

May 8, 2002

MEMORANDUM TO: James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Richard B. Ennis, Sr. Project Manager, Section 2 /RA/
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2,
FACSIMILE TRANSMISSION, ISSUES TO BE DISCUSSED IN AN
UPCOMING CONFERENCE CALL (TAC NO. MB2639)

The attached information was transmitted by facsimile on May 8, 2002, to Mr. David Dodson of Dominion Nuclear Connecticut, Inc. (the licensee). This information was transmitted to facilitate a upcoming conference call in order to clarify the licensee's submittal dated April 30, 2002, pertaining to Nuclear Regulatory Commission (NRC) Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles." This memorandum and the attachment do not convey a formal request for information or represent an NRC staff position.

Docket No. 50-336

Attachment: Issues for Discussion in Upcoming Telephone Conference

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**Issues for Discussion in Upcoming Telephone Conference
Related to NRC Bulletin 2001-01
“Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles”
Millstone Nuclear Power Station, Unit No. 2**

NRC Bulletin 2001-01, Question 5, part a, requests licensees to provide, within 30 days after plant restart following the next refueling outage, a description of the extent of vessel head penetration (VHP) nozzle leakage and cracking detected at your plant, including the number, location, size, and nature of each crack detected. Dominion Nuclear Connecticut, Inc., provided the 30-day response for Question 5 for Millstone Unit No. 2 by letter dated April 30, 2002. The staff has reviewed your response and would like to discuss the following issues:

- 1) The response stated that three control element drive mechanism (CEDM) nozzles were determined to contain indications of discontinuities that could be attributed to service induced degradation and that indications were found in CEDM Nozzles 21, 34 and 50. The response included tables to provide details regarding the indications found in each of the 3 nozzles. The tables for Nozzles 34 and 50 each contain a row to provide the depth in inches for the indications found. However, the table for Nozzle 21 does not provide depth information. Please provide this information.
- 2) The information in the three tables states that the location of each of the indications found is “OD downhill.” The staff requests more specific information regarding the locations. If possible, please provide information similar to that provided in Figures 8 through 12 of the Root Cause Analysis Report for Davis-Besse dated April 18, 2002. These figures are shown on the NRC website at:

<http://www.nrc.gov/reactors/operating/ops-experience/vessel-head-degradation/vessel-head-degradation-files/figures-cr02-0891.pdf>

ATTACHMENT