

February 6, 1995

Mr. Jerry W. Yelverton  
Vice President, Operations ANO  
Entergy Operations, Inc.  
Route 3 Box 137G  
Russellville, Arkansas 72801

SUBJECT: ISSUANCE OF AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE  
NO. DPR-51 - ARKANSAS NUCLEAR ONE, UNIT NO. 1 (TAC NO. M90739)

Dear Mr. Yelverton:

The Commission has issued the enclosed Amendment No. 175 to Facility Operating License No. DPR-51 for the Arkansas Nuclear One, Unit No. 1 (ANO-1). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 8, 1994.

The amendment revises the technical specification section that describes the frequency for performing the containment integrated leak rate tests.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:  
George Kalman, Senior Project Manager  
Project Directorate IV-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-313

Enclosures: 1. Amendment No. 175 to DPR-51  
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink, appearing to read "George Kalman".

George Kalman, Senior Project Manager  
Project Directorate IV-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-313

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2. Safety Evaluation

cc w/encls: See next page

Mr. Jerry W. Yelverton  
Entergy Operations, Inc.

Arkansas Nuclear One, Unit 1

cc:

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Pope County Courthouse  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY OPERATIONS INC.

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175  
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated November 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 175 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Kalman, Senior Project Manager  
Project Directorate IV-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the  
Technical Specifications

Date of Issuance: February 6, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE PAGE

80

INSERT PAGE

80

|        |                    |  |
|--------|--------------------|--|
| Where: | (L <sub>a</sub> )  | Design Basis Accident Leakage Rate at Pressure P <sub>a</sub>                                    |
|        | (L <sub>t</sub> )  | Maximum Allowable Test Leakage Rate at Reduced Test Pressure P <sub>t</sub> Under Test Condition |
|        | (L <sub>ao</sub> ) | Maximum allowable operational leakage rate at pressure P <sub>a</sub>                            |
|        | (L <sub>to</sub> ) | Maximum allowable leakage rate at pressure P <sub>t</sub>  |
|        | (L <sub>am</sub> ) | Initial Measured Leakage Rate at Pressure P <sub>a</sub>   |
|        | (L <sub>tm</sub> ) | Initial Measured Leakage Rate at Pressure P <sub>t</sub>   |
|        | (P <sub>a</sub> )  | Peak Test Pressure of 59 psig  |
|        | (P <sub>t</sub> )  | Reduced Test Pressure of 30 psig   |

#### 4.4.1.1.3 Conduct of Tests

- a. Leakage rate tests should not be started until essential temperature equilibrium has been attained. Containment test conditions should stabilize for a period of about four hours prior to the start of a leakage rate test.
- b. The leakage rate test period shall extend to 24 hours of retained internal pressure. If it can be demonstrated to the satisfaction of those responsible for the acceptance of the containment structure that the leakage rate can be accurately determined during a shorter test period, the agreed upon shorter period may be used.
- c. Test accuracy shall be verified by supplementary means, such as measuring the quantity of air required to return to the starting point or by imposing a known leak rate to demonstrate the validity of measurements.
- d. Closure of reactor building isolation valves for the purpose of the test shall be accomplished by the means provided for normal operation of the valves without preliminary exercise or adjustment.

#### 4.4.1.1.4 Frequency of Test

Testing frequencies shall be in accordance with 10CFR50, Appendix J, except as modified by approved exemptions.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 175 TO

FACILITY OPERATING LICENSE NO. DPR-51

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 1

DOCKET NO. 50-313

1.0 INTRODUCTION

By letter dated November 8, 1994, Entergy Operations, Inc. (the licensee), submitted a request to change the Arkansas Nuclear One, Unit No. 1 (ANO-1) technical specifications (TSs). The requested change revises the TS section that describes when containment integrated leak rate tests (ILRTs) should be performed. The existing specification paraphrases ILRT frequency requirements that are contained in 10 CFR Part 50, Appendix J. The proposed change references 10 CFR Part 50 Appendix J rather than paraphrasing the requirements. In paraphrasing Appendix J, the existing TS introduced a subtle difference in testing frequency requirements. The difference between the TS and Appendix J wording became apparent during preparations for refueling outages 12 and 13. By referencing Appendix J rather than paraphrasing the requirements, a source of confusion is removed from TS requirements related to when ILRTs should be performed.

2.0 EVALUATION

Governing regulations related to ILRT requirements are contained in 10 CFR Part 50, Appendix J. Section III.D of Appendix J addresses the ILRT (type A test) periodic retest schedule and states: "After the preoperational leakage rate tests, a set of three Type A tests shall be performed, at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections."

The ANO-1 TS, Section 4.4.1.1.4 paraphrases Appendix J and states: "After the initial preoperational leakage rate test, a set of three integrated leak rate tests shall be performed at approximately equal intervals during each 10-year service period. The test may coincide with the plant inservice inspection shutdown periods." The subtle difference in wording between Appendix J and the TS is in the second sentence. Appendix J states that the third test shall be conducted when the plant is shutdown for the 10-year plant inservice inspection. The TS states that the test may coincide with the plant inservice inspection shutdown periods.

The difference in wording has been innocuous until planning commenced for the 12th and 13th refuelings that are scheduled for 1995 and 1996. If Appendix J requirements were followed, an ILRT would be required during the 12th refueling outage to comply with the requirement for equally spaced ILRTs during each 10-year service period. A second ILRT would be required during the 13th refueling outage to comply with the requirement for the third test of the set to be performed when the plant is shutdown for the 10-year inservice inspection.

The licensee requested an exemption from the Appendix J schedular section to delete the requirement for two ILRT's in two successive years. The regulations never intended licensees to perform ILRTs on successive years. NRC is responding to the exemption request by separate correspondence.

In the course of reviewing ILRT related requirements, the licensee noted the wording discrepancy between Appendix J and the TS. The proposed amendment deletes the ILRT frequency description from the TS and in its place, references Appendix J requirements. The TS amendment also includes a statement that the ILRT frequency may be modified by exemption.

We have reviewed the proposed changes and conclude that they are administrative in nature and do not impact plant safety or operations. The proposed changes are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 502). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such

activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: George Kalman

Date: February 6, 1995