

NOV 0 - 1975

Enclosures:

1. Amendment No. 13
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

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NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 30-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPK-46

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District (the licensee) dated December 2, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPK-46 is hereby amended to read as follows:

OFFICE						
SURNAME						
DATE						

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 16."

3. This license amendment becomes effective 30 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

Attachment:
Change No. 16 to the
Technical Specifications

Date of Issuance:

NOV 04 1975

OFFICE						
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DATE						

ATTACHMENT TO LICENSE AMENDMENT NO. 13

CHANGE NO. 16 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace pages i, iii, 1, 5, 5a, 221, 224, 233, 235, 236, 237, 238, 239, 240 and 241 of the Appendix A portion of the Technical Specifications with the attached pages bearing the same numbers (except that there is no revised page 5a) and additional page 241a. Changed areas on the revised pages are reflected by marginal lines.

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*Note: This Section is included in Appendix B, "Environmental Technical Specifications."

1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the specifications may be achieved.

A. Thermal Parameters

1. Critical Power Ratio (CPR) - The critical power ratio is the ratio of that assembly power which causes some point in the assembly to experience transition boiling to the assembly power at the reactor condition of interest as calculated by application of the GEXL correlation. (Reference NEDO-10958)
2. Maximum Total Peaking Factor - The Maximum Total Peaking Factor (MTPF) is the lowest Total Peaking Factor which limits a fuel type to a Linear Heat Generation Rate (LHGR) corresponding to the operating limit at 100% power.
3. Minimum Critical Power Ratio (MCPR) - The minimum in-core critical power ratio corresponding to the most limiting fuel assembly in the core.
4. Total Peaking Factor - The ratio of the maximum fuel rod surface heat flux in an assembly to the average surface heat flux of the core.
5. Transition Boiling - Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.

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- B. Alteration of the Reactor Core - The act of moving any component in the region above the core support plate, below the upper grid and within the shroud. Normal control rod movement with the control rod drive hydraulic system is not defined as a core alteration. Normal movement of in-core instrumentation is not defined as a core alteration.
- C. Cold Condition - Reactor coolant temperature equal to or less than 212°F.
- D. Design Power - Design power means a steady-state power level of 2486 thermal megawatts. This is 105% of Rated Power and the power to which the safety analysis applies.
- E. Engineered Safeguard - An engineered safeguard is a safety system the actions of which are essential to a safety action required to maintain the consequences of postulated accidents within acceptable limits.

- U. Safety Limits - The safety limits are limits within which the reasonable maintenance of the fuel cladding integrity and the reactor coolant system integrity are assured. Violation of such a limit is cause for unit shut-down and review by the Atomic Energy Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences but it indicates an operational deficiency subject to regulatory review.
- V. Secondary Containment Integrity - Secondary containment integrity means that the reactor building is intact and the following conditions are met:
1. At least one door in each access opening is closed.
 2. The standby gas treatment system is operable.
 3. All automatic ventilation system isolation valves are operable or secured in the isolated position.
- W. Shutdown - The reactor is in a shutdown condition when the mode switch is in the "Shutdown" or "Refuel" position.
1. Hot Shutdown means conditions as above with reactor coolant temperature greater than 212°F.
 2. Cold Shutdown means conditions as above with reactor coolant temperature equal to or less than 212°F and the reactor vessel vented.
- X. Surveillance Frequency - Unless otherwise stated in these specifications, periodic surveillance tests, checks, calibrations, and examinations shall be performed within the specified surveillance intervals. These intervals may be adjusted plus or minus 25%. The operating cycle interval as pertaining to instrument and electrical surveillance shall never exceed 15 months. In cases where the elapsed interval has exceeded 100% of the specified interval, the next surveillance interval shall commence at the end of the original specified interval.
- Y. Surveillance Interval - The surveillance interval is the calendar time between surveillance tests, checks, calibrations and examinations to be performed upon an instrument or component when it is required to be operable. These tests may be waived when the instrument, component or system is not required to be operable, but the instrument, component or system shall be tested prior to being declared operable or as practicable following its return to service.

6.2 (cont'd.)

- f. Investigate all reported instances of violations of Technical Specifications, including reporting evaluation and recommendations to prevent recurrence, to the Power Production Manager and to the Chairman of the NPPD Safety Review and Audit Board.
- g. Perform special reviews and investigations and render reports thereon as requested by the Chairman of the Nuclear Safety Review and Audit Board.
- h. Review all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours. | 16
- i. Review drills on emergency procedures (including plant evacuation and adequacy of communication with off site groups.
- j. Review all procedures required by these Technical Specifications, including procedures of the Emergency Plan and the Security Plan with a frequency commensurate with their safety significance but at an interval of not more than two years.

5. Authority

- a. The Station Operations Review Committee shall be advisory.
- b. The Station Operations Review Committee shall recommend to the Station Superintendent approval or disapproval of proposals under items 4, a through e and j above. In case of disagreement between the recommendations of the Station Operations Review Committee and the Station Superintendent, the course determined by the Station Superintendent to be the more conservative will be followed. A written summary of the disagreement will be sent to the Power Production Manager and to the NPPD Safety Review and Audit Board.
- c. The Station Operations Review Committee shall report to the Chairman of the NPPD Safety Review and Audit Board on all reviews and investigations conducted under items 4, f, g, h, and i.
- d. The Station Operations Review Committee shall make tentative determinations regarding whether or not proposals considered by the Committee involve unreviewed safety questions. This determination shall be subject to review and approval by the NPPD Safety Review and Audit Board.

6. Records:

Minutes shall be kept for all meetings of the Station Operations Review Committee and shall include identification of all documen-

6.2 (cont'd.)

- c. Proposed changes in procedures, equipment, or systems which may involve an unreviewed safety question as defined in § 50.59(c) Part 50, Title 10, Code of Federal Regulations; or changes which are referred by the operating organization.
 - d. Proposed changes in Technical Specifications or licenses.
 - e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having safety significance.
 - f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment.
 - g. All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours. | 16
 - h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
 - i. Minutes of meetings of the Station Operations Review Committee to determine if matters considered by the committee involve unreviewed safety questions or changes to the operating license.
 - j. Training, qualification and performance of operating staff.
 - k. Disagreement between the recommendations of the Station Operations Review Committee and the Plant Superintendent.
 - l. Security and emergency plans and their implementing procedures.
 - m. Environmental Monitoring Program and its results.
 - n. Quality Assurance program.
 - o. Review of events covered under e, f, g, and h above include reporting to appropriate members of management on the results of investigations and recommendations to prevent or reduce the probability of recurrence.
5. Authority: The NPPD Safety Review and Audit Board shall be advisory to the General Manager and shall have authority to:
- a. Approve proposed changes to the operating license including Technical Specifications and Safety Analysis Report for submission to the NRC.

6.4 Actions to be Taken in the Event of Occurrences Specified in Section 6.7.2.A.

6.4.1 Occurrences, as specified in Section 6.7.2.A., shall be promptly reported to the Station Superintendent, Director of Power Supply and the Chairman of the NPPD Safety Review and Audit Board and shall be promptly reviewed by the Station Operations Review Committee. This committee shall prepare a separate report. This report shall include an evaluation of the cause of the occurrence, a record of the corrective action taken, and recommendations for appropriate action to prevent or reduce the probability of a repetition of the occurrence. Copies of all such reports shall be submitted to the Power Supply Department and the NPPD Safety Review and Audit Board Chairman for review and approval of any recommendations.

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6.4.2 All occurrences as specified in Section 6.7.2.A. shall be reported to the General Manager on a periodic basis.

6.6 Station Operating Records

6.6.1 Records and/or logs relative to the following items shall be kept in a manner convenient for review and shall be retained for at least 5 years unless a longer period is required by applicable regulations.

- A. Records of normal station operation, including power levels and periods of operation at each power level.
- B. Records of periodic checks, inspection and/or calibrations performed to verify that Surveillance Requirements are being met.
- C. Record of changes to plant procedures.
- D. Records of special tests and experiments.
- E. Records of wind speed and direction.

6.6.2 Records and logs relating to the following items shall be kept for the life of the plant.

- A. Records of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety.
- B. Records of occurrences and safety limit violations as specified in 6.4 and 6.5.
- C. Records of changes made to the station as described in the Safety Analysis Report and amendments and reflected in updated, corrected and as-built drawings and records.
- D. Records of new and spent fuel inventory and assembly histories.
- E. Records of station radiation and contamination surveys.
- F. Records of off-site environmental monitoring surveys.
- G. Records of radiation exposure for all station personnel, including all contractors and visitors to the station in accordance with 10 CFR 20.
- H. Records of radioactivity in liquid and gaseous wastes released to the environment.
- I. Design Fatigue Usage Evaluation
 - 1. Monitoring, recording, evaluating, and reporting requirements contained in Section 6.7.3.A.2, will be met for various portions of the reactor coolant pressure boundary (RCPB) for which detailed fatigue

6.7. Station Reporting Requirements

6.7.1 Routine Reports

A. In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate NRC Regional Office of Inspection and Enforcement unless otherwise noted.

B. Startup Report

1. A summary report of plant startup and power escalation testing shall be submitted following:
 - a. Receipt of an operating license.
 - b. Amendment to the license involving a planned increase in power level.
 - c. Installation of fuel that has a different design or has been manufactured by a different fuel supplier.
 - d. Modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

The report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

2. Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

C. Annual Operating Report

1. Routine operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to March 1 of each year.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

2. Each annual operating report shall include:

- a. A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 6.7.1.C.2.b.5 below.
- b. For each outage or forced reduction in power^{1/} of over twenty percent of design power level where the reduction extends for greater than four hours:
 1. The proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction).
 2. A brief discussion of (or reference to reports of) reportable occurrences pertaining to the outage or power reduction. (Ref. Section 6.7.2)
 3. Corrective action taken to reduce the probability of recurrence, if appropriate.
 4. Operating time lost as a result of the outage or power reduction (for scheduled or forced outages,^{2/} use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power).
 5. A description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

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^{1/} The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

^{2/} The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

6. A report of any single release of radio-activity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

c. A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,^{3/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

d. Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

D. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.7.2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

^{3/} This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

A. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

1. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 6.7.2.A.5, 6.7.2.A.6, or 6.7.2.B.1 below,

2. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 6.7.2.B.2 below.

3. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

4. Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to $1\% \Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than $0.5\% \Delta k/k$ or occurrence of any unplanned criticality.
5. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
6. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.7.2.A.5 and 6.7.2.A.6 ~~reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.7.2.B.2 and 6.7.2.B.3 below.~~

7. Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
8. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
9. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

B. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

1. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
2. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 6.7.2.B.1 and 6.7.2.B.2 need not be reported except where test results themselves reveal a degraded mode as described above.

3. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
4. Abnormal degradation of systems other than those specified in item 6.7.2.A.3 above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.7.3. Unique Reporting Requirementss

Reports shall be submitted to the Director, Divison of Reactor Licensing, USNRC, Washington, D. C. 20555, as follow:

A. Reports on the following areas shall be submitted as noted:

<u>Area</u>	<u>Reference</u>	<u>Submittal Date</u>
1. Secondary Containment Leak Rate Testing(1)	4.7.C.1	90 days after completion of each test.
2. Design Fatigue Usage(2)	6.6.2.I	With Annual Operating Report.

NOTES: (1) Each integrated leak rate test of the secondary containment shall be the subject of a summary technical report. This report should include data on the wind speed, wind direction, outside and inside temperatures during the test, concurrent reactor building pressure, and emergency ventilation flow rate. The report shall also include analyses and interpretations of those data which demonstrate compliance with the specified leak rate limits.

(2) In the Annual Operating Report, a listing of the number of events identified in 6.6.2.I.2.b will be tabulated and compared to the design or allowed quantity of comparable or more severe events. In those cases where recalculation of fatigue usage is required per 6.6.2.I.2.c and the calculated usage exceeds two times the design usage limit of the Code, the report will define the inservice inspections that will be performed on that portion of the RCPB to monitor for crack initiation.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 13 TO FACILITY LICENSE NO. DPR-46

CHANGE NO. 16 TO THE TECHNICAL SPECIFICATIONS

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

Introduction

By letter dated December 2, 1974, Nebraska Public Power District proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-46, for the Cooper Nuclear Station. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrence" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired Regulatory position. These changes were discussed with the licensee's staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for the reporting of operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information Appendix A Technical Specifications", Revision 4.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 4, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-298

NEBRASKA PUBLIC POWER DISTRICT

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Facility Operating License No. DPR-46, issued to Nebraska Public Power District, which revised Technical Specifications for operation of the Cooper Nuclear Station located in Nehema County, Nebraska. The amendment becomes effective 30 days after the date of issuance.

This amendment revises the reporting requirements of the Technical Specifications for the Cooper Nuclear Station.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

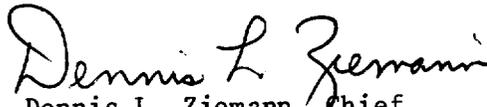
For further details with respect to this action, see (1) the application for amendment dated December 2, 1974, (2) Amendment No. 13 to License No. DPR-46, with Change No. 16, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public

inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Auburn Public Library, 1118 - 15th Street, Auburn, Nebraska 68305.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 4th day of November, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

DATE					
SURNAME					
OFFICE					

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Evaluation

no longer needed for assessment of safety related activities. The proposed reporting requirements would delete any reports identified by the regulations and those unique to this facility. The reports consistent with section 208. The proposed change to required reports consistent with section 208, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required use by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - health or safety. The term "abnormal occurrence" is reserved for Commission determines is significant from the standpoint of public occurrence" is defined as an unscheduled incident or event which the In section 208 of the Energy Reorganization Act of 1974 "abnormal

definition change. The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and abnormal occurrence

Discussion

involve changes to the reporting requirements. By letter dated December 2, 1974, Nebraska Public Power District proposed changes to the Technical Specifications appended to Facility Operating License No. NEF-46, for the Cooper Nuclear Station. The proposed changes

Introduction

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REGULATORY COMMISSION
 SUPPORTING AMENDMENTS NO. 13 TO FACILITY LICENSE NO. NEF-46
 CHAPTER NO. 16 TO THE TECHNICAL SPECIFICATIONS
 NEBRASKA PUBLIC POWER DISTRICT
 COOPER NUCLEAR STATION
 Docket No. 56-298

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired Regulatory position. These changes were discussed with the licensee's staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for the reporting of operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information Appendix A Technical Specifications", Revision 4.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-298

NEBRASKA PUBLIC POWER DISTRICT

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Facility Operating License No. DPR-46, issued to Nebraska Public Power District, which revised Technical Specifications for operation of the Cooper Nuclear Station located in Nehema County, Nebraska. The amendment becomes effective 30 days after the date of issuance.

This amendment revises the reporting requirements of the Technical Specifications for the Cooper Nuclear Station.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated December 2, 1974, (2) Amendment No. 13 to License No. DPR-46, with Change No. 16, and (3) the Commission's concurrently issued

related Safety Evaluation. All of these items are available for public

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inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Auburn Public Library, 1118 - 15th Street, Auburn, Nebraska 68305.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this *4th* day of *November, 1975*.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

Not needed per J. Gallo note of 11-3-75 - see attached.

OFFICE ▶	RL:ORB #2	RL:ORB #2	RL:ORB #2	OELD	RL:ORB #2	
SURNAME ▶	<i>RMDiggs</i>	MHFletcher	RDSilver		DLZiemann	
DATE ▶	10/19/75	10/9/75	10/9/75	10/ /75	10/4/75	

ROUTING AND TRANSMITTAL SLIP		ACTION	
1 TO (Name, office symbol or location) OELD - f/concurrences	INITIALS	CIRCULATE	
	DATE	COORDINATION	
2 DLZiemann - f/signatures	INITIALS	FILE	
	DATE	INFORMATION	
3 Reba - for final checks	INITIALS	NOTE AND RETURN	
	DATE	PER CONVERSATION	
4	INITIALS	SEE ME	
	DATE	SIGNATURE	
REMARKS <p>Attached for your concurrence are five packages (Dresden Station, Quad Cities Station, Cooper, Pilgrim and Calvert Cliffs) of nine from ORB 2 which incorporate standard reporting requirement sections into the Appendix A Technical Specifications. One package, Pilgrim also revises the entire administrative controls section.</p> <p>It is requested that, in the interest of review consistency, these packages (and the 4 future reporting requirements packages) be assigned to one OELD reviewer.</p> <p>Questions may be directed to the PM for the particular case or to Mike Fletcher, coordinator for reporting (Exts. 7403, 7450)</p> <p><i>11/3/75 No need for OELD concurrence this time on subject</i></p> <p>Do NOT use this form as a RECORD of approvals, concurrences, disapprovals, clearances, and similar actions</p>			
FROM (Name, office symbol or location) DLZiemann <i>DLF</i>		DATE 11-3-75	
		PHONE 7380	