

May 10, 2002

Mr. A. Christopher Bakken III, Senior Vice President
and Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - REQUEST FOR
ADDITIONAL INFORMATION, "RESPONSES TO GENERIC LETTER (GL)
96-06 ASSURANCE OF EQUIPMENT OPERABILITY AND CONTAINMENT
INTEGRITY DURING DESIGN-BASIS ACCIDENT CONDITIONS,"
(TAC NOS. M96801 AND M96802)

Dear Mr. Bakken:

By letter dated August 31, 2001, Indiana Michigan Power Company (I&M) submitted its response to the Nuclear Regulatory Commission's (NRC's) request for additional information (RAI) dated June 6, 2001, concerning GL 96-06. The staff has reviewed your response and concluded that it does not provide technical information in sufficient detail to enable the staff to make an independent assessment regarding the equipment operability and containment integrity during design-basis accident conditions in terms of regulatory requirements and the protection of public health and safety. The NRC staff finds that the additional information is needed as identified in the enclosed RAI.

Draft questions were discussed with Mr. Bob Vasey et al., of your staff on April 29, 2002. The questions in the enclosed RAI are the same as the draft questions with the exception of minor modifications that were made to provide clarification. A mutually agreeable target date of July 02, 2002, for your response was established. The NRC staff will continue review of your responses to GL 96-06 when your response to the enclosed RAI is received.

If circumstances result in the need to revise the target date, please contact me at (301) 415-1345 at the earliest opportunity.

Sincerely,

/RA/

John F. Stang, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosure: RAI

cc w/encl: See next page

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Office of Nuclear Reactor Regulation

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REQUEST FOR ADDITIONAL INFORMATION FOR

DONALD C. COOK, UNITS 1 AND 2

SUBMITTAL C0801-05 RESPONSES TO GENERIC LETTER 96-06,

DATED AUGUST 31, 2001

1. Provide a simplified diagram of the fan cooler system showing the piping areas, elevations, and flow velocities at the time before the calculated waterhammer occurs. This will allow the NRC staff to perform its own calculation using the Joukowski equation.
2. Provide the margin of safety of the calculated loads and stresses in the piping system from a postulated waterhammer event to the maximum allowable loads and stresses.
3. Provide an evaluation showing whether the discharge flow orifice will drain and therefore be a potential waterhammer site.
4. Provide information concerning operator experience in restarting the fan cooler system in a voided condition.
5. Provide a comparison of predictions by the SYSFLOW code with predictions of the Joukowski equation for waterhammer pressure pulse.

ENCLOSURE

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

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