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ROBERT C. MECREDY
Vice President
Nuclear Operations

April 26, 2002

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Robert Clark
Project Directorate I
Washington, D.C. 20555

Subject: 10CFR50.46 Annual ECCS Report
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref. (a) Westinghouse Letter RGE-02-002, Subject: 10CFR50.46 Annual Notification and Reporting for 2001, dated March 1, 2002.

Dear Mr. Clark

In accordance with the requirements in 10CFR50.46 paragraph (a)(3)(ii), this annual ECCS report is hereby submitted.

Westinghouse, the provider of LOCA analysis for the R. E. Ginna Nuclear Power Plant, has provided RG&E with an update to the peak cladding temperature (PCT) margin for Ginna Station (Reference a).

The large break LOCA PCT has increased 13°F from the value previously reported due to an oxidation thickness input error (+15°F) and the removal of reconstituted fuel assemblies from Cycle 30 (-2°F). The new large break LOCA PCT is 2166°F and is summarized in Attachment 1 to this letter.

The small break LOCA PCT has not changed since the last report (Reference a). The small break LOCA PCT remains at 1346°F and is summarized in Attachment 1 to this letter.

Very truly yours,

Robert C. Mecredy

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Attachment

cc: Mr. Robert Clark (Mail Stop O-8-C2)
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
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U.S. NRC Ginna Senior Resident Inspector

ATTACHMENT I

LOCA PCT SUMMARY

APRIL 2002 UPDATE

ATTACHMENT I

LOCA PT SUMMARY

Large Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

	Evaluation Model:	UPI SECY	Fuel: OFA
	$F_Q = 2.45$	$F_{\Delta H} = 1.75$	SGTP = 15%

A.	Analysis of Record (5/95) (effective 6/96)	PCT = 2051° F
B.	1995 10CFR50.46 Model Assessments	
	1. Fixed heat transfer node assignment Error/Accumulator water injection error	ΔPCT = 48°F
C.	1996 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
D.	1997 10CFR50.46 Model Assessment	
	1. Accumulator Initial Water Volume Restart Data Transportation Error Plant Specific Analytical Reassessment of 1995 Model Assessments	ΔPCT = 58°F
	2. Accumulator Initial Water Vol. = 1125 ft^3	ΔPCT = -25°F
	3. 1-D Transition Boiling Heat Transfer Error	ΔPCT = -13°F
	4. Vessel Channel DX Error	ΔPCT = 18°F
	5. Input Consistency	ΔPCT = -41°F
E.	1998 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
F.	1999 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
G.	2000 10CFR50.46 Model Assessments	
	1. 100 psig IFBA RIP Modeling Error	ΔPCT = 2°F
	2. Channel Splitting Error	ΔPCT = 52°F
H.	2001 10CFR50.46 Model Assessments	
	1. Oxidation Thickness Input Error	ΔPCT = 15°F
I.	Ginna Evaluations	
	1. Service Water Temp. ≥30° (1997 evaluation; SEV-1090)	ΔPCT = 1°F
J.	Other Margin Allocations	
	1. None	ΔPCT = 0°F
	Licensing Basis	PCT = 2166°F

Revision Date: 4/2002

ATTACHMENT I

LOCA PT SUMMARY

Small Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model:	NOTRUMP	Fuel: OFA
$F_Q = 2.50$	$F_{\Delta H} = 1.75$	SGTP = 15%

A.	Analysis of Record (6/95) (effective 6/96)	Δ PCT = 1308°F
B.	1995 10CFR50.46 Model Assessments	
	1. Notrump Specific Enthalpy Error	Δ PCT = 20°F
	2. SALIBRARY Double Precision Errors	ΔPCT = -15°F
C.	1996 10CFR50.46 Model Assignments	
	1. SBLOCA Fuel Rod Initialization Error	ΔPCT = 10°F
D.	1997 10CFR50.46 Model Assessment	
	1. None	ΔPCT = 0°F
E.	1998 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
F.	1999 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
G.	2000 10CFR50.46 Model Assessments	
	1. NOTRUMP - Mixture Level Tracking/ Region Depletion Errors	ΔPCT = 13°F
H.	2001 10CFR50.46 Model Assessments	
	1. None	ΔPCT = 0°F
I.	Ginna Evaluations	
	1. Annular Axial Pellets (1997 evaluation; SEV-1108)	ΔPCT = 10°F
J.	Other Margin Allocations	
	1. None	ΔPCT = 0°F
	Licensing Basis	PCT = 1346°F

Revision Date: 4/2002