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Docket No. 50-298

DEC 2 0 1976

Nebraska Public Power District
ATTN: Mr. J. M. Pilant, Director
Licensing and Quality Assurance
Post Office Box 499
Columbus, Nebraska 68601

Gentlemen:

In response to your letter dated November 24, 1976, the Commission has issued the enclosed Amendment No. 33 to Facility Operating License No. DPR-46 for Cooper Nuclear Station. The amendment extends the interval for inspection of certain hydraulic shock suppressors to 6 months + 25% from November 10, 1976, or until the first plant shutdown during which access to the inaccessible snubbers is afforded, and makes several minor corrections to Table 3.6.1 of the CNS Technical Specifications.

Copies of the related Safety Evaluation Report and Notice of Issuance also are enclosed.

Sincerely,

Original Signed by:
Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
1. Amendment No. 33 to License No. DPR-46
2. Safety Evaluation Report
3. Notice

cc w/enclosures:
See next page

cont 1

SEE PREVIOUS YELLOW FOR CONCURRENCES

OFFICE	DOR:ORB #2	DOR:ORB #2	DOR:ORB #2	OELD	DOR:ORB #2	
SURNAME	RMDiggs	MHFletcher:ab	RPSnaider	DSwanson	DLZiemann	
DATE	12/ /76	12/20/76	12/ /76	12/ /76	12/20/76	

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U.S. GOVERNMENT PRINTING OFFICE: 1974-526-166

DFRoss
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cc w/enclosures:
See next page

as modified on p. 1376 of tech spec

OFFICE	DOR:ORB-2	DOR:ORB-2	DOR:ORB-2	OELD	DOR:ORB-2	
SURNAME	RMDiggs	MFletcher:esp	RPSnaider	DSwanson	DLZiemann	
DATE	12/8/76	12/9/76	12/9/76	12/10/76	12/10/76	

Form AEC-318 (Rev. 9-53) AECM 0240

☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

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U. S. Environmental Protection
Agency
Region VII
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1735 Baltimore Avenue
Kansas City, Missouri 64108

Chief, Energy Systems Analyses
Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection
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401 M Street, S. W.
Washington, D. C. 20460

Mr. William Siebert, Commissioner
Nemaha County Board of Commissioners
Nebraska County Courtroom
Auburn, Nebraska 68305

cc w/enclosures and cy of NPPD's
filing dtd. 11/24/76:
Director, Department of Environmental
Control
Executive Building, 2nd Floor
Lincoln, Nebraska 68509

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 33
License No. DPR-46

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Nebraska Public Power District (the licensee) dated November 24, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: DEC 20 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 33

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace existing pages 137b through 137i of the Appendix A portion of the Technical Specifications with the attached revised pages bearing the same numbers. The changed areas on the revised pages are indicated by a marginal line.

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT

4.6.H Shock Suppressors (Snubbers)

3. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule in Specification 4.6.H.1, it shall be assumed that the facility had been on a 6 month inspection interval.

4. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of piston movement, lock up and bleed. For each unit and subsequent unit found inoperable, an additional 10% or ten hydraulic snubbers shall be so tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb need not be functionally tested.

5. The inspection interval of Technical Specification 4.6.H.1 for inaccessible hydraulic shock suppressors is extended to 6 months ± 25% from November 10, 1976, or until the first plant shutdown during which access to the inaccessible snubbers is afforded.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
		892'9"		X		
AS-S-110(2)	Torus Area	890'9"		X		X
AS-S-111	Torus Area	891'9"		X		X
AS-S-112	Torus Area	891'9"		X		X
AS-S-113	Torus Area	893'		X		X
BS-S-1	Torus	870'8"		X		X
BS-S-15	Torus Area	893'4"		X		X
CS-S-1	S.E. Quad	918'				X
CS-S-2	S.E. Quad	929'				X
CS-S-3	S.E. Quad	946'3"				X
CS-S-4	Drywell	948'	X	X	X	
CS-S-5	Drywell	951'	X	X	X	
CS-S-8	Drywell	948'	X	X	X	
CS-S-9	Drywell	951'	X	X	X	
CS-S-10	Rx Bldg, 931'	946'3"				X
CS-S-11	Rx Bldg, 931'	946'3"				X
CU-S-3(2)	Drywell	925'	X	X	X	
HP-S-4	S.W. Quad	872'7"				X
HP-S-11	S.W. Quad	869'11"				X
HP-S-15	S.W. Quad	874'11"				X
MS-S-1	S.W. Quad	864'				X
MS-S-2	S.W. Quad	868'5"				X
MS-S-3	S.W. Quad	880'4"				X

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Amendment No. ~~29~~, 33

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
MS-S-4	S.W. Quad	873'5"				X
MS-S-7(2)	S.W. Quad	874'11"				X
MS-S-8	Torus Area	885'2"		X		X
MS-S-9(2)	Torus Area	894'11"		X		X
MS-S-10	Torus Area	899'11"		X		X
MS-S-11	Torus Area	897'		X		X
MS-S-12	Torus Area	888'		X		X
MS-S-13	S. RHR Hx Rm.	904'10"				X
MS-S-14	S. RHR Hx Rm.	923'				X
MS-S-15	S. RHR Hx Rm.	934'				X
MS-S-16	Torus Area	885'		X		X
MS-S-17	N. RHR Hx Rm.	904'10"				X
MS-S-18	N. RHR Hx Rm.	905'6"				X
MS-S-19	N. RHR Hx Rm.	923'				X
MS-S-20	N. RHR Hx Rm.	934'				X
MS-S-21	Drywell	920'8"	X	X	X	
MS-S-22	Drywell	919'1"	X	X	X	
MS-S-23	Torus Area	898'		X		X
MS-S-24	Torus Area	898'		X		X
MS-S-25	N.E. Quad	877'6"				X
MS-S-26	N.E. Quad	879'6"				X
MS-S-33D	Torus Area	894'		X		X
MS-S-63	Drywell	920'10"	X	X	X	
MS-S-75	N. RHR Hx Rm.	932'				X
MS-S-76	S. RHR Hx Rm.	932'				X
RCC-S-3	Rx Bldg, 931'	945'11"				X
RCC-S-4	Rx Bldg, 931'	943'6"				X
RCC-S-20	Rx Bldg, 931'	953'3"				X
RCC-S-21	Rx Bldg, 931'	953'3"				X
RCC-S-22	Rx Bldg, 931'	953'3"				X

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
RF-S-1	N.E. Quad	898'6"				X
RF-S-2	Torus Area	896'		X		X
RF-S-3	S.W. Quad	870'				X
RF-S-4	Torus Area	894'6"				X
RF-S-5	Torus Area	897'10"				X
RF-S-6	Torus Area	891'		X		X
RF-S-8	Drywell	923'8"	X	X	X	
RF-S-9	Drywell	923'8"	X	X	X	
RF-S-10	Drywell	925'6"	X	X	X	
RF-S-11	Drywell	923'8"	X	X	X	
RF-S-12	Drywell	923'8"	X	X	X	
RF-S-13	Drywell	925'6"	X	X	X	
RF-S-14	Drywell	925'	X	X	X	
RF-S-15	Drywell	922'9"	X	X	X	
RF-S-16	Drywell	923'8"	X	X	X	
RF-S-17	Drywell	924'4"	X	X	X	
RF-S-18	Drywell	923'8"	X	X	X	
RF-S-19	Drywell	923'8"	X	X	X	
RH-S-3	Reactor Cavity	982'9"	X	X	X	
RH-S-4	Reactor Cavity	982'9"	X	X	X	
RH-S-5	Drywell	922'	X	X	X	
RH-S-6	Drywell	921'6"	X	X	X	
RH-S-7	Drywell	921'8"	X	X	X	
RH-S-8(2)	Drywell	920'	X	X	X	
RH-S-9	Drywell	915'	X		X	
RH-S-10	Drywell	912'	X		X	
RH-S-11	Drywell	917'	X		X	
RH-S-13	Drywell	923'8"	X	X	X	
RH-S-14	Drywell	922'	X	X	X	
RH-S-15	Drywell	922'	X	X	X	

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
RH-S-16	Drywell	918'	X		X	
RH-S-17	Drywell	918'	X		X	
RH-S-18	Drywell	916'6"	X		X	
RH-S-19	Drywell	916'6"	X		X	
RH-S-20	Rx Bldg, 903	912'6"				X
RH-S-21	Rx Bldg, 903	911'				X
RH-S-22	Torus Area	895'9"		X		X
RH-S-23	Torus Area	892'		X		X
RH-S-24	Torus Area	897'		X		X
RH-S-25	N. RHR Hx Rm.	927'				X
RH-S-26	N. RHR Hx Rm.	929'				X
RH-S-29	Rx Bldg, 903'	904'6"				X
RH-S-30(2)	Torus Area	898'6"		X		X
RH-S-32	Torus Area	894'7"		X		X
RH-S-33D	Torus	892'3"		X		X
RH-S-34	Rx Bldg, 903'	919'6"				X
RH-S-35	S. RHR Hx Rm.	912'				X
RH-S-36	S. RHR Hx Rm.	914'3"				X
RH-S-37	S. RHR Hx Rm.	916'4"				X
RH-S-38	S. RHR Hx Rm.	930'				X
RH-S-39	S. RHR Hx Rm.	927'6"				X
RH-S-40	S. RHR Hx Rm.	915'6"				X
RH-S-41	S.W. Quad	873'				X
RH-S-42	S.W. Quad	874'				X
RH-S-43	Torus Area	897'		X		X
RH-S-44	S.W. Quad	884'6"				X
RH-S-45	S.W. Quad	884'				X
RH-S-48	N.W. Quad	884'6"				X
RH-S-49	N.W. Quad	885'				X
RH-S-51	N. RHR Hx Rm.	914'3"				X

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
RH-S-52	N. RHR Hx Rm.	915'				X
RH-S-54	N.W. Quad	873'1"				X
RH-S-55	N.W. Quad	874'				X
RH-S-56	N. RHR Hx Rm.	927'6"				X
RH-S-57	N. RHR Hx Rm.	927'6"				X
RH-S-58	N. RHR Hx Rm.	921'11"				X
RH-S-59	Torus Area	896'		X		X
RH-S-65	S.W. Quad	887'2"				X
RH-S-66	Rx Bldg, 903'	907'4"				X
RH-S-67	Drywell	917'8"	X	X	X	
RH-S-68	Drywell	917'2"	X		X	
RH-S-69(2)	Drywell	916'	X		X	
RH-S-70	Drywell	914'2"	X		X	
RH-S-71	Drywell	914'2"	X		X	
RH-S-72	Drywell	914'2"	X		X	
RH-S-72A	Drywell	914'2"	X		X	
RH-S-73	Drywell	918'	X		X	
RH-S-76(2)	Torus Area	898'		X		X
RH-S-77	Torus Area	890'11"		X		X
RH-S-78	Torus Area	897'		X		X
RH-S-80	N.W. Quad	889'				X
RH-S-98	N.W. Quad	891'				X
SS-A2	Drywell	923'8"	X	X	X	
SS-A3	Drywell	922'6"	X	X	X	
SS-B2	Drywell	923'8"	X	X	X	
SS-B3	Drywell	922'6"	X	X	X	
SS-C2	Drywell	923'8"	X	X	X	
SS-C3	Drywell	922'6"	X	X	X	
SS-D2	Drywell	923'8"	X	X	X	
SS-D3	Drywell	922'6"	X	X	X	
SS-7(4)	Drywell	913'8"	X		X	
SS-8(2)	Drywell	917'6"	X		x	

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-PPIB-1RR	Drywell	891'	X		X	
SS-PPIB-2RR	Drywell	898'	X		X	
SS-PPIB-3RR(2)	Drywell	904'	X		X	
SS-PPIB-4RR	Drywell	910'	X		X	
SS-PPIB-5RR	Drywell	898'	X		X	
SS-PPIA-1RR	Drywell	891'	X		X	
SS-PPIA-2RR	Drywell	898'	X		X	
SS-PPIA-3RR(2)	Drywell	904'	X		X	
SS-PPIA-4RR	Drywell	910'	X		X	
SS-PPIA-5RR	Drywell	898'	X		X	
SWH-WH-23A	Intake Str.	904'3"				X
SWH-WH-23B	Intake Str.	904'3"				X
SWH-WH-23C	Intake Str.	904'3"				X
SWH-WH-23D	Intake Str.	904'3"				X
VR-S-1	Drywell	901'8"	X	X	X	
VR-S-2	Drywell	919'7"	X	X	X	
VR-S-3	Drywell	900'	X	X	X	
VR-S-4	Drywell	918'4"	X		X	
VR-S-5(2)	Drywell	902'	X	X	X	
VR-S-6	Drywell	904'9"	X	X	X	
VR-S-7(2)	Drywell	899'	X	X	X	
VR-S-8	Drywell	899'7"	X	X	X	
H61D	Drywell	898'3"	X	X	X	
H62C	Drywell	898'10"	X	X	X	
H62B	Drywell	899'	X	X	X	
H63B	Drywell	897'7"	X	X	X	
H63C	Drywell	898'10"	X	X	X	
H64D	Drywell	897'	X	X	X	

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS) (Cont'd)

Snubber No.	Location	Elevation	Snubber in High* Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
55-9-Y	Drywell	920'6"	X	X	X	
55-9-Z	Drywell	918'	X	X	X	
55-23-X	Drywell	909'	X	X	X	
55-23-Y	Drywell	906'9"	X	X	X	
55-23-Z	Drywell	908'	X	X	X	
56-12-Y	Drywell	916'	X	X	X	
56-26-Y	Drywell	916'	X	X	X	
56-24-X	Drywell	910'	X	X	X	
56-24-Z	Drywell	909''	X	X	X	
57-12-Y	Drywell	924'6"	X	X	X	
58-12-Y	Drywell	924'	X	X	X	
59-7-X	Drywell	921'4"	X	X	X	
59-7-Z	Drywell	919'6"	X	X	X	
60-7-X	Drywell	921'4"	X	X	X	
60-7-Z	Drywell	920'9"	X	X	X	
61-8-X	Drywell	916'	X	X	X	
61-8-Y	Drywell	917'	X	X	X	
61-8-Z	Drywell	920'	X	X	X	
61-17-X	Drywell	914'3"	X	X	X	
61-17-Z	Drywell	914'3"	X	X	X	
62-8-X	Drywell	920'	X	X	X	
62-8-Y	Drywell	917'	X	X	X	
62-8-Z	Drywell	920'	X	X	X	
62-17-X	Drywell	913'2"	X	X	X	
62-17-Z	Drywell	913'2"	X	X	X	
10050	RHR Hx Rm.	932'				X

* Modification to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 33 TO LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

INTRODUCTION

By letter dated November 24, 1976, the Nebraska Public Power District (NPPD, the licensee) requested that Technical Specification 4.6.H.1 of the Cooper Nuclear Station (CNS) Technical Specifications be modified to extend the surveillance interval for certain inaccessible hydraulic shock suppressors (snubbers). NPPD also requested that several corrections be made to Table 3.6.1 of the Technical Specification, "Safety Related Shock Suppressors (Snubbers)."

BACKGROUND

Snubbers are designed to prevent unrestrained motion of piping systems under dynamic loads as might occur during an earthquake or severe transient while allowing normal thermal expansion and contraction during startup and shutdown. The consequence of an inoperable snubber is to increase the probability of structural damage to piping systems resulting from seismic or other severe dynamic loads.

Specification 4.6.H.1 of the CNS Technical Specifications describes a variable surveillance interval for snubbers which is a function of the number of inoperable snubbers found during the most recent inspection. The greater the number of snubbers found inoperable, the shorter the time interval until the next inspection. For example, the specification requires an inspection interval of 31 days $\pm$ 25% if eight or more snubbers are found inoperable during an inspection. Specification 4.6.H.1 also divides snubbers into two categories: "accessible" and "inaccessible". A snubber is considered inaccessible if it is not accessible for inspection during reactor operation.

During the September 1976 refueling outage at CNS, the licensee inspected the safety related shock suppressors and found ten inoperable because their hydraulic fluid reservoirs were devoid of fluid. In accordance with Specification 4.6.H.1, this occurrence would require a reinspection of all hydraulic snubbers within 31 days ± 25%. However, based on the information below, NPPD has requested that the inspection interval, for inaccessible snubbers only, be extended to 6 months ± 25% from the date of issuance of Amendment 32 to Operating License DPR-46 for CNS issued on November 10, 1976. NPPD also requested that several corrections be made to Table 3.6.1 of the CNS Technical Specifications. The corrections include (1) the addition of one snubber which was inadvertently omitted from the original table, (2) the deletion of three snubbers which were incorrectly included in the original table, and (3) the addition of an asterisk for the footnote on each page of the table.

EVALUATION

NPPD has initiated a program of replacing hydraulic shock suppressors with a mechanical type snubber which does not depend on a hydraulic fluid to function. During the recent CNS refueling outage, NPPD replaced 100 of the 114 inaccessible hydraulic snubbers with the mechanical type. The 14 remaining inaccessible hydraulic snubbers, manufactured by the Bergen Paterson Company, have been previously inspected with satisfactory results during five inspections over a 32 month period (March 1974, February 1975, October 1975, May 1976, and October 1976). Furthermore none of the 10 hydraulic snubbers that were found inoperable were manufactured by Bergen Paterson. Because of the small number of inaccessible hydraulic snubbers remaining at CNS and in view of the record of successful performance of these 14 snubbers, we conclude that a one-time extension of the surveillance interval to 6 months ± 25% from November 10, 1976 (the issuance date of Amendment 32 to the CNS operating license) for inaccessible snubbers at CNS is justified.

In addition, we conclude that the requested changes to Table 3.6.1 "Safety Related Shock Suppressors (Snubbers)" in the CNS Technical Specifications are acceptable because, with the changes incorporated, Table 3.6.1 more closely conforms to the actual plant arrangement for CNS.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves

an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 20, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-298

NEBRASKA PUBLIC POWER DISTRICT

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 32 to Facility Operating License No. DPR-46, issued to the Nebraska Public Power District (the licensee), which revised Technical Specifications for operation of the Cooper Nuclear Station (the facility) located in Nemaha County, Nebraska. The amendment is effective as of its date of issuance.

The amendment extended the surveillance interval for certain hydraulic shock suppressors to 6 months ± 25%, or until the first plant shutdown during which access to the inaccessible snubbers is afforded, and makes several minor corrections to Table 3.6.1 "Safety Related Shock Suppressors (Snubbers)" of the facility's Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

OFFICE➤						
SURNAME➤						
DATE➤						

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 24, 1976, (2) Amendment No. 39 to License No. DPR-46, and (3) the Commission's concurrently issued Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Auburn Public Library, 118 - 15th Street, Auburn, Nebraska 68305. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 20th day of December, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-298

Nebraska Public Power District
ATTN: Mr. J. M. Pilant, Director
Licensing and Quality Assurance
Post Office Box 499
Columbus, Nebraska 68601

Gentlemen:

In response to your letter dated November 24, 1976, the Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-46 for Cooper Nuclear Station. The amendment extends the interval for inspection of certain hydraulic shock suppressors to 6 months ± 25% from November 10, 1976 and makes several minor corrections to Table 3.6.1 of the CNS Technical Specifications.

Copies of the related Safety Evaluation Report and Notice of Issuance also are enclosed.

Sincerely,

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
1. Amendment No. to License No. DPR-46
2. Safety Evaluation Report
3. Notice

cc w/enclosures:
See next page

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT

4.6.H Shock Suppressors (Snubbers)

3. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule in Specification 4.6.H.1, it shall be assumed that the facility had been on a 6 month inspection interval.

4. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of piston movement, lock up and bleed. For each unit and subsequent unit found inoperable, an additional 10% or ten hydraulic snubbers shall be so tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb need not be functionally tested.

5. The inspection interval of Technical Specification 4.6.H.1 for inaccessible hydraulic shock suppressors is extended to 6 months ± 25% from November 10, 1976.