

October 29, 1984

DMB 016

Docket Nos. 50-313  
and 50-368

Mr. John M. Griffin, Senior Vice President  
of Energy Supply  
Arkansas Power and Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

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NRC PDR DBrinkman  
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Dear Mr. Griffin:

The Commission has issued the enclosed Amendments Nos. 85 and 57 to Facility Operating License Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in partial response to your application dated March 16, 1984.

These amendments revise the TSs to require administrative controls for shift overtime for plant staff who perform safety-related functions.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next monthly Federal Register Notice.

Sincerely,

"ORIGINAL SIGNED BY  
JOHN F. STOLZ"

John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

"ORIGINAL SIGNED BY:"

James R. Miller, Chief  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

1. Amendment No. 85 to DPR-51
2. Amendment No. 57 to NPF-6
3. Safety Evaluation

cc w/enclosures:  
See next page

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cc w/enclosure(s):

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Russellville, Arkansas 72801

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Honorable Ermil Grant  
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Health Protection  
Department of Health  
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ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85  
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated March 16, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

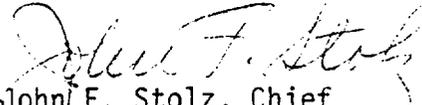
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 29, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

117

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Insert

117

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## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

- 6.1.1 The ANO General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

#### OFFSITE

- 6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2.-1.

#### FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2.-2. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.-1.
- 6.2.2.1 Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).

### 6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for (1) the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for fire protection training shall be maintained and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 with the exception of frequency of training which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 Plant Safety Committee (PSC) Function

6.5.1.1 The Plant Safety Committee shall function to advise the ANO General Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the: (See Page 121)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57  
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated March 16, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
James R. Miller, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 29, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is provided to maintain document completeness.

Remove Page

6-1

Insert Page

6-1

## 6.0 ADMINISTRATIVE CONTROLS

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### 6.1 RESPONSIBILITY

6.1.1 The ANO General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

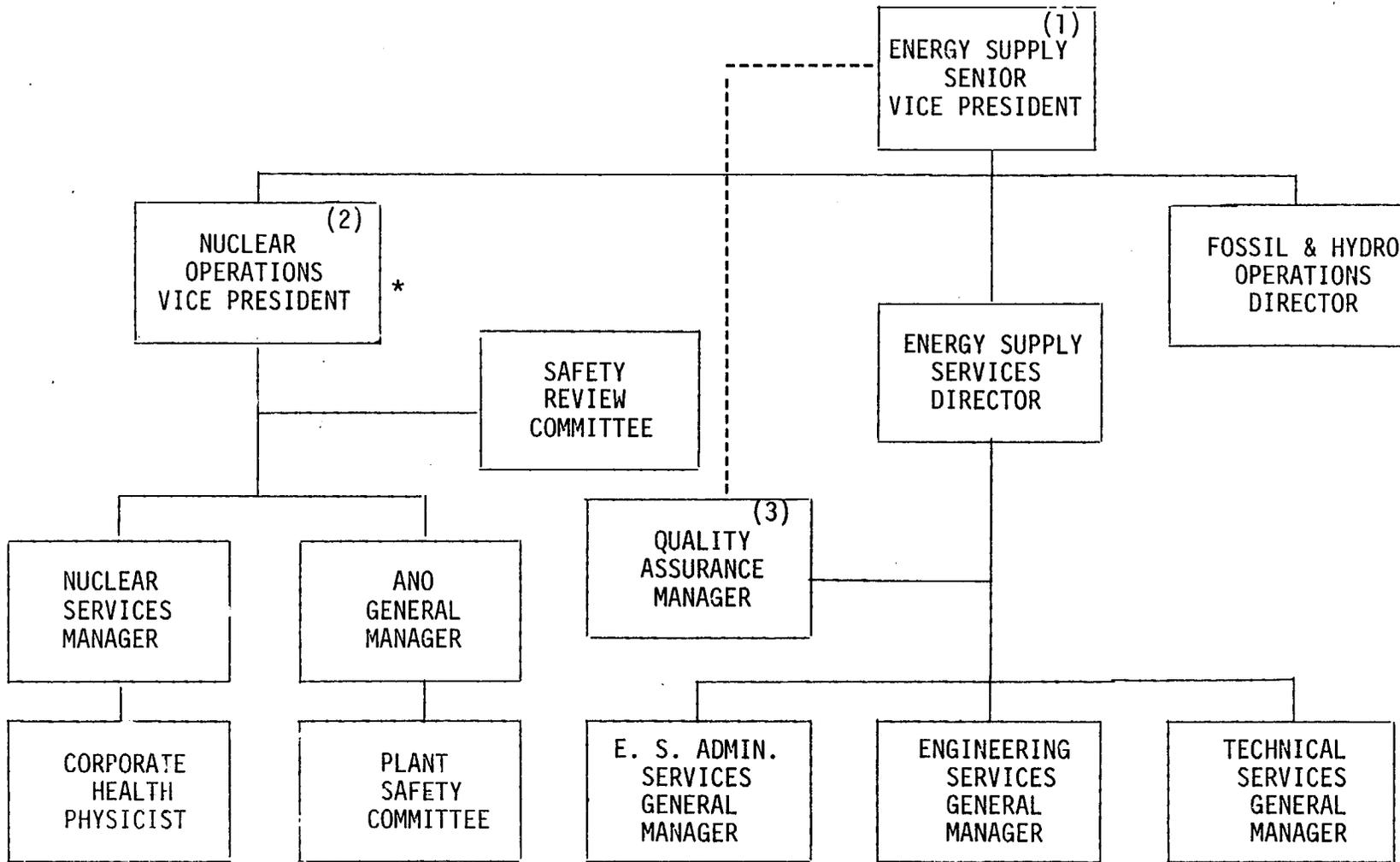
#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

#### FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
- g. Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).



\* Corporate Responsibility for Fire Protection Program

- (1) The Energy Supply Senior Vice President is on a temporary leave of absence from AP&L, serving as the Senior Vice President of Nuclear Operations for Louisiana Power and Light Company. During this absence, Mr. J. D. Phillips, Senior Vice President will assume senior management of the Energy Supply Department reporting directly to the President and Chief Executive Officer.
- (2) In the absence of the Energy Supply Senior Vice President, the Nuclear Operations Vice President will report directly to the President and Chief Executive Officer.
- (3) The Quality Assurance Manager will report (dashed line function) to the Vice President, Nuclear Operations during the absence of the Energy Supply Senior Vice President.

Figure 6.2-1 Management Organization Chart



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENTS NOS. 85 AND 57 TO FACILITY OPERATING LICENSES

NOS. DPR-51 AND NPF-6

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNITS NOS. 1 AND 2

DOCKETS NOS. 50-313 AND 50-368

Introduction

By letter dated March 16, 1984, Arkansas Power and Light Company (AP&L or the licensee) requested amendments to the Technical Specifications (TSs), Appendix A, appended to Facility Operating Licenses DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1 & 2 (ANO-1&2). The proposed amendments would change the TSs for both ANO-1&2 by requiring administrative controls for shift overtime for all plant staff who perform safety-related functions. The proposed amendments would also provide other changes which will be considered in a separate action.

Background

Following the accident at Three Mile Island, Unit 2, the NRC staff developed NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident", to provide a comprehensive and integrated plan to improve safety at nuclear power reactors. Specific NUREG-0660 items that were approved by the Commission for implementation were issued in "Clarification of TMI Action Plan Requirements", NUREG-0737. NUREG-0737 and its Supplement 1 (Generic Letter (GL) 82-33, "Requirements for Emergency Response Capability") specified that amended TSs would be required in order to implement several of the items. Subsequently, on September 20, 1982, NRC issued GL 82-16, "NUREG-0737 Technical Specifications", requesting all pressurized water reactor licensees to (a) review their existing TSs against the GL 82-16 guidance and (b) submit proposed TSs for those items that deviated from the GL 82-16 guidance.

On December 28, 1982 and March 16, 1984, AP&L responded to GL 82-16. The latter submittal to the NRC proposed one change to the ANO-1&2 TSs related to GL 82-16 and several changes related to a subsequent generic letter, GL 83-37. Only the proposed changes relating to GL 82-16 are considered in this action. Proposed changes relating to GL 83-37 will be considered in a separate action. The proposed changes considered herein are responsive to NUREG-0737 Item I.A.1.3, "Limit Overtime".

Discussion and Evaluation

On June 15, 1982, the staff issued GL 82-12, which contained a revised version of the Commission's Policy Statement on nuclear power plant staff working hours. Subsequently, by GL 82-16 dated September 20, 1982, AP&L was requested

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to revise the administrative controls section of the ANO-1&2 TSs to adhere to the policy statement guidelines. The objective of the controls is to assure that, to the extent practicable, personnel are not assigned to shift duties while in a fatigued condition that could significantly reduce their mental alertness or their decision making capability. The controls are to be applicable to all plant staff who perform safety-related functions (e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel).

Subsequently, AP&L responded with the proposed TSs for ANO-1&2 that incorporate administrative procedures for controlling shift overtime. The proposed TSs generally satisfy the intent and requirements of GL 82-12. Therefore, we have determined that the proposed TSs are acceptable and that AP&L has fulfilled the necessary requisites of NUREG-0737 Item I.A.1.3.

### Environmental Consideration

These amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 29, 1984

Principal Contributors: M. Nevius  
D. Powers  
G. Vissing