

May 19, 1994

Mr. Jerry W. Yelverton
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

Dear Mr. Yelverton:

SUBJECT: ISSUANCE OF AMENDMENT NO. 172 TO FACILITY OPERATING LICENSE
NO. DPR-51 - ARKANSAS NUCLEAR ONE, UNIT NO. 1 (TAC NO. M86264)

The Commission has issued the enclosed Amendment No. 172 to Facility Operating License No. DPR-51 for the Arkansas Nuclear One, Unit No. 1 (ANO-1). This amendment consists of changes to the Technical Specifications (TS) in response to your application dated March 19, 1993.

The amendment changes TS 4.18.6 and Table 4.18-2 to make the requirements for C-3 reports on steam generator tube inspections consistent with the Babcock & Wilcox Standard TS. This deletes the current requirement for prior NRC approval of the remedial actions described in the C-3 report.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:
George Kalman, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 172 to DPR-51
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION

Docket File	NRC/Local PDR	PD4-1 Reading	OGC (15B18)
E. Adensam	P. Noonan	G. Kalman (2)	OPA (2G5)
D. Hagan (3206)	G. Hill (2)	W. Johnson, RIV	J. Roe
C. Grimes (11E22)	ACRS (10) (P315)	OC/LFMB (4503)	W. Beckner

* See previous concurrence

w/ change noted

OFC	LA:PD4-1	PM:PD4-1	EMCB*	OGC	D:PD4-1 <i>WD</i>
NAME	<i>P Noonan</i>	<i>G Kalman: pk</i>	JStrosnider	<i>OGC</i>	WBeckner
DATE	<i>5/14/94</i>	<i>5/14/94</i>	4/29/94	<i>5/19/94</i>	<i>5/19/94</i>
COPY	YES/NO	YES/NO	NO	YES/NO	YES/NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 19, 1994

Docket No. 50-313

Mr. Jerry W. Yelverton
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

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A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, reading "George Kalman", is written over a horizontal line.

George Kalman, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 172 to DPR-51
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Jerry W. Yelverton
Entergy Operations, Inc.

Arkansas Nuclear One, Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS INC.

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 172
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated March 19, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 172, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

William D. Beckner

William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 19, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 172

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES

110n

110o1

INSERT PAGES

110n

110o1

4.18.6 Reports

Following each inservice inspection of steam generator tubes, the complete results of the inspection shall be reported to the NRC. This report, to be submitted within 45 days of inspection completion, shall include:

- a. Number and extent of tubes inspected;
- b. Location and percent of wall-thickness penetration for each indication of an imperfection; and
- c. Identification of tubes plugged and tubes sleeved.

This report shall be in addition to a Special Report (per Specification 6.12.5.d) required for the results of steam generator tube inspections which fall into Category C-3 as denoted in Table 4.18-2. The Commission shall be notified of the results of steam generator tube inspections which fall into Category C-3 prior to resumption of plant operation. The written Special Report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

Bases

The surveillance requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

TABLE 4.18-2

STEAM GENERATOR TUBE INSPECTION^{2, 3}

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S.G. ¹	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug or sleeve defective tubes and inspect additional 2S tubes in this S.G.	C-1	None	N/A	N/A
			C-2	Plug or sleeve defective tubes and inspect additional 4S tubes in this S.G.	C-1	None
			C-3	Perform action for C-3 result of first sample.	C-2	Plug or sleeve defective tubes
					C-3	Perform action for C-3 result of first sample
	C-3	Inspect all tubes in this S.G. plug or sleeve defect- ive tubes and in- spect 2S tubes in other S.G. Special Report to NRC pursuant to 6.12.5.d.	C-3	Perform action for C-3 result of first sample.	N/A	N/A
			Other S.G. is C-1	None	N/A	N/A
			Other S.G. is C-2	Perform action for C-2 results of second sample.	N/A	N/A
			Other S.G. is C-3	Inspect all tubes in each S.G. and plug or sleeve defective tubes. Special Report to NRC pursuant to 6.12.5.d.	N/A	N/A

NOTES: ¹ $S = 3 \frac{N}{n} \%$ Where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection.

²For tubes inspected pursuant to 4.18.3.a.3: No action is required for C-1 results. For C-2 results in one or both steam generators plug or sleeve defective tubes. For C-3 results in one or both steam generators, plug or sleeve defective tubes and provide a Special Report to NRC pursuant to 6.12.5.d.

³No more than ten thousand (10,000) sleeves may be installed in both ANO-1 steam generators combined.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 172 TO

FACILITY OPERATING LICENSE NO. DPR-51

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 1

DOCKET NO. 50-313

1.0 INTRODUCTION

By letter dated March 19, 1993, Entergy Operations, Inc. (the licensee) submitted a request for changes to the Arkansas Nuclear One, Unit No. 1 (ANO-1) Technical Specifications (TSs). The requested changes would modify TS 4.18.6 and Table 4.18-2 to make the requirements for C-3 reports on steam generator (SG) tube inspections consistent with the Babcock & Wilcox Standard TS. This is done by deleting the current requirement for prior NRC approval of the remedial actions described in the C-3 report.

2.0 EVALUATION

As currently written TS 4.18.6, 6.12.5.d, and Table 4.18-2 for ANO-1 require that, if the results of an inservice inspection (ISI) of the tubes of either or both SGs is characterized as C-3 (that is, if more than 10% of the total tubes inspected are degraded or more than 1% of the inspected tubes are defective), a special report shall be submitted to the NRC prior to resumption of plant operation. This report is to contain the SG tube inspection results and a description of the remedial action taken. As the TS are currently written, the remedial action described must also receive NRC approval.

This requirement for approval of the remedial action was added to Table 4.18-2 by the NRC in Amendment No. 41, issued April 11, 1979. The basis for this addition was not addressed in the associated NRC safety evaluation. In the presently considered license amendment, it is requested that the Action Required statement of Table 4.18-2 be changed to read "Special Report to NRC pursuant to 6.12.5.d" and that Note 2 for the table be revised to read "For C-3 results in one or both steam generators, plug or sleeve defective tubes and provide a Special Report to NRC pursuant to 6.12.5.d." A statement reading "The written Special Report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence" shall be added to TS 4.18.6 to clarify the required contents of a C-3 report. Finally, the reference to TS 6.12.5 is changed to TS 6.12.5.d for greater specificity.

The change, then, would eliminate the requirement for the NRC to approve the remedial action described in the C-3 report. This administrative action (for the NRC to approve the remedial action) is not contained in the previous

Standard TS "Steam Generator Tube Inspection" table (NUREG-0103, Rev. 4), nor is NRC approval of any part of the C-3 report discussed in the recently issued Babcock & Wilcox Standard TS (NUREG-1430, Rev. 0). Category C-3 reports will of course still be submitted prior to resumption of operation and will describe any corrective measures taken.

Since reference to remedial action is being deleted from Table 4.18-2, a description of the contents of a Category C-3 report is being added to TS 4.18.6. This added sentence is consistent with the language used in the Babcock & Wilcox Standard Technical Specifications. A typographical error introduced in Amendment No. 118 is being corrected by properly referencing TS 6.12.5.d rather than a nonexistent TS 6.12.4.5.

The proposed changes are appropriate, are consistent with current regulatory guidance and requirements, and are therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes recordkeeping, reporting, or administrative procedures or requirements, and changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 39049). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Bevan

Date: May 19, 1994