



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
June 21, 1989

Docket

Docket Nos. 50-313
and 50-368

Mr. T. Gene Campbell
Vice President, Nuclear Operations
Arkansas Power and Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Campbell:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 124 AND 98 TO FACILITY OPERATING
LICENSE NO. DPR-51 AND NPF-6 - ARKANSAS NUCLEAR ONE, UNITS 1
AND 2 (TAC NOS. 72984 AND 72985)

The Commission has issued the enclosed Amendment Nos. 124 and 98 to Facility Operating License Nos. DPR-51 and NPF-6 for the Arkansas Nuclear One, Unit 1 and 2 (ANO-1&2). These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 24, 1989.

The amendments modify the TSs for each unit to reflect planned changes in Arkansas Power & Light Company's organization for Arkansas Nuclear One. The amendments include both title changes and organizational restructuring. The amendments also change the designated members of the Plant Safety Committee.

A copy of the Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

C. Craig Harbuck, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Chester Poslusny, Jr., Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 124 to DPR-51
2. Amendment No. 98 to NPF-6
2. Safety Evaluation

cc w/enclosures:
See next page

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Operating License Nos. DPR-51 and NPF-6 for the Arkansas Nuclear One, Unit 1
and 2 (ANO-1&2). These amendments consist of changes to the Technical Specifici-
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The amendments modify the TSs for each unit to reflect planned changes in
Arkansas Power & Light Company's organization for Arkansas Nuclear One. The
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Office of Nuclear Reactor Regulation

/s/

Chester Poslusny, Jr., Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 124 to DPR-51
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2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION:

Docket File	BGrimes	PNoonan (2)	ACRS (10)
NRC PDR	TMeek (8)	CHarbuck (2)	GPA/PA
Local PDR	Wanda Jones	FHebdon	ARM/LFMB
PD4 Reading	JCalvo	OGC-Rockville	DHagan
EJordan	Plant File		

LTR NAME: ANO 1&2 AMEND TACS 72984/85

*SEE PREVIOUS CONCURRENCES:

PD4/LA*	PD4/PM*	PD4/PM	OGC-Rockville	PD4/D.H.
PNoonan	CPoslusny:sr	CHarbuck		FHebdon
06/01/89	06/01/89	06/21/89	06/10/89	06/15/89

06/21/89
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June 21, 1989

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The amendments modify TSs for each unit to reflect planned changes in Arkansas Power & Light Company's (AP&L's) organization for Arkansas Nuclear One (ANO). The amendments include both title changes and organizational restructuring. The amendments also change the designated members of the Plant Safety Committee.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

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C. Craig Harbuck, Project Manager
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

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Project Directorate - IV
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LTR NAME: ANO 1&2 AMEND TACS 72984/85			

PD4/LA *[Signature]*
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PD4/PM *[Signature]*
CHarbuck
06/2/89

OGC-Rockville
06/ /89

PD4/D
FHebdon
06/ /89

Mr. T. Gene Campbell
Arkansas Power & Light Company

Arkansas Nuclear One
Unit Nos. 1 and 2

cc:

Mr. Dan R. Howard, Manager
Licensing
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
Combustion Engineering, Inc.
12300 Twinbrook Parkway, Suite 330
Rockville, Maryland 20852

Mr. James M. Levine, Executive Director
Site Nuclear Operations
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Honorable Joe W. Phillips
County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Nicholas S. Reynolds, Esq.
Bishop, Cook, Purcell
& Reynolds
1400 L Street, N.W.
Washington, D.C. 20005-3502

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Office of Executive Director for
Operations
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
1 Nuclear Plant Road
Russellville, Arkansas 72801

Ms. Greta Dicus, Director
Division of Environmental Health
Protection
Arkansas Department of Health
4815 West Markam Street
Little Rock, Arkansas 72201

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220
1700 Rockville Pike, Suite 525
Rockville, Maryland 20852



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated April 24, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

2. Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 124, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Frederick J. Hebdon, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 21, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 124

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Revise the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES

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INSERT PAGES

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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The Vice President, Nuclear shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the QA Manual Operations.
- b. The Director, Nuclear Operations shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President, Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

FACILITY STAFF

- 6.2.2 The Operations Superintendent and Shift Supervisor shall hold a senior reactor operator license. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

6.2.2.1 Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety-related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter 82-12).

6.3. FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for (1) the designated radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager, Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager, Training and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 Plant Safety Committee (PSC) Function

6.5.1.1 The Plant Safety Committee shall function to advise the Director, Nuclear Operation on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the:

Chairman:	Manager, Nuclear Standards
Member:	Plant Manager, ANO-1
Member:	Plant Manager, ANO-2
Member:	Designated radiation protection manager
Member:	Manager, Plant Engineering
Member:	Superintendent, Reactor Engineering
Member:	Superintendent, Quality Assurance
Member:	Superintendent, Operations Assessment
Member:	Manager, Training

The Director, Nuclear Operations should designate in writing the Alternate Chairman in the absence of the Chairman.

ALTERNATES

- 6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

- 6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

- 6.5.1.5 The minimum quorum of the PSC necessary for the performance of the PSC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The Plant Safety Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification 6.8 and revisions thereto, 2) any other proposed procedures or revisions thereto as determined by the Director, Nuclear Operations to affect nuclear safety.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Appendix "A" Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Director, Nuclear Operations and to the Chairman of the Safety Review Committee.
 - f. Review of REPORTABLE EVENTS.

- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Director, Nuclear Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Director, Nuclear Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Director, Nuclear Operations and the Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and the Process Control Program.

AUTHORITY

6.5.1.7. The Plant Safety Committee shall:

- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President, Nuclear and the Safety Review Committee of disagreement between the PSC and the Director, Nuclear Operations; however, the Director, Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Director, Nuclear Operations and Chairman of the Safety Review Committee.

6.5.2 Safety Review Committee (SRC)

FUNCTION

- 6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:
 - a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry

- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

- 6.5.2.2 The Safety Review Committee shall be composed of a Chairman and eight members which collectively have the experience and competence required by ANSI/ANS-3.1-1981 to review problems in the areas specified in Section 6.5.2.1, a-h.

The Vice President, Nuclear shall designate, in writing, the Chairman and all SRC members.

The Chairman shall designate, in writing, the alternate Chairman in the absence of the SRC Chairman.

ALTERNATES

- 6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

- 6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

- 6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

- 6.5.2.6 The minimum quorum of the SRC necessary for the performance of the SRC review and audit functions of these technical specifications shall consist of the Chairman or his designated alternate and at least 4 SRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of 10CFR50 Appendix "B", at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of unit operation considered appropriate by the SRC or the Vice President, Nuclear.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified off-site licensee personnel or an outside fire protection firm.
- j. The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.

AUTHORITY

- 6.5.2.9 The SRC shall report to and advise the Vice President, Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of SRC activities shall be prepared, approved and distributed as indicated below:
 - a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear within 14 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above shall be forwarded to the Vice President, Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.1 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

6.6.2 The following actions shall be taken for REPORTABLE EVENTS:

- a. A report shall be submitted to the Commission pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSC, and the results of this review shall be submitted to the SRC and the Vice President, Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least hot shutdown within one hour.
- b. The Nuclear Regulatory Commission shall be notified pursuant to 10 CFR 50.72 and a report submitted pursuant to the requirements of 10 CFR 50.36 and Specification 6.6.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. New and spent fuel storage.
- h. Offsite Dose Calculation Manual and Process Control Program implementation at the site.
- i. Post accident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and the containment atmosphere).

- 6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the Director, Nuclear Operations; Unit 1 Plant Manager; or responsible General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.
- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.
 - c. The change is documented, and approved by the Director, Nuclear Operations; Unit 1 Plant Manager or responsible General Manager, within 14 days of implementation.

6.11.2 The requirements of 6.11.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and access to these areas shall be maintained under the administrative control of the Shift Supervisor on duty and/or the designated radiation protection manager.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER AND LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated April 24, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

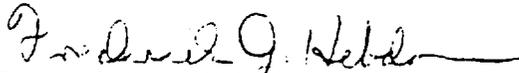
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 98, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Frederick J. Hebdon, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 21, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 98

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Revise the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES

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INSERT PAGES

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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President, Nuclear shall be responsible for overall facility operations and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the QA Manual Operations.
- b. The Director, Nuclear Operations shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President, Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 FACILITY STAFF

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.

- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
- g. Administrative control shall be established to limit the amount of overtime worked by plant staff performing safety-related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).
- h. The Operations Superintendent and Shift Supervisor shall hold a senior reactor operator license.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the designated radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a bachelors degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager, Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager, Training and shall meet or exceed the requirements of Section 27 of the NFPA Code - 1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY COMMITTEE (PSC)

FUNCTION

6.5.1.1 The Plant Safety Committee shall function to advise the Director, Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the:

Chairman:	Manager, Nuclear Standards
Member:	Plant Manager, ANO-1
Member:	Plant Manager, ANO-2
Member:	Designated radiation protection manager
Member:	Manager, Plant Engineering
Member:	Superintendent, Reactor Engineering
Member:	Superintendent, Quality Assurance
Member:	Superintendent, Operations Assessment
Member:	Manager, Training
Member:	Nuclear Software Expert*

The Director, Nuclear Operations shall designate in writing the Alternate Chairman in the absence of the PSC Chairman.

*See Page 6-5a

ADMINISTRATIVE CONTROLS

*If one of the above members of the Plant Safety Committee meets the qualification requirements for this position, the requirement to have this member is satisfied. This membership may be filled by two appropriately qualified individuals who shall ballot with a single combined vote. Generic qualifications for this membership shall be as follows:

One Individual

The Nuclear Software Expert shall have as a minimum a Bachelor's degree in Science or Engineering, Nuclear preferred (in accordance with ANSI N18.1). In addition, he shall have a minimum of four years of technical experience, of which a minimum of two years shall be in Nuclear Engineering and a minimum of two years shall be in Software Engineering. (Software Engineering is that branch of science and technology which deals with the design and use of software. Software Engineering is a discipline directed to the production and modification of computer programs that are correct, efficient, flexible, maintainable, and understandable, in reasonable time spans, and at reasonable costs). The two years of technical experience in Software Engineering may be general software experience not necessarily related to the software of the Core Protection Calculator System. One of these two years of experience shall be with certified computer programs.

Two Individuals

One of the individuals shall meet the requirements of the Nuclear Engineering portion of the above. The second individual shall have a Bachelor of Science degree (digital computer speciality) and meet the Software Engineering requirements of the above.

The membership (the Nuclear Software Expert or the Digital Computer Specialist) shall be knowledgeable of the Core Protection Calculator System with regard to:

- a. The software modules, their interactions with each other and with the data base.
- b. The relationship between operator's module inputs and the trip variables.
- c. The relationship between sensor input signals and the trip variable.
- d. The design basis of the Core Protection Calculator System.
- e. The approved software change procedure and documentation requirements of a software change.
- f. The security of the computer memory and access procedures to the memory.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PSC necessary for the performance of the PSC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Director, Nuclear Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendation to prevent recurrence to the Director, Nuclear Operations and to the Chairman of the Safety Review Committee.

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- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses, and reports thereon as requested by the Director, Nuclear Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Director, Nuclear Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Director, Nuclear Operations and Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and Process Control Program.

AUTHORITY

- 6.5.1.7 The Plant Safety Committee shall:
- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the Vice President, Nuclear and the Safety Review Committee of disagreement between the PSC and the Director, Nuclear Operations; however, the Director, Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Director, Nuclear Operations and Chairman of the Safety Review Committee.

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6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The Safety Review Committee shall be composed of a Chairman and eight members which collectively have the experience and competence required by ANSI/ANS-3.1-1981 to review problems in the areas specified in Section 6.5.2.1, a-h.

The Vice President, Nuclear shall designate, in writing, the Chairman and all SRC members.

The Chairman shall designate, in writing, the alternate Chairman in the absence of the SRC Chairman.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training, and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structure, systems, or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix "B", at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of unit operation considered appropriate by the SRC or the Vice President, Nuclear.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
- k. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Vice President, Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.10 Records of SRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. A report shall be submitted to the Commission pursuant to the requirements of Section 50.73 to 10CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSC and, the results of this review shall be submitted to the SRC and the Vice President, Nuclear.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Vice President, Nuclear and the SRC shall be notified within 24 hours.
- c. The Nuclear Regulatory Commission shall be notified pursuant to 10CFR50.72 and a report submitted pursuant to the requirements of 10CFR50.36 and Specification 6.6.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. Modification of Core Protection Calculator (CPC) Addressable Constants. These procedures should include provisions to assure that sufficient margin is maintained in CPC Type I addressable constants to avoid excessive operator interaction with the CPCs during reactor operation.

NOTE: Modifications to the CPC software (including changes of algorithms and fuel cycle specific data) shall be performed in accordance with the most recent version of "CPC Protection Algorithm Software Change Procedure," CEN-39(A)-P that has been determined to be applicable to the facility. Additions or deletions to CPC addressable constants or changes to addressable constant software limit values shall not be implemented without prior NRC approval.

- h. New and spent fuel storage.
- i. ODCM and PCP implementation.
- j. Postaccident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluent, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, the changes thereto, shall be reviewed by the PSC and approved by the Director, Nuclear Operations; Unit 2 Plant Manager or responsible General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

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6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PSC and approved by the Director, Nuclear Operations; Unit 2 Plant Manager or responsible General Manager within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Administrator of the Regional Office unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

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- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10CFR50.59.
- k. Records of meetings of the PSC and the SRC.
- l. Records of changes to the Core Protection Calculator System (CPCS) SOFTWARE. Changes to the CPCS SOFTWARE shall be made in accordance with methods approved by the NRC. These records shall include the following:
 - 1. Purpose of change.
 - 2. Detailed description of change including algorithms, changes to the assembly listings, checksums and disk identification numbers.
 - 3. Summary of validation test results.
- m. Records of Environmental Qualification which are covered under the provisions of paragraph 6.12.
- n. Records of the service lives of the seals of all hydraulic snubbers required by Specification 3.7.8, including the date at which the service life commences and associated installation and maintenance records.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 ENVIRONMENTAL QUALIFICATION

6.12.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License NPF-6 dated October 24, 1980.

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6.12.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area (as defined in 20.202(b)(3) of 10 CFR 20) in which the intensity of radiation is 1000 mrem/hr or less shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring the issuance of a radiation work permit. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a present integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified in the radiation work permit.

6.13.2 The requirements of 6.13.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and access to these areas shall be maintained under the administrative control of the Shift Supervisor on duty and/or the designated radiation protection manager.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 124 AND 98

FACILITY OPERATING LICENSE NOS. DPR-51 AND NPF-6

ARKANSAS POWER AND LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT NOS. 1 AND 2

DOCKET NOS. 50-313 AND 50-368

INTRODUCTION

By letter dated April 24, 1989, Arkansas Power and Light Company (AP&L or the licensee) requested amendments to the Technical Specifications (TSs) appended to Facility Operating License Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units 1 and 2 (ANO-1&2). The proposed amendments would change the TS for each unit to reflect planned changes in Arkansas Power and Light Company's organization for ANO. The amendments include both title changes and organizational restructuring. The amendments also change the designated members of the Plant Safety Committee.

EVALUATION

The specific organizational changes proposed are: (1) The position of Executive Director, Nuclear Operations is deleted, and this position's duties and responsibilities are divided between the corporate Vice President, Nuclear (formerly the Vice President, Nuclear Operations), and the Director, Nuclear Operations (a new position); (2) The Vice President, Nuclear will have the following managers reporting to him directly: Director, Nuclear Operations, General Manager, Engineering (formerly General Manager, Design Engineering); General Manager, Technical Support and Assessment (formerly General Manager, Plant Support); General Manager, Nuclear Quality (unchanged); and General Manager, Nuclear Support (a new title); (3) The Director, Nuclear Operations will be the onsite manager responsible for those activities necessary for safe operation and maintenance of the units. The following managers will report directly to the Director, Nuclear Operations: Unit 1 Plant Manager; Unit 2 Plant Manager; Manager Central Operations; and Manager, Nuclear Standards. Each Unit Plant Manager will be responsible for separate operations, maintenance, and outage management organizations dedicated to each unit. The Manager, Central Operations (new position) will be responsible for radiation protection, chemistry, support services, work planning and scheduling, and maintenance engineering. There will be a separate maintenance engineering group for each unit under a Superintendent of Maintenance Engineering. The Manager, Nuclear Standards (new position) will be responsible for maintaining procedures and administrative controls for both units. (4) The General Manager, Engineering will be responsible for design engineering, the onsite engineering organization, plant modifications, and activities associated with design and configuration control of the units, and

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providing engineering support as needed to operations, maintenance, licensing, and plant assessment; (5) The General Manager, Technical Support and Assessment will be responsible for Licensing, physical security, ANO training, procurement, operational events assessment, plant projects activities, nonconformance report processing, and the functioning of the Plant Safety Committee; (6) The General Manager, Nuclear Support will be responsible for the financial, administrative, and support staffs formerly under direct control of the Vice President, Nuclear; and (7) The duties and responsibilities of the General Manager, Nuclear Quality are unchanged.

The foregoing description of the new organization differs slightly from the description provided in the Federal Register Notice (54 FR 21297, dated May 17, 1989), and reflects lower tier organizational changes which have no impact on the previous no significant hazards determination.

The duties and responsibilities of the Plant Safety Committee (PSC) are unchanged. However, due to the extensive restructuring and numerous position title changes, the composition description of the PSC membership is changed. The Manager, Nuclear Standards is designated as the PSC Chairman (formerly designated as General Manager, Plant Support). Changes in positions or titles of positions designated as voting members of the PSC are both Unit Plant Managers (formerly the Operations Manager), a designated radiation protection manager (formerly the Health Physics Superintendent), the Manager, Plant Engineering (formerly the Engineering Manager), and Superintendent, Operations Assessment (new member). The other positions designated as voting PSC members are unchanged. These are the Superintendents of Reactor Engineering and Quality Assurance, the Training Manager, and for Unit 2, a Nuclear Software Expert. The Plant Licensing Supervisor position is deleted from the PSC. The total number of persons on the PSC is unchanged at 9 members. Also, the requirements regarding quorum and meeting frequency are unchanged.

The staff has evaluated the proposed changes to the Technical Specifications for Arkansas Nuclear One, Units 1 and 2 associated with the planned licensee reorganization. Although personnel assignments are revised, and position titles and reporting requirements are changed, the commitments to certain minimum qualifications and organizational reporting requirements are unchanged. It appears that the planned organizational restructuring is designed to improve accountability, promote more efficient operation, and should result in an overall improvement in safety at ANO.

The designation of the Manager, Nuclear Standards as the PSC Chairman is appropriate because he is responsible for establishing and maintaining quality in plant operational and administrative procedures, and will have administrative control of the PSC. A significant function of the PSC is the review of procedures that affect nuclear safety. The designation of the Operations Assessment Superintendent as a PSC member appears appropriate because of the PSC responsibility to review reportable events. These proposed changes are administrative in nature and do not decrease the level of management controls presently in the Technical Specifications. Overall, the staff finds that the proposed changes are

acceptable and meet the applicable acceptance criteria of Section 13.1.2 "OPERATING ORGANIZATION," of the Standard Review Plan. It appears that the proposed reorganization will enhance the effectiveness of the site management. The establishment of separate operations, maintenance and maintenance engineering organizations dedicated to each unit should be beneficial due to the significant design differences between the units.

ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 21, 1989

Principal Contributors: C. Harbuck
C. Poslusny