

NOV 21 1980

Docket No. 50-298

Mr. J. M. Pilant, Director  
Licensing & Quality Assurance  
Nebraska Public Power District  
P. O. Box 499  
Columbus, Nebraska 68601

Dear Mr. Pilant:

Distribution:

Docket	BJones (4)
NRC PDR	BScharf (10)
Local PDR	JWetmore
ORB Reading	ACRS (16)
NRR Reading	OPA (Clare Miles)
DEisenhut	RDiggs
RPurple	HDenton
TNovak	JHeltemes, AEOD
RTedesco	NSIC
GLainas	TERA
JRoe	Chairman, ASLAB
SNorris	TVWambach
VRooney	
OELD	
OI&E (5)	

The Commission has issued the enclosed Amendment No. <sup>66</sup> to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. This amendment consists of changes to the Technical Specifications in response to your submittals dated October 22, 1979 and January 16, 1980.

This amendment changes the Technical Specifications to provide additional limiting conditions of operation and surveillance requirements for fire protection equipment. Other miscellaneous editorial changes were made for purposes of consistency.

Included with this amendment is Supplement No. 1 to the Fire Protection Safety Evaluation which completes the staff's fire protection review for Cooper Nuclear Station.

The enclosed license amendment reflects those changes to your original request for license amendment which has been agreed to in discussions with your staff.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original Signed by  
T. A. Ippolito

Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. <sup>66</sup> to DPR-46
2. Supplement No. 1 to the Fire Protection Safety Evaluation for Cooper Nuclear Station
3. Notice of Issuance

cc w/enclosures: See next page

8 012800 614

1980 DEC 24 AM 9 05  
 DISTRIBUTION  
 CHARGES UNIT  
 NRC  
 REGISTRATION  
 BRANCH

*Def. no legal objection  
to and a notice CP 1*

OFFICE	ORB #2 <i>W</i>	ORB #2 <i>W</i>	ORB #5 <i>W</i>	AD:DL <i>W</i>	OELD	ORB #2 <i>W</i>
SURNAME	SNorris	VRooney:kf	TVWambach	TNovak	DSWANGEN	TAIppolito
DATE	10/8/80	10/9/80	10/11/80	10/13/80	11/15/80	10/12/80

Mr. J. M. Pilant  
Nebraska Public Power District

- 2 -

November 21, 1980

cc:

Mr. G. D. Watson, General Counsel  
Nebraska Public Power District  
P. O. Box 499  
Columbus, Nebraska 68601

Mr. Arthur C. Gehr, Attorney  
Snell & Wilmer  
3100 Valley Center  
Phoenix, Arizona 85073

Cooper Nuclear Station  
ATTN: Mr. L. Lessor  
Station Superintendent  
P. O. Box 98  
Brownville, Nebraska 68321

Auburn Public Library  
118 - 15th Street  
Auburn, Nebraska 68305

Director  
Nebraska Dept. of Environmental Control  
P. O. Box 94877, State House Station  
Lincoln, Nebraska 68509

Mr. William Siebert, Commissioner  
Nemaha County Board of Commissioners  
Nemaha County Courthouse  
Auburn, Nebraska 68305

Director, Criteria and Standards  
Division  
Office of Radiation Programs (ANR-460)  
U. S. Environmental Protection Agency  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Region VII  
ATTN: EIS COORDINATOR  
1735 Baltimore Avenue  
Kansas City, Missouri 64108



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66  
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The submittals by Nebraska Public Power District dated October 22, 1979 and January 16, 1980 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the submittals, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-46 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 66, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

8012300 622

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Dated: November 21, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Remove the following pages of the Appendix "A" Technical Specifications and replace with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

216c  
216e  
216f  
216i  
216j  
216k  
216l  
216m  
219

Insert

216c  
216e  
216f  
216i  
216j  
216k  
216l  
216m  
219

## LIMITING CONDITIONS FOR OPERATION

3.15 (cont'd)

### SPECIFICATIONS

A. The fire suppression water system shall be OPERABLE with:

1. Two fire pumps, each with a capacity of 2000 gpm, with their discharge aligned to the fire suppression header.
2. An OPERABLE flow path capable of taking suction from the Missouri River and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant valves and the front valve ahead of the water flow alarm device on each sprinkler, hose standpipe or spray system riser.

B. If the requirement of 3.15 A cannot be met, restore the inoperable equipment to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.7.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system.

C. With the fire suppression system inoperable:

1. Establish a backup fire suppression water system within 24 hours, and
2. Submit a Special Report in accordance with Specification 6.7.2;
  - a) By telephone within 24 hours, and
  - b) In writing no later than the first working day following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

## SURVEILLANCE REQUIREMENTS

4.15 (cont'd)

### SPECIFICATIONS

A. The Fire Suppression Water Supply System shall be demonstrated operable:

1. At least once per 31 days by starting each pump on a staggered start-up basis and operating it for:
  - a) A minimum of 15 minutes for a diesel engine-driven fire pump, and
  - b) A minimum of 7 minutes for an electrical motor-driven fire pump.
2. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.
3. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
4. At least once per 18 months by performing a system functional test which includes simulated automatic actuation of the system throughout its operating sequence, and:
  - a) Verifying that each automatic valve in the flow path actuates to its correct position on a test signal,
  - b) Verifying that each pump develops at least 2000 gpm at 110 psi,

## LIMITING CONDITIONS FOR OPERATION

3.15 (Cont'd)

### 3.16 SPRAY AND/OR SPRINKLER SYSTEM

#### APPLICABILITY

Applies to the availability of automatic Fire Protection to Cable Spreading Room - 918 Ft. Elev. of Control Building, Cable Expansion Room - 918 Ft. Elev. of Controlled Corridor, and Northeast Corner - 903 Ft. Elev. of Reactor Building.

#### OBJECTIVE

To assure automatic Fire Protection to the Cable Spreading Room - 918 Ft. Elev. of Control Building, Cable Expansion Room - 918 Ft. Elev. of Controlled Corridor, and Northeast Corner - 903 Ft. Elev. of Reactor Building. 216e

## SURVEILLANCE REQUIREMENTS

4.15 (Cont'd)

- 2) Verifying the diesel starts from ambient conditions on the auto-start signal and operates for  $\geq 15$  minutes while loaded with the fire pump.
7. The fire pump diesel starting 24-volt battery bank and charger shall be demonstrated OPERABLE:
  - a) At least once per 7 days by verifying that:
    - 1) The electrolyte level of each battery is above the plates, and
    - 2) The overall battery voltage  $\geq 24$  volts.
  - b) At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the battery.
  - c) At least once per 18 months by verifying that:
    - 1) The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and
    - 2) The battery-to-battery and terminal connections are clean, tight, free of corrosion and coated with, anti-corrosion material.

### 4.16 SPRAY AND/OR SPRINKLER SYSTEM

#### APPLICABILITY

Applies to the availability of automatic Fire Protection to Cable Spreading Room - 918 Ft. Elev. of Control Building, Cable Expansion Room - 918 Ft. Elev. of Controlled Corridor, and Northeast Corner - 903 Ft. Elev. of Reactor Building.

## LIMITING CONDITIONS FOR OPERATION

3.16 (Cont'd)

### SPECIFICATIONS

- A. The Automatic Sprinkler Systems protecting the Cable Spreading Room, Cable Expansion Room, and Northeast Corner - 903 Ft. Elev. of Reactor Building shall be operable.
- B. If the requirement of 3.16.A cannot be met, establish a continuous fire watch with backup fire suppression equipment for the unprotected area within 1 hour; restore the system to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.7.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

## 3.17 CARBON DIOXIDE SYSTEM

### APPLICABILITY

Applies to the operational status of the High Pressure Carbon Dioxide Extinguishing System protecting the Diesel Generator Rooms.

### OBJECTIVE

To assure continuous Automatic Fire Protection for the Diesel Generator Rooms.

### SPECIFICATIONS

- A. The High Pressure Carbon Dioxide Extinguishing System protecting the Diesel Generator Rooms shall be operable.

## SURVEILLANCE REQUIREMENTS

4.16 (Cont'd)

### SPECIFICATIONS

- A. The Automatic Sprinkler Systems protecting the Cable Spreading Room, Cable Expansion Room, and Northeast Corner - 903 Ft. Elev. of Reactor Building shall be demonstrated to be operable by:
  - 1. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
  - 2. At least once per 18 months:
    - a) By performing a system functional test which includes simulated automatic actuation of the system, and
      - 1) Verifying that the automatic valves in the flow path actuate to their correct positions on a test signal, and
      - 2) Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel.
    - b) By inspection of the spray headers to verify their integrity.

## 4.17 CARBON DIOXIDE SYSTEM

### APPLICABILITY

Applies to the operational status of the High Pressure Carbon Dioxide Extinguishing System protecting the Diesel Generator Rooms.

### SPECIFICATIONS

- A. The High Pressure Carbon Dioxide Extinguishing System protecting the Diesel Generator Rooms shall be demonstrated operable by:



3.20 YARD FIRE HYDRANT AND HYDRANT HOSE HOUSEAPPLICABILITY

Applies to the operational status of the yard fire hydrant, HT-1 and associated hydrant hose house located near the northwest corner of the Intake Structure.

OBJECTIVE

To assure continuous manual fire fighting capability for the fire water pumps and service water pumps in the Intake Structure provided by the existing yard fire hydrant, HT-1, and the associated hydrant hose house.

SPECIFICATIONS

- A. Yard fire hydrant HT-1 and its associated hydrant hose house shall be operable.
- B. If the requirement of 3.20A cannot be met, provide sufficient additional lengths of 2½ inch diameter hose located in an adjacent operable hydrant hose house to provide service to the unprotected area within one hour, if the inoperable fire hydrant or associated hydrant hose house is the primary means of fire suppression; otherwise provide the additional hose within 24 hours. Restore the hydrant or hose house to operable status within 14 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.7.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the hydrant or hose house to operable status.

4.20 YARD FIRE HYDRANT AND HYDRANT HOSE HOUSEAPPLICABILITY

Applies to the operational status of the yard fire hydrant, HT-1, and associated hydrant hose house located near the northwest corner of the Intake Structure.

SPECIFICATIONS

- A. Yard fire hydrant HT-1 and associated hydrant hose house shall be demonstrated operable:
  1. At least once per 31 days by:
    - a) Visually inspecting the hydrant hose house to assure all required equipment is at the hose house.
  2. At least once per six months (once during March, April or May and once during September, October or November) by:
    - a) Visually inspecting the yard fire hydrant and verifying that the hydrant barrel is dry and that the hydrant is not damaged.
  3. At least once per 12 months by:
    - a) Conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at the yard fire hydrant.
    - b) Inspecting all the gaskets and replacing any degraded gaskets in the couplings.
    - c) Performing a flow check of the hydrant to verify its operability.

# FIRE DETECTION INSTRUMENTS

TABLE 3.14

INSTRUMENT ID NO.

1 Reactor Building

859 & 881 Elev.  
(Covers RCIC,  
Core Spray, RHR, and  
HPCI Pumps)

FD-ID-13-2  
FD-ID-13-3  
FD-ID-13-2  
FD-ID-13-3  
FD-ID-19-2  
FD-ID-19-3  
FD-ID-20-2  
FD-ID-20-3  
FD-ID-20-4  
FD-ID-20-5  
FD-ID-21-2  
FD-ID-21-3

903 Elev.

FD-ID-18-1  
FD-ID-18-4  
FD-ID-19-1  
FD-ID-20-1  
FD-ID-21-1

931 Elev.

FD-ID-23-1  
FD-ID-23-2  
FD-ID-23-3  
FD-ID-23-4

958 Elev.

FD-ID-24-1  
FD-ID-24-2  
FD-ID-24-3  
FD-ID-24-4  
FD-ID-24-5  
FD-ID-24-6

976 Elev.  
Covers Standby  
Liquid Control Pump and  
Standby Gas Treatment

FD-ID-26-1  
FD-ID-26-2  
FD-ID-26-1  
FD-ID-26-2  
FD-ID-25-3  
FD-ID-25-4  
FD-ID-25-5

1001 Elev.  
Also Fuel  
Storage Area

FD-ID-25-3  
FD-ID-25-4  
FD-ID-25-5  
FD-ID-25-7

## Legend

FP = Fire Protection  
TD = Thermal Detector  
FD = Flame Detector  
SD = Smoke Detector

CO2 = Carbon Dioxide  
DG = Diesel Generator  
1st Digit Instr. ID No. = Zone  
2nd Digit Instr. ID No. = Instr. No.

## INSTRUMENT LOCATION

## INSTRUMENT ID NO.

2	<u>Control Room</u>	FP-SD-17-1 FP-SD-17-2 FP-SD-17-3
3	<u>Cable Spreading Room</u>	FP-SD-16-1 FP-SD-16-2 FP-SD-16-3 FP-SD-16-4 FP-SD-16-5 FP-SD-16-6
4	<u>Switchgear Rooms</u>	
	DC Switchgear Rooms	FP-SD-15-2 FP-SD-15-3
	Critical Switchgear Room	FP-SD-22-1 FP-SD-22-2
5	<u>Station Battery Rooms</u>	FP-SD-15-1 FP-SD-15-4 FP-SD-15-1A FP-SD-15-4A
6	<u>Diesel Generator Rooms</u>	FP-SD-10-1 FP-SD-10-2 FP-SD-10-3 FP-SD-10-4 CO2-SD-DG1-A CO2-SD-DG1-B CO2-SD-DG1-C CO2-SD-DG1-D CO2-SD-DG2-A CO2-SD-DG2-B CO2-SD-DG2-C CO2-SD-DG2-D
7	<u>Diesel Fuel Storage Rooms</u>	CO2-TD-DG-1A CO2-TD-DG-1B
8	<u>Safety Related Pumps not in Reactor Building</u>	
	RHR Service Water Booster Pumps	FP-SD-14-3
	Emergency Condensate Storage Tanks	FP-SD-14-1
	Fire Water Pumps & Service Water Pumps	FP-FD-32-1 FP-FD-32-2
9	<u>Auxiliary Relay Room &amp; Reactor Protection System Rooms</u>	
	Auxiliary Relay Room	FP-SD-15-9
	Reactor Protection System Room 1A	FP-SD-15-7
	Reactor Protection System Room 1B	FP-SD-15-8

Table 3-18  
FIRE HOSE STATIONS

<u>Location</u>	<u>Identification #</u>	<u>Hose Size</u>
1. <u>Reactor Building</u>		
859 & 881 Elevation	HV-31	1 1/2"
	HV-32	1 1/2"
	HV-33	1 1/2"
903 Elevation	HV-35	1 1/2"
	HV-36	1 1/2"
	HV-38	1 1/2"
	HV-34	1 1/2"
	HV-62	1 1/2"
931 Elevation	HV-37	1 1/2"
	HV-57	1 1/2"
	HV-59	1 1/2"
968 Elevation	HV-39	1 1/2"
	HV-40	1 1/2"
	HV-42	1 1/2"
976 Elevation	HV-41	1 1/2"
	HV-46	1 1/2"
	HV-56	1 1/2"
	HV-58	1 1/2"
	HV-63	1 1/2"
1001 Elevation	HV-43	1 1/2"
	HV-44	1 1/2"
	HV-45	1 1/2"
2. <u>Cable Spreading Room</u>	HV-18	1 1/2"
3. <u>Computer &amp; Control Rooms</u>	HV-19	1 1/2"
4. <u>Switchgear Rooms &amp; Station Battery Room</u>		
D.C. Switchgear Room and Station Battery Rooms	HV-17	1 1/2"
5. <u>Safety Related Pumps not in Reactor Building</u>		
RHR Service Water Booster Pumps	HV-16	1 1/2"

### 3.14-3.19/4.14-4.19 BASES

#### 3.14/4.14 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.

#### 3.15-3.18/4.15-4.18 FIRE SUPPRESSION SYSTEMS

THE OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, CO<sub>2</sub> and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected equipment can be restored to service.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

#### 3.19/4.19 FIRE BARRIER PENETRATION SEALS

The functional integrity of the fire barrier penetration seals ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The fire barrier penetration seals are a passive element in the facility fire protection program and are subject to periodic inspections.

During periods of time when the seals are not functional, a continuous fire watch is required to be maintained in the vicinity of the affected seal until the seal is restored to functional status.

Fire barrier penetration seals include cable penetration barriers, fire doors, and fire dampers.

6.0 ADMINISTRATIVE CONTROLS

6.1 Organization

6.1.1 The Station Superintendent shall have the over-all fulltime onsite responsibility for the safe operation of the Cooper Nuclear Station. During periods when the Station Superintendent is unavailable, he may delegate his responsibility to the Assistant to Station Superintendent or, in his absence, to one of the Department Supervisors.

6.1.2 The portion of the Nebraska Public Power District management which relates to the operation of this station is shown in Figure 6.1.1.

6.1.3 The organization for conduct of operation of the station is shown in Fig. 6.1.2. The shift complement at the station shall at all times meet the following requirements. Note: Higher grade licensed operators may take the place of lower grade licensed or unlicensed operators.

- A. A licensed senior reactor operator (SRO) shall be present at the station at all times when there is any fuel in the reactor.
- B. A licensed reactor operator shall be in the control room at all times when there is any fuel in the reactor.
- C. Two licensed reactor operators shall be in the control room during all startup, shutdown and other periods involving significant planned control rod manipulations. A licensed SRO shall either be in the Control Room or immediately available to the Control Room during such periods.
- D. A licensed senior reactor operator (SRO) with no other concurrent duties shall be directly in charge of any refueling operation, or alteration of the reactor core.

A licensed reactor operator (RO) with no other concurrent duties shall be directly in charge of operations involving the handling of irradiated fuel other than refueling or reactor core alteration operations.

- E. An individual who has been trained and qualified in health physics techniques shall be on site at all times that fuel is on site.
- F. Minimum crew size during reactor operation shall consist of three licensed reactor operators (one of whom shall be licensed SRO) and three unlicensed operators. Minimum crew size during reactor cold shutdown conditions shall consist of two licensed reactor operators (one of whom shall be licensed SRO) and one unlicensed operator.

In the event that any member of a minimum shift crew is absent or incapacitated due to illness or injury a qualified replacement shall be designated to report on-site within two hours.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SUPPLEMENT NO. 1  
TO THE  
FIRE PROTECTION  
SAFETY EVALUATION REPORT  
BY THE  
OFFICE OF NUCLEAR REACTOR REGULATION  
U.S. NUCLEAR REGULATORY COMMISSION  
IN THE MATTER OF  
NEBRASKA PUBLIC POWER DISTRICT  
COOPER NUCLEAR STATION  
DOCKET NO. 50-298

License Amendment No. 66  
DATE: November 21, 1980

8012300 631

By letter dated January 16, 1980, the Nebraska Public Power District (licensee) provided information regarding three specific fire protection modifications identified as "open" in the Cooper Nuclear Station Fire Protection Safety Evaluation Report (SER), issued May 23, 1979. We have completed our review and it is presented below:

Item 3.8 - Reactor Building Cable Trays; EL 903'

In the SER, we stated that an automatic fire suppression system be installed in the northeast corner of the reactor building at elevation 903 feet to preclude the possibility of a fire affecting redundant sets of cable trays. The utility proposed an automatic wet-pipe sprinkler system designed and installed per NFPA Standard No. 13. The sprinkler system water supply header will be connected to the existing fire protection header. We have reviewed the proposed design modification and calculations, and determined that an appropriate design basis has been selected.

Based on our evaluation, we conclude that the installation of the fire suppression system per the appropriate NFPA Standards will preclude the possibility of a fire affecting redundant sets of cables at elevation 903 feet, and, therefore, it is acceptable.

Item 3.11 - Reactor and Control Building Automatic Suppression Water Feed  
Independency

The concern identified in the SER was for establishing an independence between the automatic and manual fire protection systems water supply. The licensee agreed to provide this water supply independence to preclude the possibility of a single failure in the fire water system from impairing both automatic and manual fire protection systems. The new water supply header will be installed per NPFA Standard No. 13. We have reviewed the proposed design modification and calculations, and conclude that a proper design basis has been selected.

Based on our evaluation, we conclude that the installation of an independent water supply for the manual fire protection systems per the appropriate NFPA Standards will preclude the possibility of a single failure from impairing both automatic and manual fire protection systems and, therefore, it is acceptable.

Item 3.16 - Penetration Firestop Barriers

In the SER, we stated that additional information was necessary to determine the adequacy of the penetration firestop barriers. The licensee has provided this information in the form of drawings, materials description, and fire tested design references. The fire test reference was the fire endurance tests incorporating silicone foam, ceramic fiber, cellular concrete, and marinite XL panels conducted on May 10, 1978, for Bechtel Power Corporation by Factory Mutual Research at the National Gypsum Research Center, FC 258, FC 259, and FC 260. This series of tests were witnessed by the NRC and have been approved by ANI and the NRC.



Based on our evaluation, we conclude that sufficient information and documentation has been provided to substantiate the design adequacy of the licensee's cable penetration firestop ratings, and, therefore, is acceptable.

In summary, with the evaluation of the above three items, we have completed the review of the Cooper Nuclear Station, Unit 1, Fire Protection program.

#### Technical Specifications and Additional Information

Amendment No. 56 to Facility Operating License No. DPR-46 requested that the licensee propose revised Technical Specifications related to fire protection facility modifications and provide certain additional information. Additional information has been provided as summarized in Table 1.

A safety evaluation describing the licensee's complete fire protection program, including administrative procedures and the equipment to which these Technical Specifications pertain was issued on May 23, 1979, accompanying License Amendment No. 56.

Technical Specifications incorporating limiting conditions of operation and surveillance requirements for fire protection systems were proposed by the licensee by letter dated October 22, 1979 and subsequently modified based on telephone discussions between the staff and the licensee. We have reviewed the proposed changes and determined that the proposed additions and changes include all necessary items contained in our fire protection evaluation, are consistent with licensing guidance contained in the NRC Standard Technical Specifications, and are therefore acceptable.

We find that the Fire Protection Program for the Cooper Nuclear Station with the improvements already made by the licensee is adequate and meets the guidelines contained in Appendix A to Branch Technical Position 9.5-1 and is, therefore, acceptable.

#### Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 21, 1980

TABLE 1

ADDITIONAL FIRE PROTECTION INFORMATION FOR COOPER

<u>Item #</u>	<u>Item Description</u>	<u>Status</u>
3.3.1	Foam & Spr Sys	Closed by 6/28/79 ltr
3.6	Cable Chase/Turbine N. Wall	Closed by 6/28/79 ltr
3.7	MG Set Oil Coolers	Closed by 10/19/79 ltr
3.12	Cable Expansion Room	Closed by 10/19/79 ltr
3.15.2	Particle Board Wall	Closed by 10/19/79 ltr
3.19	Laundry Room Spr Sys	Closed by 10/19/79 ltr
3.27	Fire Pump Discharge Header	Closed by 6/28/79 ltr

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-298

NEBRASKA PUBLIC POWER DISTRICT

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 66 to Facility Operating License No. DPR-46, issued to Nebraska Public Power District (the licensee), which revised the Technical Specifications for operation of the Cooper Nuclear Station, located in Nemaha County, Nebraska. The amendment is effective as of its date of issuance.

This amendment changes the Technical Specifications to provide additional limiting conditions of operation and surveillance requirements for fire protection equipment. Other miscellaneous editorial changes were made for purposes of consistency.

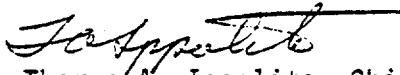
The licensee's submittals comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the licensee's submittals dated October 22, 1979 and January 16, 1980, (2) Amendment No. 66 to License No. DPR-46, and (3) the Commission's Supplement 1 to the Fire Protection Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Auburn Public Library, 118 - 15th Street, Auburn, Nebraska 68305. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C., 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 21st day of November 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing