

DISTRIBUTION

Docket  
NRC PDR  
Local PDR  
ORB #2 Reading  
OELD  
OI&E (3)  
NDtbe  
BJones  
JMMcGough  
JSaltzman, OAI  
RMDiggs  
RDSilver  
DLZiemann  
SKari  
BScharf (15)  
TJCarter  
PCollins

SVarga  
CHbbron  
ACRS (16)  
AESteen  
JRBuchanan  
TBAbernathy  
DEisenhut

AUG 01 1975

Docket No. 50-298

Nebraska Public Power District  
ATTN: Mr. J. M. Pilant, Manager  
Licensing and Quality Assurance  
Post Office Box 499  
Columbus, Nebraska 68601

Gentlemen:

Enclosed is a signed original of the "Order for Modification of License" issued by the Commission on August 1, 1975, for your Cooper Nuclear Station. The Order modifies a provision to License No. DPR-46 by clarifying a revision added by the "Order for Modification of License" issued by the Commission on July 22, 1975.

The August 1 Order is in response to your letter of July 31, 1975 which requested that the operating period within which certain verification testing had to be completed be extended from 45 equivalent full flow days to 55 equivalent full flow days. Because of the desirability of an expeditious decision, we are not acting on the full request at this time. However, by the enclosed Order we have made clear that you may extend the subject operating period to 50 equivalent full flow days since that period is within the intended scope of the July 22, 1975 Order.

As noted in the enclosed Order, further evaluation would be required prior to the granting of any additional extension of time. We will make that evaluation if requested.

A copy of the Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Original Signed by:  
Dennis L. Ziemann

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Reactor Licensing

*DLZ*  
1

Enclosure:  
"Order for Modification  
of License" dtd. 8/1/75

OFFICE	RL:ORB #2 enclosure: see next page	OELD see next page	RL:AD/ORS	RL:DIR	NRR/AD/DIR	NRR:DIR
SURNAME	RDSilver:ak DLZiemann	AKaryalia	KRGoller	AGiambusso	EGCase	BCRusche
DATE	8/1/75	8/1/75	8/1/75	8/1/75	8/1/75	8/1/75

AUG 01 1975

cc w/enclosure:

Gene Watson, Attorney  
Barlow, Watson & Johnson  
P. O. Box 81686  
Lincoln, Nebraska 68501

Mr. Arthur C. Gehr, Attorney  
Snell & Wilmer  
400 Security Building  
Phoenix, Arizona 85004

Anthony Z. Roisman, Esquire  
Berlin, Roisman and Kessler  
1712 N Street, N. W.  
Washington, D. C. 20036

Auburn Public Library  
1118 - 15th Street  
Auburn, Nebraska 68305

Mr. William Siebert, Commissioner  
Nemaha County Board of Commissioners  
Nebraska County Courtroom  
Auburn, Nebraska 68305

cc w/enclosure and cy of NPPD's  
filing dtd. 7/31/75:

Mr. James L. Higgins, Director  
Department of Environmental Control  
Executive Building, 2nd Floor  
Lincoln, Nebraska 68509

Mr. Ed Vest  
Environmental Protection Agency  
1735 Baltimore Avenue  
Kansas City, Missouri 64108

Chief, TIRB  
Technology Assessment Division  
Office of Radiation Programs  
U. S. Environmental Protection  
Agency  
Room 647A East Tower  
Waterside Mall  
401 M Street, S. W.  
Washington, D. C. 20460

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of

NEBRASKA PUBLIC POWER DISTRICT  
(Cooper Nuclear Station)

)  
)  
)

Docket No. 50-298

ORDER FOR MODIFICATION OF LICENSE

1. Nebraska Public Power District (the Licensee) is the holder of Facility Operating License DPR-46 which authorizes operation of the Cooper Nuclear Station (the Facility), a boiling water reactor located at the Licensee's site near Brownville, Nemaha County, Nebraska.
2. For many weeks the Facility has been under close surveillance by the Licensee and by the Nuclear Regulatory Commission due to anomalous incore nuclear detector instrument readings indicative of possible damage to the Facility's fuel channel boxes. As a result of these readings, the Commission has twice issued orders -- once on April 26, 1975 and again on July 22, 1975 -- confirming appropriate limitations on the operation of the Facility. The July 22 order, which superseded the order of April 26, confirmed a plan whereby the Facility was to operate at a maximum bundle power of 3.20 MWt and a maximum core coolant flow of 40 percent of design flow for the balance of a period of approximately 45 equivalent full flow days from April 26, 1975. The plan also contemplated that the

- Licensee would, during that period, operate the Facility at rated flow and power for a limited period to conduct incore tests, including accelerometer tests, and to obtain certain water quality data. At the end of approximately 45 equivalent full flow days from April 26, the Licensee was, under the plan, to shut down the Facility unless the efficacy of the 40 percent flow limit had been verified by accelerometer data. Finally, the limits on core flow and bundle power were to be re-evaluated in light of data collected during the interim period.
3. The license requirements added to Facility Operating License DPR-46 by the order of July 22, 1975 provide, in paragraph 3, that "[t]he Licensee shall shut down the facility following 45 equivalent full flow days from April 26, 1975," unless within that period certain specified tests have been completed with specified results. (See Appendix A of July 22, 1975 order.) As such, the July 22 order arguably admits of the interpretation that the decision date was to be exactly 45 equivalent full flow days from April 26, 1975. This, however, was not the intent of the July 22 order. As paragraphs 7 and 8 of the same order clearly imply, the approved plan contemplated restricted operation of the Facility for the balance of a period of approximately 45 equivalent full flow days from April 26, 1975.
4. By letter dated July 31, 1975, the Licensee advised that on or about August 3, 1975, the Facility will have completed 45 equivalent full

3.

flow days of operation counted from the April 26 reference date. However, according to the letter, the Licensee does not yet have on site all the equipment needed for the accelerometer tests. For this reason (and because of the desirability, from a power demand standpoint, to schedule the installation of the testing equipment during a weekend or other low load period), the Licensee requested in its July 31 letter authority to operate the Facility for ten additional equivalent full flow days, i.e., to a total of 55 equivalent full flow days from April 26, 1975.

5. As stated above, limitations on the operation of the Facility were imposed as a result of anomalous incore instrument readings. Those anomalous readings appear to have been caused by instrument tube vibration. The limitations were designed to reduce the vibration to acceptable levels. On the basis of recent data obtained at the Facility, there is reason to believe that the limitations have been effective in minimizing channel box wear.
  
6. Operation of the Facility for the full additional period requested in the Licensee's July 31 letter would clearly involve more interim operation than that contemplated by the July 22 order. Accordingly, the full request cannot be granted without further evaluation. However, operation of the Facility for five additional equivalent

full flow days (i.e., to a total of 50 equivalent full flow days from April 26, 1975) is acceptable now since operation for that period is within (though near the upper limit of) the approximately 45 equivalent full flow day period contemplated in the July 22 order. To the extent that the July 22 order, by using the term "45 equivalent full flow days" rather than "approximately 45 equivalent full flow days," may be construed as precluding such additional interim operation, the July 22 order should be clarified.

7. Accordingly, pursuant to the Atomic Energy Act of 1954, as amended, and the Commission Rules and Regulations in 10 CFR Part 2 and 50, Facility Operating License DPR-46 is hereby amended by substituting for paragraph 3 of the license provisions added by the Order for Modification of License dated July 22, 1975 the following:

3. The Licensee shall shut down the facility following approximately 45 equivalent full flow days from April 26, 1975, unless within such period a traversing incore probe surveillance program as specified in Section VIII of the Evaluation of Channel Box Wear dated July 22, 1975, has been completed and accelerometer tests performed with appropriate requirements specified in Section VII of the Evaluation of Channel Box Wear dated July 22, 1975 have been completed which indicate the efficacy of the 40 percent flow limit.

FOR THE NUCLEAR REGULATORY COMMISSION



Ben C. Rusche, Director  
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland  
this 1st day of August, 1975.