Docket No. 50-313

Mr. Neil S. Carns Vice President, Operations ANO Entergy Operations, Inc. Route 3 Box 137G Russellville, Arkansas 72801

Dear Mr. Carns:

SUBJECT: CORRECTION TO TECHNICAL SPECIFICATION PAGE 18a AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE NO. DPR-51 FOR ARKANSAS NUCLEAR ONE, UNIT NO. 1 (TAC NO. M77665)

On November 14, 1991, the Commission issued Amendment No. 154 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1. The amendment revised the reactor coolant system Technical Specifications (TS) pressure/temperature operating limits for the first 15 effective full power years, using the methodology of Regulatory Guide 1.99, Revision 2. The amendment also revised the low-temperature overprotection (LTOP) enable temperature.

Correction is being made by incorporating previous TS changes (Amendment No. 140) that were inadvertently omitted when Amendment No. 154 was issued. A corrected TS page 18a is enclosed for your use. We apologize for any inconvenience this omission may have caused you.

Sincerely,

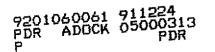
Original signed by Sheri R. Peterson 🕚

for Thomas W. Alexion, Project Manager Project Directorate IV-1 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stated	<u>DISTRIBUTION</u> Docket File	NRC & LPDRs	PD4-1 r/f
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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

December 24, 1991

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Sincerely,

Shin R. Paterson

Thomas W. Alexion, Project Manager Project Directorate IV-1 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page Mr. Neil S. Carns Entergy Operations, Inc.

cc:

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- 3.1.2.7 Prior to reaching fifteen effective full power years of operation, Figures 3.1.2-1, 3.1.2-2 and 3.1.2-3 shall be updated for the next service period in accordance with 10CFR50, Appendix G, Section V.B. The service period shall be of sufficient duration to permit the scheduled evaluation of a portion of the surveillance data scheduled in accordance with the latest revision of Topical Report BAW-1543(5). The highest predicted adjusted reference temperature of all the beltline region materials shall be used to determine the adjusted reference temperature at the end of the service period. The basis for this prediction shall be submitted for NRC staff review in accordance with Specification 3.1.2.8. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
- 3.1.2.8 The updated proposed technical specifications referred to in 3.1.2.7 shall be submitted for NRC review at least 90 days prior to the end of the service period. Appropriate additional NRC review time shall be allowed for proposed technical specifications submitted in accordance with 10 CFR Part 50, Appendix G. Section V.C.
- 3.1.2.9 With the exception of ASME Section XI testing and when the core flood tank is depressurized, during a plant cooldown the core flood tank discharge valves shall be closed and the circuit breakers for the motor operators opened before depressurizing the reactor coolant system below 600 psig.
- 3.1.2.10 With the exception of ASME Section XI testing, fill and vent of the reactor coolant system, emergency RCS makeup and to allow maintenance of the valves, when the reactor coolant temperature is less than 300°F, the High Pressure Injection motor operated valves shall be closed with their opening control circuits for the motor operators disabled.
- 3.1.2.11 The plant shall not be operated in a water solid condition when the RCS pressure boundary is intact except as allowed by Emergency Operating Procedures and during System Hydrotest.

Amendment No. 57, 53, 95, 138, 18a 149,154 ARKANSAS - UNIT 1

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