May 20, 2002

Mr. Harold W. Keiser Chief Nuclear Officer & President PSEG Nuclear LLC - X04 Post Office Box 236 Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NO. 1, REQUEST FOR

ADDITIONAL INFORMATION RE: RELIEF REQUESTS S1-RR-B01 AND

S1-RR-C01 (TAC NO. MB3811)

Dear Mr. Keiser:

By letter dated January 8, 2002, PSEG Nuclear LLC submitted two requests for relief from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, for Salem Nuclear Generating Station, Unit No. 1 (Salem). The requests for relief are for the second 10-year inservice inspection (ISI) interval in which Salem adopted the 1983 Edition of ASME Section XI, including Summer 1983 Addenda, as the Code of record.

The U.S. Nuclear Regulatory Commission staff is reviewing your requests for relief and has determined that additional information is necessary in order to complete its evaluation. We discussed the enclosed request for additional information with your staff during a telephone call on May 10, 2002. During the call, you agreed to respond to this letter within 45 days from the date of this letter. If circumstances result in the need to revise the target date, please contact me at (301) 415-1324, understanding that a delay in your response is likely to adversely affect our review schedule.

Sincerely,

/RA/

Robert J. Fretz, Project Manager, Section 2 Project Directorate I Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-272

Enclosure: Request for Additional Information

cc w/encl: See next page

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OFFICE	PDI-2/PM	PDI-2/LA	PDI-2/SC
NAME	RFretz	TLClark	JClifford
DATE	5/17/02	5/15/02	5/20/02

REQUEST FOR ADDITIONAL INFORMATION

POWER UPRATE AMENDMENT REQUEST

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

By letter dated January 8, 2002, PSEG Nuclear LLC (PSEG) submitted two requests for relief from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, for Salem Nuclear Generating Station, Unit No. 1 (Salem 1). The requests for relief are for the second 10-year inservice inspection (ISI) interval in which Salem 1 adopted the 1983 Edition of ASME Section XI, including Summer 1983 Addenda, as the Code of record.

- 1. PSEG's letter does not identify the start and end dates for the Salem 1 second 10-year ISI interval. Please include this information in your response.
- 2. PSEG submitted relief requests S1-RR-B01 and S1-RR-C01 in accordance with 10 CFR 50.55a(a)(3)(ii). This section of the regulations addresses licensee alternatives where compliance with Code requirements would result in a hardship or unusual difficulty without a compensating increase in the level of quality or safety. Relief Requests S1-RR-B01 and S1-RR-C01, as contained in the letter and attachments, propose no alternatives. Instead, relief is sought for limited examinations, based on accessibility, physical obstructions or material properties of the subject components. No arguments pertaining to hardship or unusual difficulties, or why completing the limited examinations would not provide a compensating increase in quality or safety, is presented. PSEG should review the basis for these requests, then determine and submit the necessary information for a proposed alternative under 10 CFR 50.55a(a)(3)(i) or 10 CFR 50.55a(a)(3)(ii), or submit the relief requests based on impracticality pursuant to 10 CFR 50.55a(g)(5)(iii) (see below).

The licensee may request relief in accordance with 10 CFR 50.55a(g)(5)(iii), if it can be shown that the Code examinations are impractical at their facility. In this case, the licensee must present adequate information to support a determination that the required examinations are impractical (not simply inconvenient) to perform to the extent required by the Code. This information should include drawings, or other physical descriptions, of the component examination areas, including examination coverage(s), weld cross-sections, etc., as necessary to support the request. In addition, the licensee should submit argument(s) as to why the use of other methods would not reasonably increase the examination coverage(s).

For example, in Attachment 1 for S1-RR-B01 of the licensee's submittal (table on page 5 of 11), the limitation discussion for upper longitudinal seam weld 1-RPV-1042B states that the reduced volumetric coverage (66%) is "due to outlet nozzle boss interference." This discussion does not provide an adequate description of the physical limitation, or indicate whether or not other techniques such as different interrogation angles could have practically increased the coverage.

Similar issues exist with the remainder of the components listed in this table, as well as in the attached table for S1-RR-C01. Provide a technical basis for each component, detailing the examination limitation, and discuss whether other examinations may be used to increase or supplement the limited examinations.

PSEG Nuclear LLC

CC:

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