxygen Concentration

- d. Prior to reactor startup after each refueling, a leak test of the drywell to suppression chamber structure shall be conducted to demonstrate that the requirement of 3.7.A.4.c is met.

- a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.

5. Oxygen Concentration

- b. Within the 24-hour period subsequent to placing the reactor in the Run mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

- a. The primary containment oxygen concentration shall be measured and recorded at least twice weekly.

- c. When the containment atmosphere oxygen concentration is required to be less than 4%, the minimum quantity of liquid nitrogen in the liquid nitrogen storage tank shall be 500 gallons.

- b. The quantity of liquid nitrogen in the liquid nitrogen storage tank shall be determined twice per week when the volume requirements of 3.7.A.5.c are in effect.

- d. If the specifications of 3.7.A.5.a thru c cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.

- e. The specifications of 3.7.A.5.a thru d are not applicable during a 48 hour continuous period between the dates of March 22, 1982 and March 25, 1982.

B. Standby Gas Treatment System

B. Standby Gas Treatment System

- 1. Except as specified in 3.7.B.3 below, both circuits of the standby gas treatment system and the diesel generators

- 1. At least once per operating cycle the following conditions shall be demonstrated:
 - a. Pressure drop across the combined HEPA filters and charcoal adsorber banks is

3.7 (cont'd)

E. Drywell-Suppression Chamber
Differential Pressure

1. Differential pressure between the drywell and suppression chamber shall be maintained at equal to or greater than 1.0 psid except as specified in a, b, and c below.
 - a. This differential shall be established within 26 hours after placing the mode switch in run.
 - b. This differential may be decreased to less than 1.0 psid 24 hours prior to placing mode switch in refuel or shutdown.
 - c. This differential may be decreased to less than 1.0 psid for a maximum of four (4) hours during required operability testing of the HPCI system pump, the RCIC system pump, and the drywell-pressure suppression chamber vacuum breakers.
2. If the differential pressure of specification 3.7.E.1 cannot be maintained, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in Hot Standby in six (6) hours and in a Cold Shutdown condition within the following 18 hours.
3. The specifications of 3.7.E.1 and 3.7.E.2 are not applicable during a 48-hour continuous period between the dates of March 22, 1982 and March 25, 1982.

4.7 (cont'd)

E. Drywell-Suppression Chamber
Differential Pressure

1. The pressure differential between the drywell and suppression chamber shall be recorded at least once each shift.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 79 TO FACILITY

OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

Author: B. Siegel

INTRODUCTION

By telecopied letter dated March 23, 1982 the Nebraska Public Power District (the licensee) proposed changes to the Technical Specifications of Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The proposed changes would permit the facility to be operated during startup with the containment deinerted up to 60% of the licensed thermal power for a 48-hour continuous period between the dates of March 22, 1982 and March 25, 1982.

DISCUSSION

The Cooper Nuclear Station (CNS) shutdown the weekend of March 20, 1982 due to generator voltage fluctuations. On March 22, 1982 plant startup commenced including containment inerting. During the return to power a failure in the ductwork used to inert the containment with nitrogen was discovered. This failure resulted in enough nitrogen leaking into the reactor building that it was perceived that the technical specification inerting requirement could not be met. This requirement permits a 24-hour period to reduce the containment oxygen concentration to less than 4% by volume subsequent to placing the reactor in the RUN mode following shutdown. An alternate path for inerting the containment, which bypassed and isolated the failed section of the ductwork, was utilized and the licensee requested a Technical Specification change that would allow operation of an additional 24 hours in a deinerted condition. As a safety precaution the licensee decided to limit operation to 60% of licensed thermal power during this time.

EVALUATION

The following discusses the areas of safety related to the licensee's request and our evaluation of these areas.

1. Operating the Containment Purge Valves - We are presently reviewing the adequacy of containment purging for the Cooper Nuclear Power Station. The licensee in a January 4, 1979 letter provided justification that

the purge valves are designed to close against the dynamic forces of a LOCA. Based on this information and recent information provided by the licensee in response to our containment purge and vent review, we conclude that the purge valves would close in the unlikely event of a LOCA during the time period they were open to complete containment purging.

2. Maintaining Delta P Control - The licensee has been operating the plant with a pressure differential between the drywell and wetwell to reduce the effects of pool dynamics following a LOCA. However, the licensee, as a result of the Mark I Containment Program, has truncated the downcomers and made structural modifications to the torus which eliminates the need for maintaining the Delta P control. The licensee has stated this will be documented in the Plant Unique Analysis for CNS which will be submitted in April 1982. On this basis we conclude the licensee can operate without maintaining Delta P control for the short period involved.

In addition to the above justifications, the Technical Specifications do allow deinerting and a lack of Delta P control for the same periods, i.e., approximately 48 hours under shutdown conditions followed by an immediate startup. Because of all of the above, including the low probability of an accident occurring during this time period and operation at no greater than 60% of licensed thermal power to significantly reduce the severity of any accident, we conclude that the temporary change to the Technical Specifications is acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 10, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-298NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 79 to Facility Operating License No. DPR-46 issued to Nebraska Public Power District (the licensee) which revised the Technical Specifications for operation of the Cooper Nuclear Station located in Nemaha County, Nebraska. The amendment is effective as of March 23, 1982.

The amendment modifies the Technical Specifications to permit operation of the facility during startup for a period not to exceed 48 hours between March 22 and March 25, 1982 with the containment de-inerted and without drywell suppression chamber differential pressure.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 23, 1982, (2) the Commission's letter to the licensee dated March 23, 1982, (3) Amendment No. 79 to License No. DPR-46 and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Auburn Public Library, 118 - 15th Street, Auburn, Nebraska 68304. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of May 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing