April 24, 1985

DMB 016

Docket Nos. 50-313
and 50-368

Mr. John M. Griffin, Senior Vice President
of Energy Supply
Arkansas Power and Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Griffin:

On January 31, 1985, the Commission issued Amendment Nos. 94 and 63 to Facility Operating License Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Unit Nos. 1 and 2. There were several typographical errors on the revised Technical Specifications (TS) pages 45d and 71 for Unit 1 TS and page 6-13 for Unit 2 TS.

Please accept our apologies for any inconvenience these errors may have caused.

Sincerely,

"ORIGINAL SIGNED BY
JOHN F. STOLZ"

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

"ORIGINAL SIGNED BY:"

James R. Miller, Chief
Operating Reactors Brandch #3
Division of Licensing

Enclosures:
1. Pages 45d and 71 to Amendment No. 94
2. Page 6-13 to Amendment No. 63

cc w/enclosures:
See next page

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cc w/enclosure(s):

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Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Regional Radiation Representative
EPA Region VI
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Dallas, Texas 75270

Mr. Robert Martin, Regional Administrator
U. S. Nuclear Regulatory Commission, Region IV
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Arlington, Texas 76011

Mr. Frank Wilson
Director, Division of Environmental
Health Protection
Department of Health
Arkansas Department of Health
4815 West Markham Street
Little Rock, Arkansas 72201

Amendment No. 30, 60, 69, 82, 87, 94, 45d

TABLE 3.5.1-1 (Cont'd)

OTHER SAFETY RELATED SYSTEMS
(Cont'd)

		1	2	3	4	5
		No. of channels	No. of channels for system trip	Min. operable channels	Min. degree of redundancy	Operator action if conditions of column 3 or 4 cannot be met.
2.	Pressurizer level channels	2	N/A	2	1	Note 10
3.	Emergency Feedwater Flow channels	2/S.G.	N/A	1	0	Note 10
4.	RCS subcooling margin monitors	2	N/A	1	0	Note 10
5.	Electromatic relief valve flow monitor	2	N/A	1	0	Note 11
6.	Electromatic relief block valve position indicator	1	N/A	1	0	Note 12
7.	Pressurizer code safety valve flow monitors	2/valve	N/A	1/valve	0	Note 10
8.	Degraded Voltage Monitoring					
	a. 4.16KV Emergency Bus Undervoltage	2/Bus	1/Bus	2/Bus	0	Note 14
	b. 460V Emergency Bus Undervoltage	*1/Bus	1/Bus	1/Bus	0	Notes 13, 14
9.	Chlorine Detection Systems	2	1	2	0	Notes 17, 18
10.	Containment High Range Radiation Monitoring	2	N/A	2	0	Note 20
11.	Containment Pressure - High Range	2	N/A	2	0	Note 21
12.	Containment Water Level - Wide Range	2	N/A	2	0	Note 21

*Two undervoltage relays per bus are used with a coincident trip logic (2-out-of-2)

Table 4.1-1 (Cont'd)

	Channel Description	Check	Test	Calibrate	Remarks
20.	Reactor Building Spray System System Logic Channels	NA	M(1)	NA	(1) Including RB spray pump, spray valve, and chem. add. valve logic channels.
21.	Reactor Building Spray System Analog Channels				
	a. Reactor Building Pressure Channels	NA	M	R	
22.	Pressurizer Temperature Channels	S	NA	R	
23.	Control Rod Absolute Position	S(1)	NA	R	(1) Compare with Relative Position Indicator.
24.	Control Rod Relative Position	S(1)	NA	R	(1) Check with Absolute Position Indicator
25.	Core Flooding Tanks				
	a. Pressure Channels	S	NA	R	
	b. Level Channels	S	NA	R	
26.	Pressurizer Level Channels	S	NA	R	
27.	Makeup Tank Level Channels	D	NA	R	
28.	Radiation Monitoring Systems other than containment high range monitors (item 57)	W	M(1)	Q(2)	(1) Check functioning of self-checking feature on each detector. (2) R for those detectors inaccessible during normal operation.
29.	High and Low Pressure Injection Systems: Flow Channels	NA	NA	R	

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The unit shall be placed in at least HOT STANDBY within one hour.

b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations and to the SRC within 24 hours.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Vice-President, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

f. Fire Protection Program implementation.

g. Modification of Core Protection Calculator (CPC) Addressable Constants

NOTE: Modification to the CPC addressable constants based on information obtained through the Plant Computer - CPC data link shall not be made without prior approval of the Plant Safety Committee.

h. New and spent fuel storage.

i. ODCM and PCP implementation.

j. Postaccident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluent, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the ANO General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

c. The change is documented, reviewed by the PSC and approved by the ANO General Manager within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Administrator of the Regional Office unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.