April 25, 1995

Mr. Jerry W. Yelverton
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

SUBJECT: ISSUANCE OF AMENDMENT NOS. 179 AND 160 TO FACILITY OPERATING LICENSE NOS. DPR-51 AND NPF-6 - ARKANSAS NUCLEAR ONE, UNITS 1 AND 2 (TAC NOS. M89815 AND M89816)

Dear Mr. Yelverton:

The Commission has issued the enclosed Amendment Nos. 179 and 160 to Facility Operating License Nos. DPR-51 and NPF-6 for the Arkansas Nuclear One, Unit Nos. 1 and 2 (ANO-1&2). These amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated June 20, 1994.

The amendments relocate the requirements of the quality assurance program, and the security and emergency plans from the administrative controls section of the TSs to the respective licensee-controlled documents.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:

George Kalman, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects- III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-313
and 50-368

Enclosures: 1. Amendment No. 179 to DPR-51
2. Amendment No. 160 to NPF-6
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

Docket File	PUBLIC	PD4-1 r/f	L. Hurley, RIV
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OPA (2G5)	OC/LFMB	RSchaaf	

Document Name: AR89815.AMD *See previous concurrence

OFC	LA/PD4-1	PM/PD4-1	PM/PD4-1	TQMB*	OTSB*	OGC*
NAME	PNoonan	RSchaaf	GKalman	SBlack	CGrimes	CMarco
DATE	4/19/95	4/25/95	4/25/95	3/31/95	3/16/95	4/13/95
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 25, 1995

Mr. Jerry W. Yelverton
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

SUBJECT: ISSUANCE OF AMENDMENT NOS. 179 AND 160 TO FACILITY OPERATING LICENSE NOS. DPR-51 AND NPF-6 - ARKANSAS NUCLEAR ONE, UNITS 1 AND 2 (TAC NOS. M89815 AND M89816)

Dear Mr. Yelverton:

The Commission has issued the enclosed Amendment Nos. 179 and 160 to Facility Operating License Nos. DPR-51 and NPF-6 for the Arkansas Nuclear One, Unit Nos. 1 and 2 (ANO-1&2). These amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated June 20, 1994.

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Project Directorate IV-1
Division of Reactor Projects- III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-313
and 50-368

Enclosures: 1. Amendment No. 179 to DPR-51
2. Amendment No. 160 to NPF-6
3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 179
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Entergy Operations, Inc. (the licensee) dated June 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 179 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Kalman

George Kalman, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 179

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Revise the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES	INSERT PAGES
iia	iia
117a	117a
121	121
122	122
122a	122a
123	123
124	124
125	125
125a	125a
126	126
127	127

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President, Operations ANO shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the QA Manual Operations.

b. The General Manager, Plant Operations shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

c. The Vice President, Operations ANO shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

FACILITY STAFF

6.2.2 The Manager, Operations and Shift Supervisor shall hold a senior reactor operator license. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

6.2.2.1 Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety-related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter 82-12).

6.3. FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for (1) the designated radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager, Training and Emergency Planning and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager, Training and Emergency Planning and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade training sessions which shall be held at least quarterly.

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6.6 REPOR LE EVENT ACTION

6.6.1 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

6.6.2 The following actions shall be taken for REPORTABLE EVENTS:

a. A report shall be submitted to the Commission pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and

b. Each REPORTABLE EVENT shall be reviewed by the PSC, and the results of this review shall be submitted to the SRC and the Vice President, Operations ANO.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least hot shutdown within one hour.

b. The Nuclear Regulatory Commission shall be notified pursuant to 10 CFR 50.72 and a report submitted pursuant to the requirements of 10 CFR 50.36 and Specification 6.6.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. (Deleted)

e. (Deleted)

f. Fire Protection Program Implementation.

g. New and spent fuel storage.

h. Offsite Dose Calculation Manual and Process Control Program implementation at the site.

i. Post accident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, and changes in intent thereto, shall be reviewed and approved as required by the QAMO prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Changes to procedures of 6.8.1 above may be made and implemented prior to obtaining the review and approval required in 6.8.2 above provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on Unit 1.

c.. The change is documented, reviewed and approved as required by the QAMO, within 14 days of implementation.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 160
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Entergy Operations, Inc. (the licensee) dated June 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 160, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Kalman, Senior Project Manger
Project Directorate IV-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: April 25, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 160

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Revise the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES	INSERT PAGES
XVI	XVI
6-5	6-5
6-5a	6-5a
6-6	6-6
6-7	6-7
6-8	6-8
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DESIGN FEATURES

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the designated radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager, Training and Emergency Planning and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

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6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

a. A report shall be submitted to the Commission pursuant to the requirements of Section 50.73 to 10CFR Part 50, and

b. Each REPORTABLE EVENT shall be reviewed by the PSC and, the results of this review shall be submitted to the SRC and the Vice President, Operations ANO.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The unit shall be placed in at least HOT STANDBY within one hour.
b. The Vice President, Operations ANO and the SRC shall be notified within 24 hours.
c. The Nuclear Regulatory Commission shall be notified pursuant to 10CFR50.72 and a report submitted pursuant to the requirements of 10CFR50.36 and Specification 6.6.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
b. Refueling operations.
c. Surveillance and test activities of safety related equipment.
d. (Deleted)
e. (Deleted)
f. Fire Protection Program implementation.
g. Modification of Core Protection Calculator (CPC) Addressable Constants. These procedures should include provisions to assure that sufficient margin is maintained in CPC Type I addressable constants to avoid excessive operator interaction with the CPCs during reactor operation.

NOTE: Modifications to the CPC software (including changes of algorithms and fuel cycle specific data) shall be performed in accordance with the most recent version of "CPC Protection Algorithm Software Change Procedure," CEN-39(A)-P that has been determined to be applicable to the facility. Additions or deletions to CPC addressable constants or changes to addressable constant software limit values shall not be implemented without prior NRC approval.

h. New and spent fuel storage.
i. ODCM and PCP implementation.
j. Postaccident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluent, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, and changes in intent thereto, shall be reviewed by the PSC and approved by the General Manager, Plant Operations, Plant Manager, ANO-2 or responsible Major Department Head prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Changes to procedures of 6.8.1 above may be made and implemented prior to obtaining the review and approval required in 6.8.2 above provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on Unit 2.

c. The change is documented, reviewed and approved as required by the QAMO, within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Administrator of the Regional Office unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 179 AND 160 TO

FACILITY OPERATING LICENSE NOS. DPR-51 AND NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NOS. 1 AND 2

DOCKET NOS. 50-313 AND 50-368

1.0 INTRODUCTION

By letter dated June 20, 1994, Entergy Operations, Inc. (the licensee) submitted a request for changes to the Arkansas Nuclear One, Unit Nos. 1 and 2 (ANO-1&2), Technical Specifications (TSs). The requested changes would relocate the requirements of the quality assurance (QA) program, and the security and emergency plans from the administrative controls section of the TSs to the respective licensee-controlled documents.

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

The Commission has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies §182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co.* (Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TSs, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.[1] As a result, existing TS requirements which fall within or satisfy any of the criteria in the Final Policy Statement must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

2.0 EVALUATION

As summarized above, the Commission's policy statement provides that existing TS requirements which do not satisfy or fall within any of the four specified criteria may be relocated to appropriate licensee-controlled documents. The licensee proposed relocation of the review and audit requirements of TS 6.5 to the QA, emergency, or security plans, as appropriate.

The licensee proposed that the review and audit functions specified in existing TSs 6.5.1, 6.5.2, and 6.5.3 be relocated from the TSs to the QA program on the basis that they are adequately controlled by existing NRC requirements. These TS provisions are not necessary to assure safe operation of the facility, given that the requirements in the QA program implement the Commission's regulations pertaining to these review and audit functions as set forth below. The security and emergency plans' review and audit functions are relocated to their respective plans in accordance with Generic Letter (GL) 93-07. The emergency and security plans implement the Commission's regulations discussed below for these review and audit functions. Such an approach would

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1 The Commission recently promulgated a proposed change to 10 CFR 50.36, pursuant to which the rule would be amended to codify and incorporate these criteria (59 FR 48180). The Commission's Final Policy Statement specified that Reactor Core Isolation Cooling, Isolation Condenser, Residual Heat Removal, Standby Liquid Control, and Recirculation Pump Trip are to be included in the TS under Criterion 4. In the proposed change to §50.36, the Commission specifically requested public comments regarding application of Criterion 4. Until additional guidance has been developed, Criterion 4 is not being applied to add TS restrictions other than those indicated above.

result in an equivalent level of regulatory authority while providing for a more appropriate change control process. In addition, the following considerations support relocating these items from the TSs:

a. The on-site review function, composition, alternate membership, meeting frequency, quorum, responsibilities, authority and records for the plant safety committee (PSC) are all covered in equivalent detail in the licensee's commitment to the guidance in ANSI N18.7-1976 as documented in its 10 CFR Part 50, Appendix B QA program description. Control of changes to the QA program description are governed by the provisions of 10 CFR 50.54(a).

b. The off-site review group, the safety review committee (SRC), is also addressed in equivalent detail in the licensee's 10 CFR Part 50, Appendix B QA program description. Therefore, duplicating the review and audit function of the off-site review group in the TSs is unnecessary.

c. Audit requirements are specified in the QA program to satisfy 10 CFR Part 50, Appendix B, Criterion XVIII. The licensee has committed to or relies upon the guidance in ANSI N18.7 and ANSI N45.2 to meet the requirements of Appendix B to 10 CFR Part 50. Audits are also governed by 10 CFR 50.54(t), 10 CFR 50.54(p), and 10 CFR Part 73. Therefore, duplication of these requirements does not enhance the level of plant safety.

The staff concludes that sufficient regulatory controls exist for the QA program such that removing these provisions from the TSs and relocating them to the QA program description under the controls of 10 CFR 50.54(a) and 10 CFR Part 50 Appendix B is acceptable.

The licensee proposes to relocate the requirements to establish, implement, and maintain procedures related to the Emergency Plan (existing TS 6.8.1.e) and security plan (existing TS 6.8.1.d). Since the Security Plan requirements are specified in 10 CFR 50.54, 73.40, 73.55, and 73.56 and the emergency plan requirements are specified in 10 CFR 50.54(q) and 10 CFR Part 50, Appendix E, Section V, the staff proposed removal of these requirements from TSs and relocation to their respective plans in GL 93-07.

The requirements in the existing TSs for the review of the security program and implementing procedures, and for the review of the station emergency plan and implementing procedures, will be included in the respective plans. Further changes in these review requirements must be made in accordance with 10 CFR 50.54(p) for the security plan and 10 CFR 50.54(q) for the emergency plan.

The staff concludes that the extensive requirements for emergency planning in 10 CFR 50.47, 50.54, 10 CFR Part 50, Appendix E, and for security plans in 10 CFR 50.54 and 73.55, for drills, exercises, testing, and maintenance of the

program, provide adequate assurance that the objective of the previous TSs for a periodic review of the program and changes to the programs will be met.

Duplication of the requirements contained in the regulations would not enhance the level of safety for the facility. The staff concludes that other regulatory requirements provide sufficient control of these provisions and removing them from TSs is acceptable.

The licensee proposed to revise TS 6.8.2 (for both units) and TS 6.8.3 (for ANO-1 to refer to the QA program documentation in lieu of TS 6.5 for the processes for documentation, review and approval of procedure changes (TS 6.8.2), and temporary procedure changes (TS 6.8.3).

The licensee will continue to implement a QA program in accordance with the requirements of 10 CFR Part 50, Appendix B, which provides appropriate controls for the review and approval of procedure changes. Changes to the QA program, including departures from the ANSI standard that constitute a reduction in commitment, will be governed by 10 CFR 50.54(a). The staff concludes that these regulatory requirements provide sufficient control of these provisions and removing them from the TSs is acceptable.

The licensee-controlled documents to which these requirements are being relocated include provisions for making changes consistent with the applicable regulatory requirements. The QA program documentation and implementing procedures may be changed in accordance with 10 CFR 50.54(a) and 10 CFR Part 50, Appendix B; the security plan and implementing procedures may be changed in accordance with 10 CFR 50.54(p), and the emergency plan and implementing procedures may be changed in accordance with 10 CFR 50.54(q).

On this basis, the staff concludes that these requirements are not required to be in the TSs under 10 CFR 50.36 or §182a of the Atomic Energy Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. Further, they do not fall within any of the four criteria set forth in the Commission's Final Policy Statement, discussed above. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.59, 10 CFR 50.54, and 10 CFR Part 50, Appendix B to adequately control future modifications to these requirements. Accordingly, the staff has concluded that these requirements may be relocated from the TSs to their respective licensee-controlled documents.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comment.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert G. Schaaf

Date: April 25, 1995