

Mr. William T. Cottle  
President and Chief Executive Officer  
STP Nuclear Operating Company  
South Texas Project Electric  
Generating Station  
P. O. Box 289  
Wadsworth, TX 77483

January 21, 1999

SUBJECT: SOUTH TEXAS PROJECT, UNITS 1 AND 2 - AMENDMENT NOS. 101  
AND 88 TO FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80  
(TAC NOS. MA4139 AND MA4140)

Dear Mr. Cottle:

The Commission has issued the enclosed Amendment Nos. 101 and 88 to Facility Operating License Nos. NPF-76 and NPF-80 for the South Texas Project, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 29, 1998.

The amendments relocate the TS 3/4.3.4 requirements for Turbine Overspeed Protection to the Technical Requirements Manual.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:  
Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-498 and 50-499

Enclosures: 1. Amendment No. 101 to NPF-76  
2. Amendment No. 88 to NPF-80  
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

Docket File	GHill (4)	PUBLIC	OGC
CHawes	TAlexion (2)	WBeckner	PDIV-1 r/f
ACRS	LHurley, RIV	JKilcrease, RIV f/r	KBrockman, RIV
MGamberoni	Adensam (EGA1)	THarris (TLH3)	

Document Name: STPA4139.AMD

OFC	PM/PA	PM/PA4-1	LA/PA4-1	TSB WOB	OGC	D/PA4-1
NAME	MGamberoni	TAlexion	CHawes	BBeckner	WBeckner	JHannon
DATE	12/3/98	12/3/98	12/3/98	12/4/98	12/21/98	1/5/99
COPY	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO

OFFICIAL RECORD COPY

9901250310 990121  
PDR ADOCK 05000498  
P PDR



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

January 21, 1999

Mr. William T. Cottle  
President and Chief Executive Officer  
STP Nuclear Operating Company  
South Texas Project Electric  
Generating Station  
P. O. Box 289  
Wadsworth, TX 77483

SUBJECT: SOUTH TEXAS PROJECT, UNITS 1 AND 2 - AMENDMENT NOS. 101  
AND 88 TO FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80  
(TAC NOS. MA4139 AND MA4140)

Dear Mr. Cottle:

The Commission has issued the enclosed Amendment Nos. 101 and 88 to Facility Operating License Nos. NPF-76 and NPF-80 for the South Texas Project, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 29, 1998.

The amendments relocate the TS 3/4.3.4 requirements for Turbine Overspeed Protection to the Technical Requirements Manual.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Thomas W. Alexion", is written over a horizontal line.

Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-498 and 50-499

Enclosures: 1. Amendment No. 101 to NPF-76  
2. Amendment No. 88 to NPF-80  
3. Safety Evaluation

cc w/encls: See next page

Mr. William T. Cottle  
STP Nuclear Operating Company

South Texas, Units 1 & 2

cc:

Mr. Cornelius F. O'Keefe  
Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
P. O. Box 910  
Bay City, TX 77414

Jack R. Newman, Esq.  
Morgan, Lewis & Bockius  
1800 M Street, N.W.  
Washington, DC 20036-5869

A. Ramirez/C. M. Canady  
City of Austin  
Electric Utility Department  
721 Barton Springs Road  
Austin, TX 78704

Mr. Lawrence E. Martin  
Vice President, Nuc. Assurance & Licensing  
STP Nuclear Operating Company  
P. O. Box 289  
Wadsworth, TX 77483

Mr. M. T. Hardt  
Mr. W. C. Gunst  
City Public Service Board  
P. O. Box 1771  
San Antonio, TX 78296

Office of the Governor  
ATTN: John Howard, Director  
Environmental and Natural  
Resources Policy  
P. O. Box 12428  
Austin, TX 78711

Mr. G. E. Vaughn/C. A. Johnson  
Central Power and Light Company  
P. O. Box 289  
Mail Code: N5012  
Wadsworth, TX 74483

Jon C. Wood  
Matthews & Branscomb  
One Alamo Center  
106 S. St. Mary's Street, Suite 700  
San Antonio, TX 78205-3692

INPO  
Records Center  
700 Galleria Parkway  
Atlanta, GA 30339-3064

Arthur C. Tate, Director  
Division of Compliance & Inspection  
Bureau of Radiation Control  
Texas Department of Health  
1100 West 49th Street  
Austin, TX 78756

Regional Administrator, Region IV  
U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

Jim Calloway  
Public Utility Commission of Texas  
Electric Industry Analysis  
P. O. Box 13326  
Austin, TX 78711-3326

D. G. Tees/R. L. Balcom  
Houston Lighting & Power Co.  
P. O. Box 1700  
Houston, TX 77251

Judge, Matagorda County  
Matagorda County Courthouse  
1700 Seventh Street  
Bay City, TX 77414



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

STP NUCLEAR OPERATING COMPANY

DOCKET NO. 50-498

SOUTH TEXAS PROJECT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101  
License No. NPF-76

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by STP Nuclear Operating Company\* acting on behalf of itself and for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and City of Austin, Texas (COA) (the licensees), dated October 29, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

---

\*STP Nuclear Operating Company is authorized to act for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio, Central Power and Light Company and City of Austin, Texas and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-76 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 101, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: January 21, 1999



UNITED STATES  
**NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

STP NUCLEAR OPERATING COMPANY

DOCKET NO. 50-499

SOUTH TEXAS PROJECT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88  
License No. NPF-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by STP Nuclear Operating Company\* acting on behalf of itself and for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and City of Austin, Texas (COA) (the licensees), dated October 29, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

---

\*STP Nuclear Operating Company is authorized to act for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio, Central Power and Light Company and City of Austin, Texas and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-80 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 88 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas W. Alexion, Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: January 21, 1999

ATTACHMENT TO LICENSE AMENDMENT NOS. 101 AND 88

FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80

DOCKET NOS. 50-498 AND 50-499

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

vi  
xiii  
3/4 3-84  
B 3/4 3-6

INSERT

vi  
xiii  
3/4 3-84  
B 3/4 3-6



## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<b><u>3/4.2 POWER DISTRIBUTION LIMITS</u></b>	
3/4.2.1 AXIAL FLUX DIFFERENCE .....	3/4 2-1
FIGURE 3.2.1 AXIAL FLUX DIFFERENCE LIMITS AS A FUNCTION OF RATED THERMAL POWER .....	Deleted
3/4.2.2 HEAT FLUX HOT CHANNEL FACTOR - $F_0(Z)$ .....	3/4 2-5
FIGURE 3.2-2 $K(Z)$ - NORMALIZED $F_0(Z)$ AS A FUNCTION OF CORE HEIGHT ..	Deleted
3/4.2.3 NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR .....	3/4 2-9
3/4.2.4 QUADRANT POWER TILT RATIO .....	3/4 2-10
3/4.2.5 DNB PARAMETERS .....	3/4 2-11
<b><u>3/4.3 INSTRUMENTATION</u></b>	
3/4.3.1 REACTOR TRIP SYSTEM INSTRUMENTATION .....	3/4 3-1
TABLE 3.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION .....	3/4 3-2
TABLE 3.3-2 (This table number not used) .....	3/4 3-9
TABLE 4.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-11
3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION .....	3/4 3-16
TABLE 3.3-3 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION .....	3/4 3-18
TABLE 3.3-4 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS .....	3/4 3-29
TABLE 3.3-5 (This table number not used) .....	3/4 3-37
TABLE 4.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-42
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring for Plant Operations .....	3/4 3-50
TABLE 3.3-6 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS .....	3/4 3-51
TABLE 4.3-3 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS SURVEILLANCE REQUIREMENTS .....	3/4 3-53
Movable Incore Detectors .....	3/4 3-54

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
Meteorological Instrumentation .....	3/4 3-58
TABLE 3.3-8 METEOROLOGICAL MONITORING INSTRUMENTATION .....	3/4 3-59
TABLE 4.3-5 METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-60
Remote Shutdown System .....	3/4 3-61
TABLE 3.3-9 REMOTE SHUTDOWN SYSTEM .....	3/4 3-62
TABLE 4.3-6 REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-66
Accident Monitoring Instrumentation .....	3/4 3-67
TABLE 3.3-10 ACCIDENT MONITORING INSTRUMENTATION .....	3/4 3-68
TABLE 4.3-7 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-73
Chemical Detection Systems .....	3/4 3-75
TABLE 3.3-11 (This table number is not used) .....	3/4 3-77
Radioactive Liquid Effluent Monitor Instrumentation	DELETED
TABLE 3.3-12 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION	DELETED
TABLE 4.3-8 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	DELETED
Explosive Gas Monitor Rn9 Instrumentation .....	3/4 3-79
TABLE 3.3-13 EXPLOSIVE GAS MONITORING INSTRUMENTATION .....	3/4 3-80
TABLE 4.3-9 EXPLOSIVE GAS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS .....	3/4 3-82
3/4.3.4 (This specification number is not used)	

## INDEX

### BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.0 APPLICABILITY</u> .....	B 3/4 0-1
<u>3/4.1 REACTIVITY CONTROL SYSTEMS</u>	
3/4.1.1 BORATION CONTROL .....	B 3/4 1-1
3/4.1.2 BORATION SYSTEMS .....	B 3/4 1-2
3/4.1.3 MOVABLE CONTROL ASSEMBLIES .....	B 3/4 1-3
<u>3/4.2 POWER DISTRIBUTION LIMITS</u> .....	B 3/4 2-1
3/4.2.1 AXIAL FLUX DIFFERENCE .....	B 3/4 2-1
3/4.2.2 and 3/4.2.3 HEAT FLUX HOT CHANNEL FACTOR and NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR .....	B 3/4 2-2
FIGURE B 3/4.2-1 TYPICAL INDICATED AXIAL FLUX DIFFERENCE VERSUS THERMAL POWER .....	B 3/4 2-3
3/4.2.4 QUADRANT POWER TILT RATIO .....	B 3/4 2-5
3/4.2.5 DNB PARAMETERS .....	B 3/4 2-5
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 and 3/4.3.2 REACTOR TRIP SYSTEM and ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION .....	B 3/4 3-1
3/4.3.3 MONITORING INSTRUMENTATION .....	B 3/4 3-3
3/4.3.4 (This specification number is not used)	
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION .....	B 3/4 4-1
3/4.4.2 SAFETY VALVES .....	B 3/4 4-1
3/4.4.3 PRESSURIZER .....	B 3/4 4-2
3/4.4.4 RELIEF VALVES .....	B 3/4 4-2
3/4.4.5 STEAM GENERATORS .....	B 3/4 4-2
3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE .....	B 3/4 4-3
3/4.4.7 CHEMISTRY .....	B 3/4 4-4
3/4.4.8 SPECIFIC ACTIVITY .....	B 3/4 4-5

## INDEX

### BASES

<u>SECTION</u>	<u>PAGE</u>
3/4.4.9 PRESSURE/TEMPERATURE LIMITS.....	B 3/4 4-6
TABLE B 3/4.4-1a REACTOR VESSEL TOUGHNESS (UNIT 1).....	B 3/4 4-9
TABLE B 3/4.4-1b REACTOR VESSEL TOUGHNESS (UNIT 2).....	B 3/4 4-10
FIGURE B 3/4.4-1 FAST NEUTRON FLUENCE (E>1MeV) AS A FUNCTION OF FULL POWER SERVICE LIFE.....	B 3/4 4-11
3/4.4.10 STRUCTURAL INTEGRITY.....	B 3/4 4-15
3/4.4.11 REACTOR VESSEL HEAD VENTS.....	B 3/4 4-15
 <u>3/4.5 EMERGENCY CORE COOLING SYSTEMS</u>	
3/4.5.1 ACCUMULATORS.....	B 3/4 5-1
3/4.5.2 and 3/4.5.3 ECCS SUBSYSTEMS.....	B 3/4 5-1
3/4.5.4 (This specification number is not used).....	B 3/4 5-2
3/4.5.5 REFUELING WATER STORAGE TANK.....	B 3/4 5-2
3/4.5.6 RESIDUAL HEAT REMOVAL (RHR) SYSTEM .....	B 3/4 5-3
 <u>3/4.6 CONTAINMENT SYSTEMS</u>	
3/4.6.1 PRIMARY CONTAINMENT.....	B 3/4 6-1
3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS.....	B 3/4 6-3
3/4.6.3 CONTAINMENT ISOLATION VALVES.....	B 3/4 6-4
3/4.6.4 COMBUSTIBLE GAS CONTROL.....	B 3/4 6-4
 <u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE.....	B 3/4 7-1
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION.....	B 3/4 7-3
3/4.7.3 COMPONENT COOLING WATER SYSTEM.....	B 3/4 7-3
3/4.7.4 ESSENTIAL COOLING WATER SYSTEM.....	B 3/4 7-3
3/4.7.5 ULTIMATE HEAT SINK.....	B 3/4 7-3
3/4.7.6 (Not used)	
3/4.7.7 CONTROL ROOM MAKEUP AND CLEANUP FILTRATION SYSTEM.....	B 3/4 7-4
3/4.7.8 FUEL HANDLING BUILDING EXHAUST AIR SYSTEM.....	B 3/4 7-4
3/4.7.9 SNUBBERS.....	B 3/4 7-4

**TABLE 4.3-9 (Continued)**

**TABLE NOTATIONS**

- \* (Not used)
- \*\* During GASEOUS WASTE PROCESSING SYSTEM operation.
- (1) (Not Used)
- (2) (Not Used)
- (3) (Not Used)
- (4) (Not Used)
- (5) The CHANNEL CALIBRATION shall include the use of a standard gas sample containing a nominal two volume percent oxygen, balance nitrogen.

**INSTRUMENTATION**

**3/4.3.4 Not Used**

**SOUTH TEXAS - UNITS 1 & 2**

**3/4 3-84**

**Unit 1 - Amendment No. ~~40,47,67~~, 101**  
**Unit 2 - Amendment No. ~~56,56~~, 88**

## INSTRUMENTATION

### BASES

---

#### 3/4.3.3.6 ACCIDENT MONITORING INSTRUMENTATION (Continued)

quadrant. The unit specific response to Item II.F.2 of NUREG-0737 further discusses the core exit thermocouples. Two sets of two thermocouples ensure a single failure will not disable the ability to determine the radial temperature gradient. The subcooling margin monitor requirements are not affected by allowing 2 thermocouples/channel/quadrant as long as each channel has at least four operable thermocouples in any quadrant (e.g., A Train has four operable thermocouples in one of the quadrants, and C Train has four operable thermocouples in the same quadrant or any other quadrant.). This preserves the ability to withstand a single failure.

#### 3/4.3.3.7 (Not Used)

#### 3/4.3.3.8 (Not Used)

#### 3/4.3.3.9 (Not Used)

#### 3/4.3.3.10 DELETED

#### 3/4.3.3.11 EXPLOSIVE GAS MONITORING INSTRUMENTATION

This instrumentation includes provisions for monitoring (and controlling) the concentrations of potentially explosive gas mixtures in the GASEOUS WASTE PROCESSING SYSTEM.

#### 3/4.3.4 (Not Used)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 101 AND 88 TO

FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80

STP NUCLEAR OPERATING COMPANY

DOCKET NOS. 50-498 AND 50-499

SOUTH TEXAS PROJECT, UNITS 1 AND 2

1.0 INTRODUCTION

By application dated October 29, 1998, STP Nuclear Operating Company, et.al., (the licensee) requested changes to the Technical Specifications (TSs) (Appendix A to Facility Operating License Nos. NPF-76 and NPF-80) for the South Texas Project, Units 1 and 2 (STP). The proposed changes would relocate TS 3/4.3.4, Turbine Overspeed Protection and its associated Bases to the STP Technical Requirements Manual.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires that applicants for nuclear power plant operation licenses state TSs and that these TSs be included as a part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls and states also that the Commission may include additional TSs as it finds to be appropriate. However, the regulation does not specify the particular TSs to be included in a plant's license.

The Commission has provided guidance for the contents of TS limiting conditions for operation (LCO) and associated requirements in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), 58 FR 39132 (July 22, 1993), which was codified in 10 CFR 50.36(c)(2)(ii). The four criteria to be used in determining whether a particular matter is required to be included in the TS LCO, are as follows:

(1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; or (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.



### 3.0 EVALUATION

The existing TS 3/4.3.4 Conditions, Actions, and Surveillance Requirements for Turbine Overspeed Protection are relocated to the Technical Requirements Manual (TRM). The turbine overspeed protection system instrumentation is not considered to prevent or mitigate any design basis accident or transient.

Although the design basis accidents and transients include a variety of system failures and conditions which might result from turbine missiles striking various plant systems and equipment, the system failures and plant conditions could be caused by other events as well as turbine failures. In view of the low likelihood of turbine missiles, this scenario does not constitute a part of the primary success path to prevent or mitigate such design basis accidents and transients. Similarly, the turbine overspeed control is not part of an initial condition of a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

The above relocated requirements relating to installed plant instrumentation are not required to be in the TSs under 10 CFR 50.36, because they do not fall within any of the four criteria discussed above and are therefore, not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. In addition, since the licensee has incorporated the TRM, by reference, into the Updated Final Safety Analysis Report, changes to the TRM would be controlled in accordance with approved station procedures and the requirements of 10 CFR 50.59. The staff, therefore, finds that sufficient regulatory controls exist. Accordingly, the staff has concluded that these requirements may be relocated from the TSs to the licensee's TRM.

In the October 29, 1998, application, the licensee provided the revised TRM pages that incorporate the LCO and surveillance requirements from TS 3/4.3.4 and the associated Bases information. The staff has reviewed the TRM change and has verified that the information from TS 3/4.3.4 has been appropriately relocated and that changes to the TRM will be controlled under 10 CFR 50.59.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The Nuclear Regulatory Commission staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (63 FR 69347). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Marsha Gamberoni

Date: January 21, 1999.