# Administrative Topics Outline Page 1 of 1

FORM ES-301-1

Facilit		PE CREEK		Date of Examination:	03/11/02		
Exam	ination Level: 🔀	RO 🗌	SRO	Operating Test Number: _			
	Administrative Topic/Subject Description	1. ON	Describe method of evaluation:  1. ONE Administrative JPM, OR  2. TWO Administrative Questions				
A.1	Conduct of Operations	2.1.25	•	n and interpret station refere s/monographs/and tables w			
	Plant Parameter	Question-	•	nditions, determine the requirements for extreme of			
	Verification	2.1.25	Ability to obtain and interpret station reference materials such as graphs/monographs/and tables which contain data. (2.8)				
		Question-Obtain maximum RWCU return to Feedwater Temperature. (NEW)					
	Shift Turnover		nplete shift turno	shift turnover practices. (3.0 over attachment as off-going ONRC Exam) (Modified)	•		
A.2	Equipment Control	2.2.12	Knowledge of	surveillance procedures. (3.	0)		
	Surveillance Procedures	li .	JPM-Perform alternate determination of Drywell Air Temperature. (Simulator Perform) (NEW)				
A.3	Radiation Control	2.3.11	Ability to contro	ol radiation releases (2.7)			
	Radiation Releases	JPM-Cald	culate Iodine Re	lease Rates (Simulator Per	form) (NEW)		
A.4	Emergency Plan Emergency Action	2.4.39	implementation	,	• • • •		
	Levels and Classifications	1		ed Operator Review of the C Creek (Simulator Perform)	•		

# Administrative Topics Outline Page 1 of 1

FORM ES-301-1

Facilit	y: <u>HO</u>	PE CREE	K	Date of Examination: _	03/11/02		
Exami	ination Level:	RO 🛚	SRO	Operating Test Number: _			
	Administrative Topic/Subject Description	1. 0	Describe method of evaluation:  1. ONE Administrative JPM, OR  2. TWO Administrative Questions				
	Conduct of Operations		such as graph (3.1)	in and interpret station references/monographs/and tables which	ch contain data.		
	Plant Parameter Verification	Questioi		conditions, determine the requi m requirements for extreme co			
A.1			2.1.25 Ability to obtain and interpret station reference materials such as graphs/monographs/and tables which contain data. (3.1)				
		Question-Calculate time to establish Secondary Containment following a loss of Shutdown Cooling. (NEW)					
	O1 : 17 T		2.1.3 Knowledge of shift turnover practices. (3.4)				
	Shift Turnover	JPM-Complete key verification required during CRS shift turnover. (In-plant Perform)(NEW)					
A.2	Equipment Control	2.2.12	Knowledge	of surveillance procedures. (3.4	4)		
	Surveillance Procedures		I-Perform Drywell Air Temperature Operability Check. nulator Perform) (NEW)				
A.3	Radiation Control	2.3.11 A	bility to contro	l radiation releases (3.2)			
	Radiation Exposure Control	JPM-Calculate Iodine Release Rates (Simulator Perform) (NEW)					
A.4	Emergency Plan	2.4.41		the Emergency Action Level th	nresholds and		
	Emergency Action Levels and Classifications			. (4.1) onditions, classify an event and essage Form. (Modified)	complete the		

Control Room Systems and Facility Walk-Through Test Outline FORM ES-301-2
Page 1 of 1

Facility: HOPE CREEK Date of E	Examination:03/	11/02
Examinations Developed by:     Facility   NRC		
Examination Level:	est Number:	
B.1 Control Room Systems		
System / JPM Title	Type Code*	Safety Function
a. CRD-Stuck Rod during Shutdown Rod Exercising	(N), (A), (L), (S)	1
b. EHC-Respond to Low EHC Pressure	(D), (A), (S)	3
c. Recirc-Raise Reactor Recirculation Pump Speed >100%	(N), (S)	4
d. Condensate-Place 2 <sup>nd</sup> Secondary Condensate Pump Inservice	(D), (L), (S)	2
e. RPS-Bypassing Scram during ATWS	(D), (S)	7
f. Instrument Air-Place the Emergency Instrument Air- Compressor in service & England 50 SACS	(M), (A), (S)	8
g. EDG-Non-emergency operations of an EDG	(D), (S), (E)	6
B.2 Facility Walk-Through		
a. RPS MG Set Failure to start	(M), (A), (P)	7
b. Sample Primary Containment Atmosphere	(D), (P), (R)	5
c. Respond to a Failed Open Safety Relief Valve (Abnormal)	(D), (P), (E)	3
* Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)li (S)imulator, (L)ow-Power, (P)lant, (R)CA, (E)SF	ternate path, (C)ont	rol room,

ES-301 Control Room Systems and Facility Walk-Through Page 1 of 1	Test Outline FORM	И ES-301-2
Examinations Developed by:	est Number:	11/02
B.1 Control Room Systems		
System / JPM Title	Type Code*	Safety Function
a. CRD-Stuck Rod during Shutdown Rod Exercising	(N), (A), (L), (S)	1
Instrument Air- Place the Emergency Instrument Air     Compressor in service	(M), (A), (S)	8
c. EDG-Non-emergency operations of an EDG	(D), (S), (E)	6
d.		
e.		
f.		
g.		
B.2 Facility Walk-Through		
a. Respond to a Failed Open Safety Relief Valve (Abnormal)	(D), (P), (E)	3
b. Sample Primary Containment Atmosphere	(D), (P), (R)	5
c.		
* Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)li (S)imulator, (L)ow-Power, (P)lant, (R)CA, (E)SF	ternate path, (C)ont	rol room,

Appendix D		Scenario Outline Page 1 of 1	Form ES-D-1
Facility:	Hope Creek	Scenario Number: 1	Operating Test Number:
Examiners:		Evalua	ators:
	-1		
Objectives:			
Initial Conditions:	Plant is operating at Shutdown sequence	·	rogress. IO-0004, Step 5.1.5.
Turnover:	•		ecirc. Reduce power with control act Reactor Engineer for further

Event Number	Malf. Number	Event Type*	Event Description
1.		N(PO) N(SRO)	Place 3 <sup>rd</sup> Reactor Feed Pump in Recirc.
2.		R(RO) N(SRO)	Reduce power with Control Rods
3.	RR-19	I(RO) I(SRO)	Flow Comparator Failure
4.		C(PO) C(SRO)	Loss of Reactor Building Ventilation/Secondary Containment
5.	CD-10A	C(RO) C(SRO)	CRD Pump trip
6.	RC-09/10	M(ALL)	RCIC steam leak w/ failure to isolate/High HPCI Room Temp/Scram
7.		I(PO) I(SRO)	ADS failure to initiate during Emergency Depressurization (Similar event to Audit, different initiating cause and failure.)

<sup>(</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Appendix D	Scenario Outline	Form ES-D-1
	Page 1 of 1	

Facility:	Нор	e Creek	Scenario Number: 2 Operating Test Number:
Examiner	rs:		Evaluators:
Objective	s:	•	
Initial Condition		ied ESG-0- is operatin	42. g at 95% power. D Circulating Water Pump OOS for maintenance.
Turnover	: Place	B RFP in s	service. Raise Reactor power to 100% with Recirculation flow.
Event Number	Malf. Number	Event Type*	Event Description
1.		N(PO) N(SRO)	Place RFP in service
2.		C(PO) C(SRO)	Inadvertent Loss of Circulating Water Pump B/Degrading Condenser Vacuum
3.		R(RO)	Reduce power to maintain Condenser Vacuum
4.	RR-05/ RR-06	C(RO) C(SRO)	Dual Recirc Seal Failure
5.	RR-08/ RR-26	I(ALL)	Recirc Pump Speed Control Failure/High Vibrations/Scram
6.	RR-31	M(ALL)	Recirc Line Break/Primary Containment Failure
7.		C(PO) C(SRO)	Containment Spray Valve Failure

8.

<sup>(</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Appendix D			Scenario Outline Form ES-D-1 Page 1 of 1
Facility:	Нор	e Creek	Scenario Number: 3 Operating Test Number:
Examine	rs:		Evaluators:
			SPARE
Objective	s: SPARE	SCENARI	O
Initial Condition	36% Po	wer. Read	ctor startup in progress.
Turnover	Transfer Rods.	B RPS to	the Alternate Power Supply. Raise Reactor power with Control
Event Number	Malf. Number	Event Type*	Event Description
1.		N(ALL)	Transfer B RPS to the Alternate Power Supply
2.		R(RO) N(SRO)	Raise power with Control Rods (Similar to Audit, different power)
3.	NM-21	I(RO)	APRM Malfunction (Similar to Audit, failure is different)
4.	CW-08B	C(PO) C(SRO)	Loss of a RACS pump B
5.	EG-04	M(ALL)	Loss of SWC/ATWS/SLC Pump Failure/Loss of Off-site Power
6.	CU-11A/B	I(RO) I(SRO)	Failure of RWCU Isolation Logic
7.	HP-06E	I(PO) I(SRO)	HPCI auto initiation failure

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

FORM ES-301-5

# Operating Test Number:

Annticent Tune	Contaction Toron	Minimum Number	Scenario Number				
Applicant Type	Evolution Type		1	2	3	4	
RO(1)	Reactivity	1	2				
	Normal	1		1			
	Instrument / Component	4	3,5	2,5,7			
	Major	1	6	6			

	Reactivity	1		3	
	Normal	0			
As RO	Instrument / Component	2		4,5	
	Major	1		6	
SRO-I(6)				· · · · · · · · · · · · · · · · · · ·	
	Reactivity	0			
	Normal	1	1,2		
As SRO	Instrument / Component	2	3,4,5		
	Major	1	6		

SRO-U(7)	Reactivity	0			
	Normal	1	1,2		
	Instrument / Component	2	3,4,5		
	Major	1	6		

Instructions:

- (1) Enter the operating test number and Form ES-D-1 event numbers for each evolution type.
- (2) Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.4.d) but must be significant per Section C.2.a of Appendix D.
- (3) Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements.

Author

FORM ES-301-5

## Operating Test Number:

Applicant Type	Funktion Tune	Minimum Number	Minimum Scenario Number				
	Evolution Type		1	2	3	4	
	Reactivity	1		3			
	Normal	1	1				
RO(2)	Instrument / Component	4	4,7	4,5	, ,		
	Major	1	6	6			

	Reactivity	1		
	Normal	0		
As RO	Instrument / Component	2		
	Major	1		
SRO-I				•
	Reactivity	0		
	Normal	1		
As SRO	Instrument / Component	2		
	Major	1		

	Reactivity	0			
SRO-U(8)	Normal	1	1,2		
	Instrument / Component	2	3,4,5		
	Major	1	6		

Instructions:

- (4) Enter the operating test number and Form ES-D-1 event numbers for each evolution type.
- (5) Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.4.d) but must be significant per Section C.2.a of Appendix D.
- (6) Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements.

Author:

FORM ES-301-5

Operating Test Number:

A	Evolution Tune	Minimum	Scenario Number				
Applicant Type	Evolution Type Number	1	2	3	4		
	Reactivity	1	2				
	Normal	1		1			
RO(3)	Instrument / Component	4	3,5	2,5,7			
	Major	1	6	6			

	Reactivity	1			
	Normal	0			
As RO	Instrument / Component	2			
	Major	1			
SRO-I				•	
	Reactivity	0			
	Normal	1			
As SRO	Instrument / Component	2			
	Major	1			

SRO-U	Reactivity	0		
	Normal	1		
	Instrument / Component	2		
	Major	1		

Instructions:

- (7) Enter the operating test number and Form ES-D-1 event numbers for each evolution type.
- (8) Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.4.d) but must be significant per Section C.2.a of Appendix D.
- (9) Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements.

Author

FORM ES-301-5

Operatin	a Test	Number:
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Applicant Type	Custotian Tuna	Minimum Number	Minimum Scenario Number				
	Evolution Type		1	2	3	4	
	Reactivity	1		3			
	Normal	1	1				
RO(4)	Instrument / Component	4	4,7	4,5			
	Major	1	6	6			

	Reactivity	1		
	Normal	0		
As RO	Instrument / Component	2		
	Major	1		
SRO-I				
	Reactivity	0		
	Normal	1		
As SRO	Instrument / Component	2		
	Major	1		

	Reactivity	0		
SRO-U	Normal	1	·	
	Instrument / Component	2		
	Major	1		

Instructions:

- (10) Enter the operating test number and Form ES-D-1 event numbers for each evolution type.
- (11) Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.4.d) but must be significant per Section C.2.a of Appendix D.
- (12) Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements.

Δι	ithor:
$\sim$ 1	1183638

FORM ES-301-5

### Operating Test Number:

Applicant Tune	Evalution Type	Minimum		Scenario	Number	
Applicant Type	Evolution Type	Number	1	2	3	4
	Reactivity	1	2			
	Normal	1		1		
RO(5)	Instrument / Component	4	3,5	2,5,7		
	Major	1	6	6		

	Reactivity	1		
	Normal	0		
As RO	Instrument / Component	2		
	Major	1		
SRO-I				
	Reactivity	0		
	Normal	1		
As SRO	Instrument / Component	2		
	Major	1		

	Reactivity	0		
	Normal	1		
SRO-U	Instrument / Component	2		
	Major	1		

Instructions:

- (13) Enter the operating test number and Form ES-D-1 event numbers for each evolution type.
- (14) Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.4.d) but must be significant per Section C.2.a of Appendix D.
- (15) Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements.

Author:

### Competencies Checklist Page 1 of 1

FORM ES-301-6

	RO	Applic SRO	ant #1	)-U	RO	Applic SRO	ant #2 <del>I/SRC</del>	)-U	RC		ant #3 -I/SRC		
Competencies		SCEN	IARIO			SCEN	IARIO			SCEN	IARIO	RIO	
	1	2	3	4	1	2	3	4	1	2	3	4	
Understand and Interpret Annunciators and Alarms	3,5, 6	2,6			2,5, 6,7	2,4, 5,6			3,5, 6	2,6			
Diagnose Events and Conditions	3,5, 6	2,6, 7			2,6, 7	4,5, 6			3,5, 6	2,6, 7			
Understand Plant and System Response	2,3, 5,6	1,2, 6,7			1,2, 5,6, 7	2,3, 4,5, 6			2,3, 5,6	1,2, 6,7			
Comply With and Use Procedures (1)	2,3, 5,6	1,2, 5,6, 7			1,2, 5,6, 7	4,5, 6			2,3, 5,6	1,2, 5,6, 7			
Operate Control Boards (2)	2,3, 5,6	1,2, 5,6, 7			1,2, 5,6, 7	3,4, 5,6			2,3, 5,6	1,2, 5,6, 7			
Communicate and Interact With the Crew	2,3, 5,6	1,2, 5,6, 7			1,2, 5,6, 7	3,4, 5,6			2,3, 5,6	1,2, 5,6, 7			
Demonstrate Supervisory Ability (3)													
Comply With and Use Tech. Specs. (3)													

#### Notes:

- (1) Includes Technical Specification compliance for an RO.
- (2) Optional for an SRO-U.
- (3) Only applicable to SROs.

#### Instructions:

Circle the applicant's license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

Author:	Challes Ith	
NRC Reviewer:	/ ford	

### Competencies Checklist Page 1 of 1

FORM ES-301-6

	RO	Applic SRO	ant #4 -I/SRC	)-Ų	RO	Applic SRO	ant #5 -I/SRC	)-U	RC	Ap <del>plic</del> SRO	ant #6 -I/ <del>SR</del> C	)-U
Competencies		SCEN	IARIO			SCEN	IARIO			SCEN	IARIO	
	1	2	3	4	1	2	3	4	1	2	3	4
Understand and Interpret Annunciators and Alarms	2,5, 6,7	2,4, 5,6			3,5, 6	2,6			2,4 5,6	2,4, 5,6		
Diagnose Events and Conditions	2,6, 7	4,5, 6			3,5, 6	2,6, 7			2,4, 5,6	4,5, 6		
Understand Plant and System Response	1,2, 5,6, 7	2,3, 4,5, 6			2,3, 5,6	1,2, 6,7			1,2, 3,4, 5,6	2,3, 4,5, 6		
Comply With and Use Procedures (1)	1,2, 5,6, 7	4,5, 6			2,3, 5,6	1,2, 5,6, 7			2,3, 4,5, 6	4,5, 6		
Operate Control Boards (2)	1,2, 5,6, 7	3,4, 5,6			2,3, 5,6	1,2, 5,6, 7				3,4, 5,6		
Communicate and Interact With the Crew	1,2, 5,6, 7	3,4, 5,6			2,3, 5,6	1,2, 5,6, 7			1,2, 3,4, 5,6, 7	3,4, 5,6		
Demonstrate Supervisory Ability (3)									2,3, 4,5, 6,7			
Comply With and Use Tech. Specs. (3)									4			

#### Notes:

- (1) Includes Technical Specification compliance for an RO.
- (2) Optional for an SRO-U.
- (3) Only applicable to SROs.

#### Instructions:

Circle the applicant's license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

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### Competencies Checklist Page 1 of 1

FORM ES-301-6

	RC	Applic SRO	ant #7	)-U	RC	Applic D/SRO	ant #8	)-U		
Competencies		SCEN	IARIO			SCEN	IARIO			
	1	2	3	4	1	2	3	4		
Understand and Interpret Annunciators and Alarms	2,4 5,6				2,4 5,6					
Diagnose Events and Conditions	2,4, 5,6				2,4, 5,6					
Understand Plant and System Response	1,2, 3,4, 5,6				1,2, 3,4, 5,6					
Comply With and Use Procedures (1)	2,3, 4,5, 6				2,3, 4,5, 6					
Operate Control Boards (2)										
Communicate and Interact With the Crew	1,2, 3,4, 5,6, 7				1,2, 3,4, 5,6, 7					
Demonstrate Supervisory Ability (3)	2,3, 4,5, 6,7				2,3, 4,5, 6,7					
Comply With and Use Tech. Specs. (3)	4				4					

#### Notes:

- (1) Includes Technical Specification compliance for an RO.
- (2) Optional for an SRO-U.
- (3) Only applicable to SROs.

#### Instructions:

Circle the applicant's license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

ES-401		_	_				RO Examination Outline	ES-4	+01-
S 4 a	dki						Abnormal Evolutions - Tier 1/Group 1		Di
System#		KT	KZI	K3 /	41   A	2 0	G KA Topic(s)	Imp.	PIS
295005	Main Turbine Generator Trip					:		i	
295006	SCRAM			į		>	2.1.28 Knowledge of the purpose and function of major system components and controls.	3.2	1
295006	SCRAM	X					AK1.01 Decay heat generation and removal.	3.7	1
295007	High Reactor Pressure		Х				AK2.05 Shutdown cooling: Plant-Specific	2.9	. 1
295007	High Reactor Pressure			х			AK3.04 Safety/relief valve operation: Plant-Specific	4.0	1
295009	Low Reactor Water Level			l I	>	(	AA2.01 Reactor water level	4.2	1
295010	High Drywell Pressure			.	X		AA1.02 Drywell floor and equipment drain sumps	3.6	1
295014	Inadvertent Reactivity Addition		Х	.			AK2.04 Void concentration	3.2	1
295014	Inadvertent Reactivity Addition			X	-	-	AK3.01 Reactor SCRAM	4.1	1
295015	Incomplete SCRAM								
295024	High Drywell Pressure				X		EA1.10 A.C. distribution	3.4	1
295024	High Drywell Pressure					×	2.1.6 Ability to supervise and assume a management role during plant transients and upset conditions.	2.1	1
295025	High Reactor Pressure		+		>	(	EA2.06 Reactor water level	3.7	1
295025	High Reactor Pressure	Х			:	+	EK1.05 Exceeding safety limits	4.4	1
295031	Reactor Low Water Level		X				EK2.13 ARI/RPT/ATWS: Plant-Specific	4.1	1
295037	SCRAM Condition Present and Reactor Power Above APRM Downscale or Unknown								
500000	High Containment Hydrogen Concentration			X			EK3.03 Operation of hydrogen and oxygen recombiners	3.0	1

ES-401		_					O Examination Outline	ES-4	401-
0	[A.1						onormal Evolutions - Tier 1/Group 2	1.	1
System #	Name	K1	K2 K	З А	.1 A2	2 G	KA Topic(s)	Imp.	Pt
295001	Partial or Complete Loss of Forced Core Flow Circulation			<b>)</b>	<		AA1.02 RPS	3.3	1
295002	Loss of Main Condenser Vacuum	Х					AK1.04 Increased offgas flow	3.0	1
295003	Partial or Complete Loss of A.C. Power			×	<		AA1.03 Systems necessary to assure safe plant shutdown	4.4	1
	Partial or Complete Loss of A.C. Power						2.4.9 Knowledge of low power / shutdown implications in accident (e.g. LOCA or loss of RHR) mitigation strategies.	3.3	1
295004	Partial or Complete Loss of D.C. Power		)	X			AK3.03 Reactor SCRAM: Plant-Specific	3.1	1
295008	High Reactor Water Level		,	X			AK3.04 Reactor feed pump trip: Plant-Specific	3.3	1
295008	High Reactor Water Level			×	(		AA1.01 Reactor water level control: Plant-Specific	3.7	1
295011	High Containment Temperature (Mark III Containment Only)								-
295012	High Drywell Temperature	Х				,	AK1.01 Pressure/temperature relationship	3.3	1
295013	High Suppression Pool Temperature		Х				AK2.01 Suppression pool cooling	3.6	1
295016	Control Room Abandonment			×			AA1.02 Reactor/turbine pressure regulating system	2.9	1
295017	High Off-Site Release Rate								+
	Partial or Complete Loss of Component Cooling Water								
295019	Partial or Complete Loss of Instrument Air				X		AA2.01 Instrument air system pressure	3.5	1
295019	Partial or Complete Loss of Instrument Air			X		,	AA1.02 Instrument air system valves: Plant-Specific	3.3	1
295020	Inadvertent Containment Isolation								
295022	Loss of CRD Pumps		X	-		,	AK2.03 Accumulator pressures.	3.4	1
295026	Suppression Pool High Water Temperature	x		-	-		EK1.02 Steam condensation	3.5	1
	High Containment Temperature (Mark III Containment Only)								<u>.</u>

ES-401		 -					RO Examination Outline	ES-4	01-2
System #	Name						bnormal Evolutions - Tier 1/Group 2  KA Topic(s)	Imp.	Pts.
295028	High Drywell Temperature	X					EK1.02 Equipment environmental qualification	2.9	1
295029	High Suppression Pool Water Level								
295030	Low Suppression Pool Water Level		Х				EK2.03 LPCS	3.8	1
295033	High Secondary Containment Area Radiation Levels								
295034	Secondary Containment Ventilation High Radiation								
295038	High Off-Site Release Rate	X					EK1.02 Protection of the general public	4.2	1
295038	High Off-Site Release Rate				X		EA2.03 Radiation levels	3.5	1
600000	Plant Fire On Site				-	Х	2.4.25 Knowledge of fire protection procedures.	2.9	1

ES-401					ΒV	/R I	RO Examination Outline	ES-	401-2
			Emer	geno	y ai	nd A	Abnormal Evolutions - Tier 1/Group 3		.,
System #	Name	K1	K2 k	(3 A	1 A	2	G KA Topic(s)	Imp.	Pts.
295021	Loss of Shutdown Cooling			İ					
295023	Refueling Accidents			ļ	-	>	X 2.4.11 Knowledge of abnormal condition procedures.	3.4	1
295023	Refueling Accidents			>	(		AA1.02 Fuel pool cooling and cleanup system	2.9	1
295032	High Secondary Containment Area Temperature								
295035	Secondary Containment High Differential Pressure								
295036	Secondary Containment High Sump/Area Water Level		7	Χİ			EK3.01 Emergency depressurization	2.6	1
295036	Secondary Containment High Sump/Area Water Level		Х				EK2.01 Secondary containment equipment and floor drain system	3.1	1

ES-401										nation Outline Tier 2/Group 1	ES-40	01-2
System	Name	K1	K2	K3 F	(4 K					G KA Topic(s)	Imp.	Pts
	Control Rod Drive Hydraulic System			_				Х		A3.05 Reactor water level	2.8	1
201002	Reactor Manual Control System							: : : :		X 2.4.21 Knowledge of the parameters and logic used to assess the status of safety functions including:  1.Reactivity control 2.Core cooling and heat removal 3.Reactor coolant system integrity 4.Containment conditions 5.Radioactivity release control.	3.7	1
201002	Reactor Manual Control System	]			X					K4.08 Continuous In rod insertion	3.2	1
201005	Rod Control and Information System (RCIS)							İ				
202002	Recirculation Flow Control System					Х				K6.04 Feedwater flow inputs: BWR-3, 4, 5,6	3.5	1
203000	RHR/LPCI: Injection Mode (Plant Specific)	X								K1.14 Shutdown cooling system: Plant-Specific	3.6	1
203000	RHR/LPCI: Injection Mode (Plant Specific)							-	X	A4.07 Reactor water level	4.5	1
206000	High Pressure Coolant Injection System							X	1	A3.07 Lights and alarms: BWR-2, 3, 4	3.9	1
206000	High Pressure Coolant Injection System				-	-  -	x			A1.06 System flow: BWR-2, 3, 4	3.8	1
207000	Isolation (Emergency) Condenser							· · · · <del>} · · · ·</del>		_ <del>:</del> :		-
209001	Low Pressure Core Spray System	X								K1.10 Emergency generator	3.7	1
209001	Low Pressure Core Spray System		Х							K2.02 Valve power	2.5	1
	High Pressure Core Spray System (HPCS)		i									
211000	Standby Liquid Control System					1				X 2.4.10 Knowledge of annunciator response procedures.	3.0	1
211000	Standby Liquid Control System	X				-				K1.05 RWCU	3.4	1
212000	Reactor Protection System									2.1.23 Ability to perform specific system and integrated plant procedures during different modes of plant operation.	3.9	1
212000	Reactor Protection System				X				1	K5.02 Specific logic arrangements	3.3	1

ES-401										nination Outline - Tier 2/Group 1	ES-40	01-2
System	Name	K1	K2 K	(3 k	(4 K			. <b></b>		4 G KA Topic(s)	Imp.	Pts.
215003	Intermediate Range Monitor (IRM) System	1	<del> </del>									
215004	Source Range Monitor (SRM) System								х	A3.03 RPS status	3.6	1
215005	Average Power Range Monitor/Local Power Range Monitor System		7	X						K3.05 Reactor power indication  Accorb 73	3.8	1
215005	Average Power Range Monitor/Local Power Range Monitor System			-	X					K5.05 Core flow effects on APRM trip setpoints	3.6	1
216000	Nuclear Boiler Instrumentation		Х			-				K2.01 Analog trip system: Plant-Specific	2.8	1
216000	Nuclear Boiler Instrumentation							Х		A2.08 Elevated containment temperature	3.2	1
217000	Reactor Core Isolation Cooling System (RCIC)				X					K4.05 Prevents radioactivity release to auxiliary/reactor building	3.2	1
218000	Automatic Depressurization System		.		İ	İ						
223001	Primary Containment System and Auxiliaries					X		-		K6.13 Applicable plant air system/ nitrogen make-up system.	3.2	1
223002	Primary Containment Isolation System/Nuclear Steam Supply Shut-Off		>	X						K3.16 Shutdown cooling system/RHR	3.2	1
223002	Primary Containment Isolation System/Nuclear Steam Supply Shut-Off			7	X	4				K4.01 Redundancy	3.0	1
239002	Relief/Safety Valves			İ		İ	X			A1.05 Reactor water level	3.7	1
239002	Relief/Safety Valves						X			A1.02 Acoustical monitor noise: Plant-Specific	3.7	1
241000	Reactor/Turbine Pressure Regulating System											
259001	Reactor Feedwater System											
259002	Reactor Water Level Control System											
261000	Standby Gas Treatment System					X			Ī	K6.03 Emergency diesel generator system	3.0	1
261000	Standby Gas Treatment System									A4.07 System flow	3.1	1
264000	Emergency Generators (Diesel/Jet)			+			-					
l	l	<u> </u>	<u> </u> _				1					l

ES-401											ation O er 2/Gro		ES-40	۰01
System	Name	K1	K2	K3	K4 K						Э КА То		lmp.	. F
201003	Control Rod and Drive Mechanism				X						K4.05	Rod position indication	3.2	
	Rod Sequence Control System (Plant Specific)	-												
	Rod Worth Minimizer System (RWM) (Plant Specific)			X							K3.01	Reactor manual control system: P-Spec(Not-BWR6)	3.2	
202001	Recirculation System			Х							K3.07	Vessel bottom head drain temperature	2.9	
202001	Recirculation System							. !		х	A4.04	System flow	3.7	
204000	Reactor Water Cleanup System							X			A2.14	System high temperature	3.2	
	Shutdown Cooling System (RHR Shutdown Cooling Mode)				:									
	Rod Position Information System					:		.  -			}			
215002	Rod Block Monitor System													
	RHR/LPCI: Torus/Suppression Pool Cooling Mode			-				7	X		A3.01	Valve operation	3.3	
	RHR/LPCI: Containment Spray System Mode				<del></del> -			7	X		A3.05	Containment pressure	4.0	
	RHR/LPCI: Containment Spray System Mode			i	:	:	X	İ			A1.06	System flow	3.2	
	RHR/LPCI: Torus/Suppression Pool Spray Mode				:	X		-	İ		K6.01	A.C. electrical	3.3	
239001	Main and Reheat Steam System				+	:								
	Main Turbine Generator and Auxiliary Systems				Х	(					K5.02	Turbine operation and limitations	2.8	
	Reactor Condensate System				X						K4.06	Control of extraction steam	2.8	
256000	Reactor Condensate System				-	Ī		X			A2.13	Loss of applicable plant air systems	2.9	:
262001	A.C. Electrical Distribution				İ	X					K6.01	D.C. power	3.1	<del>-</del>
262001	A.C. Electrical Distribution		х	1							K2.01	Off-site sources of power	3.3	<del></del>
262002	Uninterruptable Power Supply (A.C./D.C.)			+		+	+ +		-					

ES-401			<u></u>			nation Outline Fier 2/Group 2	ES-40	01-2
System	Name	K1 K2	K3 K4 K			G KA Topic(s)	Imp.	Pts.
263000	D.C. Electrical Distribution	Х				K2.01 Major D.C. loads	3.1	1
263000	D.C. Electrical Distribution				X	A1.01 Battery charging/discharging rate	2.5	1
271000	Offgas System	x				K1.02 Process radiation monitoring system	3.1	1
272000	Radiation Monitoring System			x		K6.03 A.C. power	2.8	1
286000	Fire Protection System							
290001	Secondary Containment							
290003	Control Room HVAC	.	7	7		K5.01 Airborne contamination (e.g., radiological, toxic gas, smoke) control	3.2	1
300000	Instrument Air System (IAS)			. !		Neces 100		-
400000	Component Cooling Water System (CCWS)							

ES-401				BWR RO Ex		ES-4	01-2
System	Name	K1 K2 K3 F		(6 A1 A2 A3		lmp.	Pts.
215001	Traversing In-Core Probe	X			K1.05 Primary containment isolation system: (Not-BWR1)	3.3	1
233000	Fuel Pool Cooling and Clean-up						
234000	Fuel Handling Equipment				_		
239003	MSIV Leakage Control System						
268000	Radwaste			x	A1.01 Radiation level	2.7	1
288000	Plant Ventilation Systems					-         	
290002	Reactor Vessel Internals			X	A2.02 Overpressurization transient	3.6	1
290002	Reactor Vessel Internals	x	.   .   .		K3.03 Reactor power	3.3	1

Facility: Hope Creek		Date of Exam 03/12/2002 Exa	m Leve	el: RO
Category	KA#	Topic	lmp.	oints
Conduct of Operations	2.1.33	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.	3.4	1
	2.1.3	Knowledge of shift turnover practices.	3.0	1
	2.1.24	Ability to obtain and interpret station electrical and mechanical drawings	. 2.8	1
	Total			3
Equipment Control	2.2.27	Knowledge of the refueling process.	2.6	1
	2.2.22	Knowledge of limiting conditions for operations and safety limits.	3.4	1
	2.2.30	Knowledge of new and spent fuel movement procedures.	2.6	1
	Total		<u> </u>	3
Radiological Controls	2.3.1	Knowledge of 10 CFR 20 and related facility radiation control requirements.	2.6	1
	2.3.11	Ability to control radiation releases.	2.7	1
	2.3.9	Knowledge of the process for performing a containment purge.	2.5	1
	2.3.2	Knowledge of facility ALARA program.	2.5	1
	Total			4
Emergency Procedures and Plan	2.4.5	Knowledge of the organization of the operating procedures network for normal, abnormal, and emergency evolutions.	2.9	1
	2.4.34	Knowledge of RO tasks performed outside the main control room during lemergency operations including system geography and system	3.8	1
	2.4.18	Knowledge of the specific bases for EOPs.	2.7	1
	Total			3
	(RO/SRO)		<u></u>	13

ES-401	·							ramination Outline	ES-4	01-1
0 4 4					<del>-</del> -			l Evolutions - Tier 1/Group 1		Lou
System #		K1 K	(2)	K3 A	1 /	(2 G	КА Тор	•	lmp.	Pts
295003	Partial or Complete Loss of A.C. Power			>	(		AA1.03	Systems necessary to assure safe plant shutdown	4.4	1
295006	SCRAM		+			X	2.1.28	Knowledge of the purpose and function of major system components and controls.	3.3	1
295006	SCRAM	X					AK1.01	Decay heat generation and removal.	3.9	1
295007	High Reactor Pressure		X				AK2.05	Shutdown cooling: Plant-Specific	3.1	1
295007	High Reactor Pressure			x			AK3.04	Safety/relief valve operation: Plant-Specific	4.1	1
295009	Low Reactor Water Level				,	x	AA2.01	Reactor water level	4.2	1
295009	Low Reactor Water Level					Х	2.4.6 K	nowledge symptom based EOP mitigation strategies.	4.0	1
295010	High Drywell Pressure	+	+	>	(		AA1.02	Drywell floor and equipment drain sumps	3.6	1
295013	High Suppression Pool Temperature	;	x				AK2.01	Suppression pool cooling	3.7	1
295014	Inadvertent Reactivity Addition			Х			AK3.01	Reactor SCRAM	4.1	1
295014	Inadvertent Reactivity Addition		X				AK2.04	Void concentration	3.3	1
295015	Incomplete SCRAM			X			AK3.01	Bypassing rod insertion blocks	3.7	1
295015	Incomplete SCRAM					X		nowledge of radiation exposure limits and contamination control, including sible levels in excess of those authorized.	3.1	1
295016	Control Room Abandonment			×			AA1.02	Reactor/turbine pressure regulating system	3.1	1
295017	High Off-Site Release Rate			İ	>	<b>(</b>	AA2.01	Off-site release rate: Plant-Specific	4.2	1
295023	Refueling Accidents			×			AA1.02	Fuel pool cooling and cleanup system	3.1	1
295023	Refueling Accidents					х	2.4.11	Knowledge of abnormal condition procedures.	3.6	1
295024	High Drywell Pressure			×			EA1.10	A.C. distribution	3.6	1
295024	High Drywell Pressure					X		bility to supervise and assume a management role during plant transients and onditions.	4.3	1

ES-401	<del>.</del>						RO Examination Outline	ES-4	401-1
	,						normal Evolutions - Tier 1/Group 1		
System #	Name	K1	K2	K3	A1 A2	2 G	(A Topic(s)	lmp.	. Pts
295025	High Reactor Pressure	X	1			E	EK1.05 Exceeding safety limits	4.7	1
295026	Suppression Pool High Water Temperature	X	Ť	+		E	EK1.02 Steam condensation	3.8	1
295027	High Containment Temperature (Mark III Containment Only)								
295030	Low Suppression Pool Water Level		X			E	EK2.03 LPCS	3.9	1
295030	Low Suppression Pool Water Level				X	E	EA2.04 Drywell/ suppression chamber differential pressure: Mark-I&II	3.7	1
295031	Reactor Low Water Level		Χ			E	EK2.13 ARI/RPT/ATWS: Plant-Specific	4.2	1
295037	SCRAM Condition Present and Reactor Power Above APRM Downscale or Unknown								: :
295038	High Off-Site Release Rate	X				E	EK1.02 Protection of the general public	4.4	1
500000	High Containment Hydrogen Concentration	-		Х		E	EK3.03 Operation of hydrogen and oxygen recombiners	3.5	1

ES-401			_					amination Outline	ES-4	ł01-1
System#	Nome						Abnormal G KA Topi	Evolutions - Tier 1/Group 2	Imp.	Pts
		1 1	NZ.	NJ		AZ (			1	FIS
295001	Partial or Complete Loss of Forced Core Flow Circulation				Х		AA1.02	RPS	3.3	1
295002	Loss of Main Condenser Vacuum	X					AK1.04	Increased offgas flow	3.3	1
295004	Partial or Complete Loss of D.C. Power			Х			AK3.03	Reactor SCRAM: Plant-Specific	3.5	1
295004	Partial or Complete Loss of D.C. Power					х	AA2.01	Cause of partial or complete loss of D.C. power	3.6	1
295005	Main Turbine Generator Trip					Х	AA2.04	Reactor pressure	3.8	1
295008	High Reactor Water Level				Х		AA1.01	Reactor water level control: Plant-Specific	3.7	1
295008	High Reactor Water Level			х			AK3.04	Reactor feed pump trip: Plant-Specific	3.5	1
295011	High Containment Temperature (Mark III Containment Only)									
295012	High Drywell Temperature	X				İ	AK1.01	Pressure/temperature relationship	3.5	1
295018	Partial or Complete Loss of Component Cooling Water					Х	AA2.03	Cause for partial or complete loss	3.5	1
295019	Partial or Complete Loss of Instrument Air				Χ		AA1.02	Instrument air system valves: Plant-Specific	3.1	1
295020	Inadvertent Containment Isolation				,		:			
295021	Loss of Shutdown Cooling					>	X 2.4.41	Knowledge of the emergency action level thresholds and classifications.	4.1	1
295022	Loss of CRD Pumps					)		Ability to interpret control room indications to verify the status and operation of and understand how operator action s and directives affect plant and system	3.8	1
295022	Loss of CRD Pumps		Χ				:	Accumulator pressures.	3.4	1
295028	High Drywell Temperature	x					EK1.02	Equipment environmental qualification	3.1	1
295029	High Suppression Pool Water Level					+	* · · · · · · · · · · · · · · · · · · ·			1
295032	High Secondary Containment Area Temperature					•	•			
295033	High Secondary Containment Area Radiation Levels					<del>-</del>	· <del>  -</del> · · · · · · · · · · · · · · · · · ·			

				BW	RO Examination Outline	E:	S-40	J1-1
		Eme	rgeno	cy ar	normal Evolutions - Tier 1/Group 2			
Name	K1	K2	<3 A	1 A2	A Topic(s)	In	np.	Pts.
Secondary Containment Ventilation High Radiation					· · · · · · · · · · · · · · · · · · ·	reported 3	.6	1
Secondary Containment High Differential Pressure						-		 I
Secondary Containment High Sump/Area Water Level			X		K3.01 Emergency depressurization	2	8.	1
Secondary Containment High Sump/Area Water Level		X			K2.01 Secondary containment equipment and floor drain system	3	.2	1
Plant Fire On Site								
	Secondary Containment High Differential Pressure Secondary Containment High Sump/Area Water Level Secondary Containment High Sump/Area Water Level	Name K1 Secondary Containment Ventilation High Radiation Secondary Containment High Differential Pressure Secondary Containment High Sump/Area Water Level Secondary Containment High Sump/Area Water Level	Name Secondary Containment Ventilation High Radiation Secondary Containment High Differential Pressure Secondary Containment High Sump/Area Water Level Secondary Containment High Sump/Area Water Level X	Name Secondary Containment Ventilation High Radiation Secondary Containment High Differential Pressure Secondary Containment High Sump/Area Water Level Secondary Containment High Sump/Area Water Level X Water Level	Name  Secondary Containment Ventilation High Radiation  Secondary Containment High Differential Pressure  Secondary Containment High Sump/Area Water Level  Emergency and Abi X	Secondary Containment Ventilation High Radiation  X 2.4.30 Knowledge of which events related to system operations/status should be to outside agencies.  Secondary Containment High Differential Pressure  Secondary Containment High Sump/Area Water Level  X EK3.01 Emergency depressurization  EK2.01 Secondary containment equipment and floor drain system  Water Level	Name    K1   K2   K3   A1   A2   G   KA Topic(s)   In Secondary Containment Ventilation High Radiation   Secondary Containment High Differential Pressure   Secondary Containment High Sump/Area Water Level   X   EK3.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment equipment and floor drain system   3   Secondary Containment High Sump/Area   X   Secondary Containment equipment and floor drain system   3   Secondary Containment High Sump/Area   X   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipment and floor drain system   3   Secondary Containment equipme	Name    K1   K2   K3   A1   A2   G   KA Topic(s)   Imp.   Secondary Containment Ventilation High Radiation   Secondary Containment High Differential Pressure   Secondary Containment High Sump/Area Water Level   X   X   EK3.01   Emergency depressurization   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment High Sump/Area   X   EK2.01   Secondary Containment and floor drain system   3.2

ES-401	· <del></del> ···							ination Fier 2/Gr		ES-4	01-1
System	Name	K1 K2	K3 K4 K							Imp.	Pts.
201005	Rod Control and Information System (RCIS)										
202002	Recirculation Flow Control System										
203000	RHR/LPCI: Injection Mode (Plant Specific)	x					-	K1.1	4 Shutdown cooling system: Plant-Specific	3.7	1
203000	RHR/LPCI: Injection Mode (Plant Specific)						X	A4.0	Reactor water level	4.5	1
206000	High Pressure Coolant Injection System			İ	X			A1.0	5 System flow: BWR-2, 3, 4	3.7	1
207000	Isolation (Emergency) Condenser			-						:	<del> </del>
209001	Low Pressure Core Spray System	X		İ				K1.10	Emergency generator	3.8	1
209001	Low Pressure Core Spray System	X						K2.02	2 Valve power	2.7	1
209002	High Pressure Core Spray System (HPCS)										
211000	Standby Liquid Control System									:	+
212000	Reactor Protection System		X					K5.02	2 Specific logic arrangements	3.4	1
215004	Source Range Monitor (SRM) System			i		Х		A3.03	3 RPS status	3.5	1
215004	Source Range Monitor (SRM) System								Knowledge of the process for making changes in procedures as ibed in the safety analysis report.	3.3	1
215005	Average Power Range Monitor/Local Power Range Monitor System		X					K5.08	Core flow effects on APRM trip setpoints	3.6	1
	Average Power Range Monitor/Local Power Range Monitor System		X					K3.05	Reactor power indication	3.8	1
	Nuclear Boiler Instrumentation	х		-				K2.01	Analog trip system: Plant-Specific	2.8	1
216000	Nuclear Boiler Instrumentation					x		A2.08	B Elevated containment temperature	3.4	1
217000	Reactor Core Isolation Cooling System (RCIC)				;	X		A2.01	System initiation signal	3.7	1
	Reactor Core Isolation Cooling System (RCIC)		X					K4.05	Prevents radioactivity release to auxiliary/reactor building	3.5	1
	Automatic Depressurization System		+ + +	+				-		:	
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ES-401												nation Outline er 2/Group 1	ES-4	01-1
System	Name	K1	K2	КЗ	K4	K5 K	(6 A	\1 A	2 A3	3 A	4 0	KA Topic(s)	lmp.	Pts.
	Primary Containment System and Auxiliaries													
223002	Primary Containment Isolation System/Nuclear Steam Supply Shut-Off			Х								K3.16 Shutdown cooling system/RHR	3.3	1
	Primary Containment Isolation System/Nuclear Steam Supply Shut-Off				X							K4.01 Redundancy	3.2	1
	RHR/LPCI: Containment Spray System Mode								X			A3.05 Containment pressure	4.0	1
239002	Relief/Safety Valves						)	Κ		İ		A1.05 Reactor water level	3.8	1
	Reactor/Turbine Pressure Regulating System	-				:	ļ		!					
259002	Reactor Water Level Control System							i			>	2.4.32 Knowledge of operator response to loss of all annunciators.	3.5	1
261000	Standby Gas Treatment System					)	Κ		İ	- <del>-</del>		K6.03 Emergency diesel generator system	3.1	1
261000	Standby Gas Treatment System							:		X		A4.07 System flow	3.2	1
262001	A.C. Electrical Distribution					>	Κ	1				K6.01 D.C. power	3.4	1
264000	Emergency Generators (Diesel/Jet)									- <del></del> -	>	2.1.11 Knowledge of less than one hour technical specification action statements for systems.	3.8	1
290001	Secondary Containment		1		i	1			:	:	i	:		

ES-401									ation Outline	ES-40	01-1
System	Name	K1 K2	K3	K4 K5					KA Topic(s)	Imp.	Pts.
201001	Control Rod Drive Hydraulic System						Х		A3.05 Reactor water level	2.8	1
201002	Reactor Manual Control System			x					K4.08 Continuous In rod insertion	3.2	1
201004	Rod Sequence Control System (Plant Specific)				-	100					
201006	Rod Worth Minimizer System (RWM) (Plant Specific)		x						K3.01 Reactor manual control system: P-Spec(Not-BWR6)	3.5	1
201006	Rod Worth Minimizer System (RWM) (Plant Specific)							Х	2.1.12 Ability to apply technical specifications for a system.	4.0	1
202001	Recirculation System		X						K3.07 Vessel bottom head drain temperature	2.9	1
204000	Reactor Water Cleanup System					×			A2.14 System high temperature	3.2	1
205000	Shutdown Cooling System (RHR Shutdown Cooling Mode)							:			
214000	Rod Position Information System										
215002	Rod Block Monitor System										
215003	Intermediate Range Monitor (IRM) System										
	RHR/LPCI: Torus/Suppression Pool Cooling Mode										
	RHR/LPCI: Torus/Suppression Pool Spray Mode				Х				K6.01 A.C. electrical	3.4	1
234000	Fuel Handling Equipment								2.2.25 Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.	3.7	1
239003	MSIV Leakage Control System										
	Main Turbine Generator and Auxiliary Systems							i			
259001	Reactor Feedwater System								· · · · · · · · · · · · · · · · · · ·		
262002	Uninterruptable Power Supply (A.C./D.C.)										
263000	D.C. Electrical Distribution	X							K2.01 Major D.C. loads	3.4	1
263000	D.C. Electrical Distribution	+ + -	-			x		+	A1.01 Battery charging/discharging rate	2.8	1

ES-401		BWR SRO Examination Outline Plant Systems - Tier 2/Group 2				
System	Name	K1 K2 K3	3 A4 G KA Topic(s)	lmp.	Pts	
271000	Offgas System	X		K1.02 Process radiation monitoring system	3.3	1
272000	Radiation Monitoring System			K6.03 A.C. power	3.0	1
286000	Fire Protection System					
290003	Control Room HVAC		<b>x</b>	K5.01 Airborne contamination (e.g., radiological, toxic gas, smoke) control	3.5	1
300000	Instrument Air System (IAS)					
	Component Cooling Water System (CCWS)					

ES-401			BWR SRO Examination Outline Plant Systems - Tier 2/Group 3				
System	Name	K1 K2		A2 A3 A4 G KA Topic(s)	lmp.	Pts.	
201003	Control Rod and Drive Mechanism		X	K4.05 Rod position indication	3.3	1	
215001	Traversing In-Core Probe	x		K1.05 Primary containment isolation system: (Not-BWR1)	3.4	1	
233000	Fuel Pool Cooling and Clean-up			X 2.1.7 Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	4.4	1	
239001	Main and Reheat Steam System				-	1	
256000	Reactor Condensate System						
268000	Radwaste		X	A1.01 Radiation level	3.1	1	
288000	Plant Ventilation Systems					-	
290002	Reactor Vessel Internals	-				i	

Facility: Hope Creek		Date of Exam 03/12/2002 Exa		SRO
Category	KA #	Topic	Imp.	oints
Conduct of Operations	2.1.14	Knowledge of system status criteria which require the notification of plant personnel.	3.3	1
	2.1.3	Knowledge of shift turnover practices.	3.4	1
	2.1.33	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.	4.0	1
	2.1.24	Ability to obtain and interpret station electrical and mechanical drawings	. 3.1	1
	2.1.34	Ability to maintain primary and secondary plant chemistry within allowable limits.	2.9	1
	Total			5
Equipment Control	2.2.26	Knowledge of refueling administrative requirements.	3.7	1
,	2.2.31	Knowledge of SRO fuel handling responsibilities.	3.8	1
ster-	2.2.27	Knowledge of the refueling process.	3.5	1
	2.2.22	Knowledge of limiting conditions for operations and safety limits.	4.1	1
1	Total			4
Radiological Controls	2.3.1	Knowledge of 10 CFR 20 and related facility radiation control requirements.	3.0	1
	2.3.10	Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure.	3.3	1
	2.3.4	Knowledge of radiation exposure limits and contamination control, including permissible levels in excess of those authorized.	3.1	1
	2.3.11	Ability to control radiation releases.	3.2	1
	Total			4
Emergency Procedures and Plan	2.4.34	Knowledge of RO tasks performed outside the main control room during emergency operations including system geography and system	3.6	1
	2.4.18	Knowledge of the specific bases for EOPs.	3.6	1
	2.4.28	Knowledge of procedures relating to emergency response to sabotage.	3.3	1
	2.4.5	Knowledge of the organization of the operating procedures network for normal, abnormal, and emergency evolutions.	3.6	1
	Total	The state of the s		4
Tier 3 Target Point Tota	I (RO/SRO)			17