



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 10, 1986

Docket Nos.: 50-313
and 50-368

Mr. T. Gene Campbell
Vice President, Nuclear Operations
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Campbell:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 99 AND 73 TO FACILITY OPERATING
LICENSE NOS. DPR-51 AND NPF-6 FOR ARKANSAS NUCLEAR ONE,
UNIT NOS. 1 AND 2 (ANO-1 & 2)

The Commission has issued the enclosed Amendment Nos. 99 and 73 to Facility Operating License Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your applications dated December 20, 1985, as supplemented by letter dated February 7, 1986.

The amendments revise the Technical Specifications to reflect a reorganization of the Nuclear Operations Department of Arkansas Power & Light Company. In addition, the amendments revise the Technical Specifications pertaining to the responsibility and membership of the Safety Review Committee (SRC). The amendments also make editorial changes and correct a typographical error.

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PDR ADDCK 05000313
P PDR

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,



Guy S. Vissing, Project Manager
PWR Project Directorate No. 6
Division of PWR Licensing-B



Robert S. Lee, Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 99 to DPR-51
1. Amendment No. 73 to NPF-6
3. Safety Evaluation

cc: w/enclosures:
See next page

Mr. T. Gene Campbell
Arkansas Power & Light Company

Arkansas Nuclear One
Units 1 and 2

cc:

Mr. J. Ted Enos, Manager
Nuclear Engineering and Licensing
Arkansas Power and Light Company
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Little Rock, Arkansas 72203

Mr. Charlie B. Brinkman, Manager
Washington Nuclear Operations
C-E Power Systems
7910 Woodmont Avenue
Suite 1310
Bethesda, Maryland 20814

Mr. James M. Levine, Director
Site Nuclear Operations
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Honorable William Abernathy
County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

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Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Office of Executive Director for
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Mr. Frank Wilson, Director
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Arkansas Department of Health
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Little Rock, Arkansas 72201

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220
7910 Woodmont Avenue
Bethesda, Maryland 20814

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

s/

Guy S. Vissing, Project Manager
PWR Project Directorate No. 6
Division of PWR Licensing-B

s/

Robert S. Lee, Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosures:

- 1. Amendment No. 99 to DPR-51
- 1. Amendment No. 73 to NPF-6
- 3. Safety Evaluation

cc: w/enclosures:
See next page

PBD7
R Lee
3/5/86

PBD7
J Lee
5/5/86

PBD6
G Vissing
4/14/86

PBD6
R Ingram
5/8/86

PBD6
C McCracken
4/1/86
5/19

OELD
S J
4/1/86

D: PBD6
J Stolz
5/20/86

D: PBD7
G W Knighton
6/3/86

no formal request

- JUN 10 1986

ISSUANCE OF AMENDMENTS NOS. 99 AND 73 TO FACILITY OPERATING LICENSE
NOS. DPR-51 AND NPF-6 FOR ARKANSAS NUCLEAR ONE, UNIT NOS. 1 and 2

DISTRIBUTION

Docket File 50-313/368

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F. Miraglia/Schroeder



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated December 20, 1985, as supplemented February 7, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

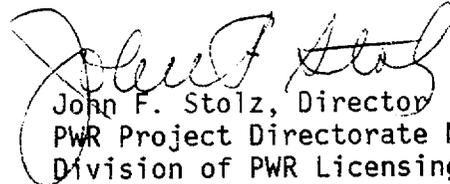
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
PWR Project Directorate No. 6
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 99FACILITY OPERATING LICENSE NO. DPR-51DOCKET NO. 50-313

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
117	117
117a	117a
119	119
120	120
121	121
-	121a
122	122
123	123
124	124
125	125
126	126
127	127

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The Director, Site Nuclear Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

- 6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

- 6.2.2.1 Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for (1) the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 Plant Safety Committee (PSC) Function

6.5.1.1 The Plant Safety Committee shall function to advise the Director, Site Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the: (See Page 121)

ARKANSAS POWER & LIGHT COMPANY

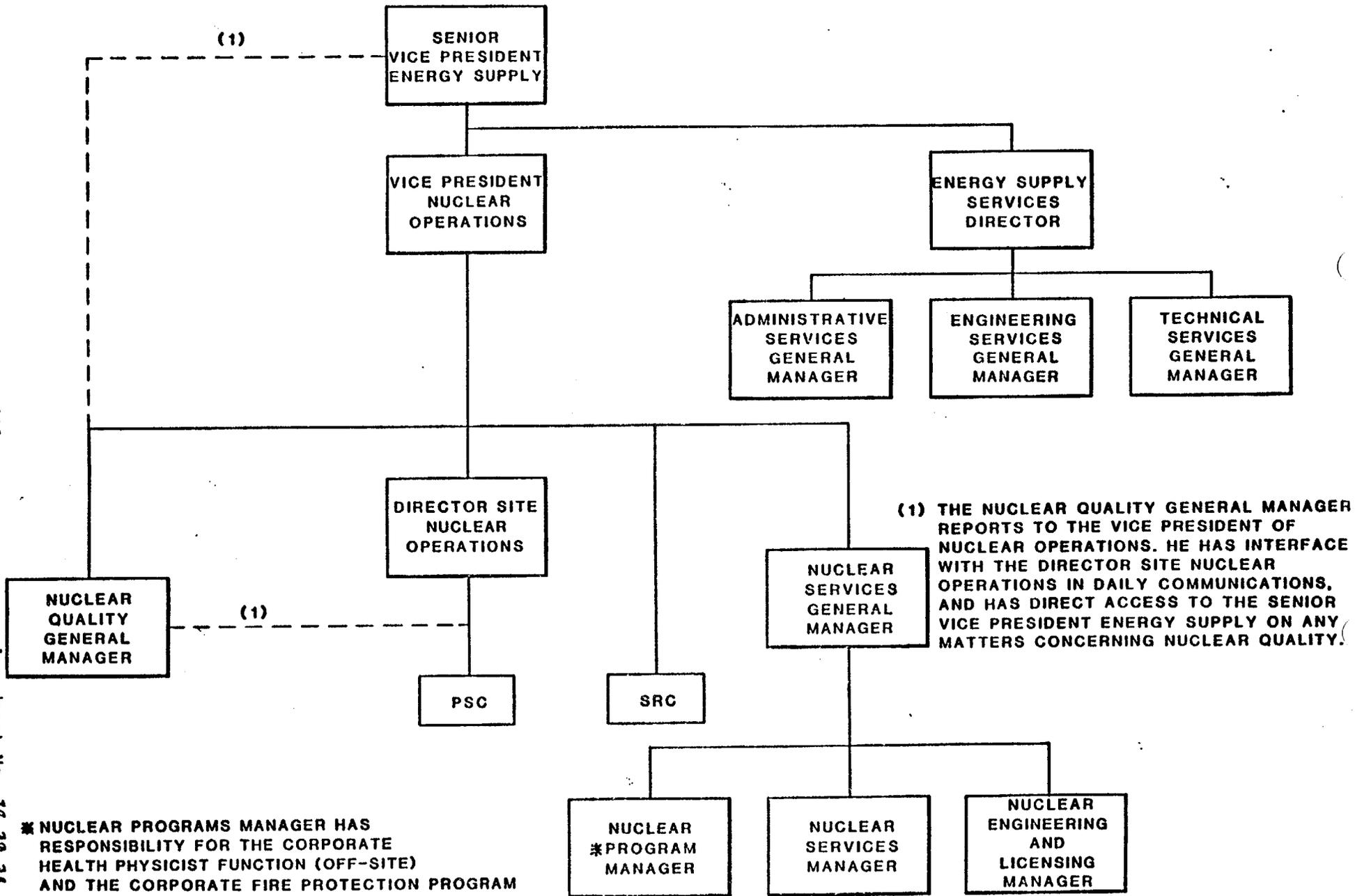


FIGURE 6.2-1 MANAGEMENT ORGANIZATION CHART

ARKANSAS POWER & LIGHT COMPANY ARKANSAS NUCLEAR ONE

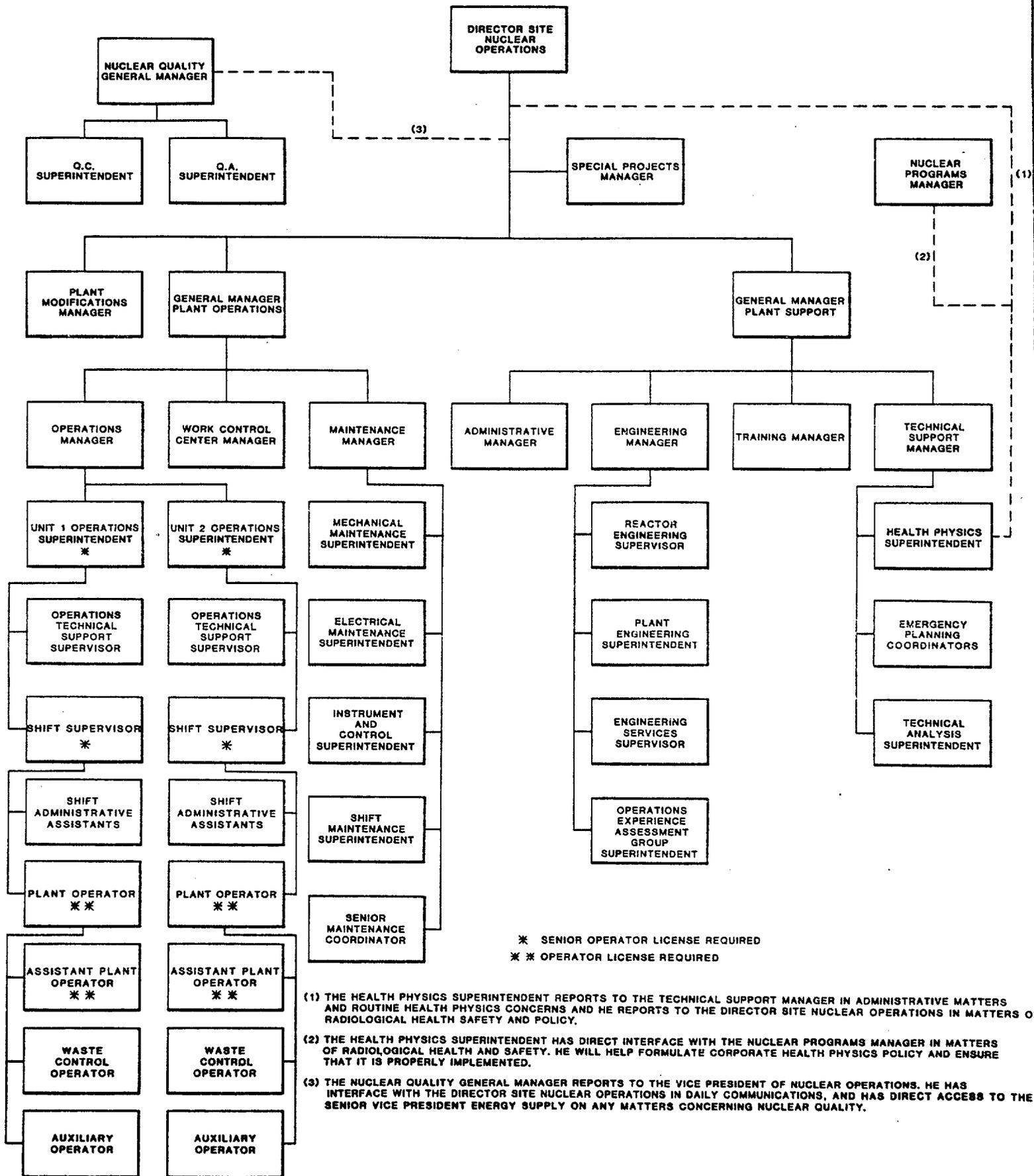


FIGURE 6.2-2 FUNCTIONAL ORGANIZATION FOR PLANT OPERATIONS

Chairman: Manager of Special Projects
Member: Operations Manager
Member: Maintenance Manager
Member: Training Manager
Member: Technical Analysis Superintendent
Member: Engineering Manager
Member: Quality Assurance Superintendent
Member: Reactor Engineering Supervisor
Member: Health Physics Superintendent

The Director, Site Nuclear Operations should designate in writing the Alternate Chairman in the absence of the Chairman.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PSC necessary for the performance of the PSC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and revisions thereto, 2) any other proposed procedures or revisions thereto as determined by the Director, Site Nuclear Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Director, Site Nuclear Operations and to the Chairman of the Safety Review Committee.
- f. Review of those Reportable Occurrences requiring 24-hour notification of the Commission.

- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Director, Site Nuclear Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Director, Site Nuclear Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Director, Site Nuclear Operations and the Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and the Process Control Program.

AUTHORITY

6.5.1.7.1 The Plant Safety Committee shall:

- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the the Vice-President, Nuclear Operations and the Safety Review Committee of disagreement between the PSC and the Director, Site Nuclear Operations; however, the Director, Site Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Director, Site Nuclear Operations and Chairman of the Safety Review Committee.

6.5.2 Safety Review Committee (SRC)
FUNCTION

- 6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry

- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The Safety Review Committee shall be composed of a Chairman and eight members which collectively have the experience and competence required by ANSI/ANS-3.1-1981 to review problems in the areas specified in Section 6.5.2.1, a-h.

The Vice President of Nuclear Operations shall designate, in writing, the Chairman and all SRC members.

The Chairman shall designate, in writing, the alternate Chairman in the absence of the SRC Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 The minimum quorum of the SRC necessary for the performance of the SRC review and audit functions of these technical specifications shall consist of the Chairman or his designated alternate and at least 4 SRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety questions as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviation from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring 24-hour notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspects of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meeting minutes of the Plant Safety Committee.
- j. Changes to Offsite Dose Calculation Manual and Process Control Program.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training, and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or methods of operation that affect nuclear safety at least once per 6 months.

- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of 10 CFR 50 Appendix "B", at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of unit operation considered appropriate by the SRC or the Vice President, Nuclear Operations.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified off-site licensee personnel or an outside fire protection firm.
- j. The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Vice President, Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of SRC activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear Operations, within 14 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear Operations, within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above shall be forwarded to the Vice President, Nuclear Operations, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

Amendment No. 18, 30, 34, 37, 42,
47, 58, 78, 82, 88,
99

6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for Reportable Occurrences:
- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.12.
 - b. Each Reportable Occurrence requiring 24-hour notification to the Commission shall be reviewed by the PSC and submitted to the SRC and the Vice President, Nuclear Operations by the Director, Site Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The facility shall be placed in at least hot shutdown within one hour.
 - b. The Nuclear Regulatory Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.36 and Specification 6.12.3.1.

6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:
- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
 - b. Refueling operations.
 - c. Surveillance and test activities of safety related equipment.
 - d. Security Plan implementation.
 - e. Emergency Plan implementation.
 - f. Fire Protection Program implementation.
 - g. New and spent fuel storage.
 - h. Offsite Dose Calculation Manual and Process Control Program implementation at the site.
 - i. Post accident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the Director, Site Nuclear Operations or the responsible General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PSC and approved by the Director, Site Nuclear Operations or the responsible General Manager within 14 days of implementation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated December 20, 1985 as supplemented February 7, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of the Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 73FACILITY OPERATING LICENSE NO. NPF-6DOCKET NO. 50-368

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

6-1
6-2
6-3
6-5
6-6
6-7
6-8
6-11
6-12
6-13
6-14

Insert Pages

6-1
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6-14

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Director, Site Nuclear Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
- g. Administrative controls shall be established to limit the amount of overtime worked by plant staff performing safety related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).

ARKANSAS POWER & LIGHT COMPANY

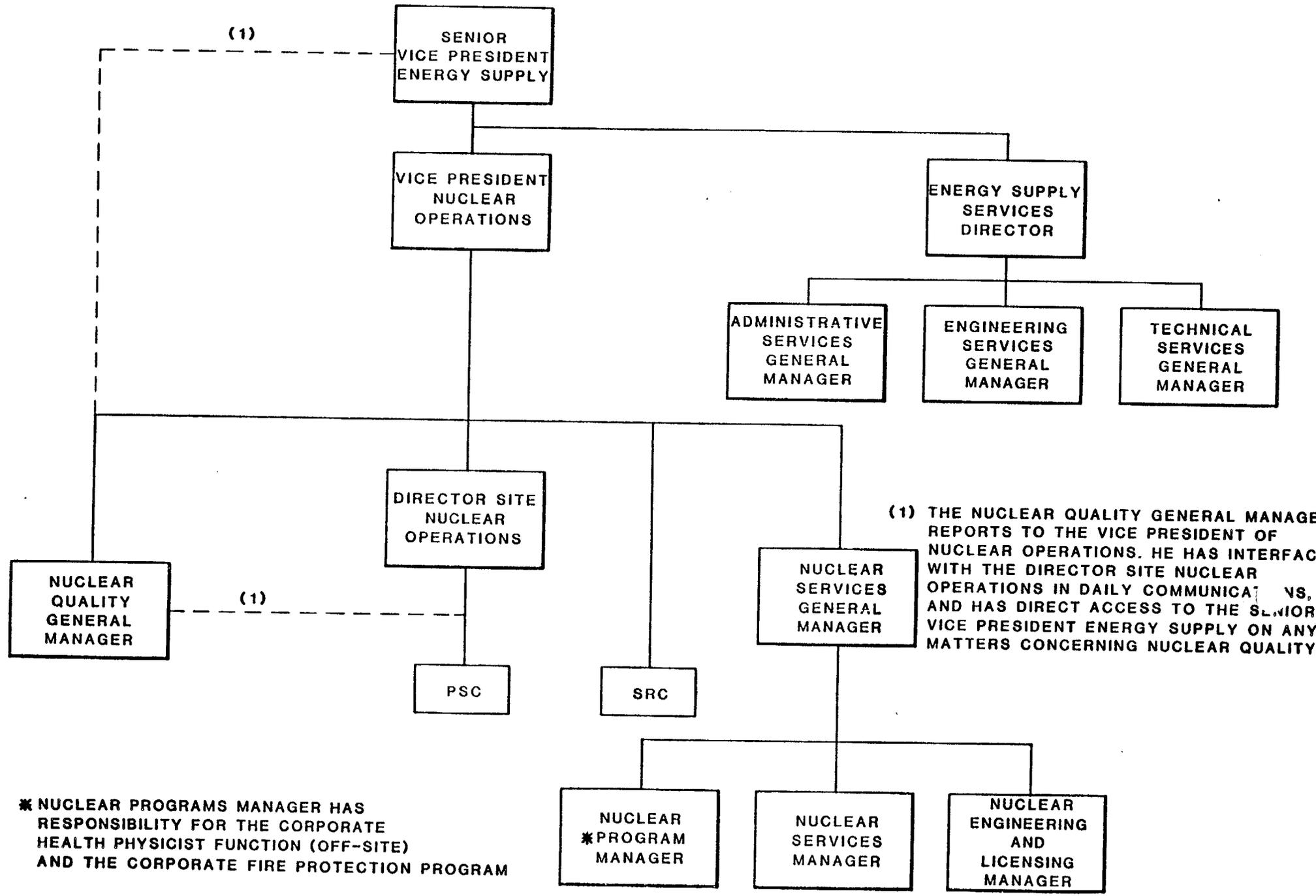


FIGURE 6.2-1 MANAGEMENT ORGANIZATION CHART

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	2	1
Shift Technical Advisor	1	None Required

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a bachelors degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements of Section 27 of the NFPA Code - 1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY COMMITTEE (PSC)

FUNCTION

6.5.1.1 The Plant Safety Committee shall function to advise the Director, Site Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of the:

Chairman: Manager of Special Projects
Member: Operations Manager
Member: Maintenance Manager
Member: Training Manager
Member: Technical Analysis Superintendent
Member: Engineering Manager
Member: Quality Assurance Superintendent
Member: Reactor Engineering Supervisor
Member: Health Physics Superintendent
Member: Nuclear Software Expert*

The Director, Site Nuclear Operations shall designate in writing the Alternate Chairman in the absence of the PSC Chairman.

*See Page 6-5a

ADMINISTRATIVE CONTROLS

*If one of the above members of the Plant Safety Committee meets the qualification requirements for this position, the requirement to have this member is satisfied. This membership may be filled by two appropriately qualified individuals who shall ballot with a single combined vote. Generic qualifications for this membership shall be as follows:

One Individual

The Nuclear Software Expert shall have as a minimum a Bachelor's degree in Science or Engineering, Nuclear preferred (in accordance with ANSI N18.1). In addition, he shall have a minimum of four years of technical experience, of which a minimum of two years shall be in Nuclear Engineering and a minimum of two years shall be in Software Engineering. (Software Engineering is that branch of science and technology which deals with the design and use of software. Software Engineering is a discipline directed to the production and modification of computer programs that are correct, efficient, flexible, maintainable, and understandable, in reasonable time spans, and at reasonable costs). The two years of technical experience in Software Engineering may be general software experience not necessarily related to the software of the Core Protection Calculator System. One of these two years of experience shall be with certified computer programs.

Two Individuals

One of the individuals shall meet the requirements of the Nuclear Engineering portion of the above. The second individual shall have a Bachelor of Science degree (digital computer speciality) and meet the Software Engineering requirements of the above.

The membership (the Nuclear Software Expert or the Digital Computer Specialist) shall be knowledgeable of the Core Protection Calculator System with regard to:

- a. The software modules, their interactions with each other and with the data base.
- b. The relationship between operator's module inputs and the trip variables.
- c. The relationship between sensor input signals and the trip variable.
- d. The design basis of the Core Protection Calculator System.
- e. The approved software change procedure and documentation requirements of a software change.
- f. The security of the computer memory and access procedures to the memory.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PSC necessary for the performance of the PSC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The Plant Safety Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Director, Site Nuclear Operations to affect nuclear safety.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to Appendix "A" Technical Specifications.
 - d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Director, Site Nuclear Operations and to the Chairman of the Safety Review Committee.

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- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses, and reports thereon as requested by the Director, Site Nuclear Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Director, Site Nuclear Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Director, Site Nuclear Operations and the Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and Process Control Program.

AUTHORITY

6.5.1.7 The Plant Safety Committee shall:

- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President, Nuclear Operations and the Safety Review Committee of disagreement between the PSC and the Director, Site Nuclear Operations; however, the Director, Site Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Director, Site Nuclear Operations and Chairman of the Safety Review Committee.

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6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The Safety Review Committee shall be composed of a Chairman and eight members which collectively have the experience and competence required by ANSI/ANS-3.1-1981 to review problems in the areas specified in Section 6.5.2.1, a-h.

The Vice President, Nuclear Operations shall designate, in writing, the Chairman and all SRC members.

The Chairman shall designate, in writing, the alternate Chairman in the absence of the SRC Chairman.

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AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training, and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or methods of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix "B", at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of unit operation considered appropriate by the SRC or the Vice President, Nuclear Operations.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified off-site licensee personnel or an outside fire protection firm.
- j. The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
- k. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Vice President, Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

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RECORDS

6.5.2.10 Records of SRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear Operations within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Nuclear Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PSC and submitted to the SRC and the Vice President, Nuclear Operations by the Director, Site Nuclear Operations.

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6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations and to the SRC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Vice-President, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. Modification of Core Protection Calculator (CPC) Addressable Constants
NOTE: Modification to the CPC addressable constants based on information obtained through the Plant Computer - CPC data link shall not be made without prior approval of the Plant Safety Committee.
- h. New and spent fuel storage.
- i. Offsite Dose Calculation Manual and Process Control Program implementation.
- j. Post-accident sampling (includes sampling of reactor coolant, radioactive iodines and particulates in plant gaseous effluent, and the containment atmosphere).

6.8.2 Each procedure of 6.8.1 above, the changes thereto, shall be reviewed by the PSC and approved by the Director, Site Nuclear Operations or the responsible General Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

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6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PSC and approved by the Director, Site Nuclear Operations or the responsible General Manager within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Administrator of the Regional Office unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 99 AND 73 TO FACILITY OPERATING LICENSE

NOS. DPR-51 AND NPF-6

ARKANSAS POWER AND LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT NOS. 1 AND 2

DOCKET NOS. 50-313 AND 50-368

1.0 INTRODUCTION

By letter dated December 20, 1985, and supplemented by information provided by letter dated February 7, 1986, Arkansas Power & Light Company (AP&L) requested changes to the Administrative Controls sections of the Technical Specifications for Arkansas Nuclear One, Unit Nos. 1 and 2. The proposed amendments would revise Technical Specifications to reflect a reorganization of the Nuclear Operations Department of AP&L. In addition, the proposed amendments would revise the Technical Specifications pertaining to the responsibility and membership of the Safety Review Committee (SRC).

2.0 EVALUATION

The proposed changes were reviewed using the applicable acceptance criteria of Sections 13.1.1, 13.1.2 and 13.4 of the Standard Review Plan (SRP). Our evaluations are provided hereinafter.

1. Figure 6.2-1 - Management Organization Chart

AP&L has revised the organization under the Vice President, Nuclear Operations. These changes include the following:

- a. AP&L has consolidated the current Energy Quality Assurance Section, the ANO Construction Quality Control Section, and the Nuclear Quality Control Section so that they all report to the new position of Nuclear Quality General Manager. The Nuclear Quality General Manager reports to the Vice President, Nuclear Operations.
- b. The new position of Nuclear Services General Manager, who reports to the Vice President, Nuclear Operations, has been created. Reporting to the Nuclear Services General Manager is the Nuclear Program Manager, the Nuclear Services Manager, and the Nuclear Engineering and Licensing Manager.

- c. The new position of Nuclear Program Manager has been created. The Nuclear Program Manager, who reports to the Nuclear Services General Manager, has responsibility for the corporate health physics function and the corporate fire protection program.

The staff has reviewed the above described changes and determined that: (1) the proposed organizational structure provides for the integrated management of activities that support the safe operation and maintenance of the facilities, and (2) clear management control and effective lines of authority and communications exist between the organizational units involved in the management, operation, and technical support for the operation of the facilities. Therefore, the above described changes meet the applicable acceptance criteria of Section 13.1.1, "MANAGEMENT AND TECHNICAL SUPPORT ORGANIZATION", of the SRP and are acceptable to the staff.

2. Figure 6.2-2 - Functional Organization for Plant Operations

AP&L has revised the site organization for Arkansas Nuclear One (ANO). The changes include the following:

- a. The new position of Director, Site Nuclear Operations who has overall responsibility for the site has been established. Reporting to the Director, Site Nuclear Operations will be the new positions of Plant Modifications Manager who is responsible for all ANO construction projects, the General Manager Plant Operations who is responsible for the operation and maintenance of ANO, the new position of General Manager Plant Support who is responsible for Engineering, Training and Technical Support, and the Special Projects Manager.
- b. The new positions of Engineering Manager and Work Control Center Manager have been created. The new positions incorporate the functions that previously were the responsibility of the Engineering and Technical Support Manager. The position of Engineering and Technical Support Manager has been abolished.
- c. The new position of Training Manager, and Work Control Center Manager have been created. The Training Manager assumes the function formerly done by the Training Superintendent. The Work Control Center Manager will be responsible for ANO's new Work Control Center.
- d. Two new positions of Operational Technical Support Supervisor have been created. They report to the respective Unit Operations Superintendents (one for Unit 1 and one for Unit 2).

The staff has reviewed these changes and determined that they will enhance the effectiveness of the Director, Site Nuclear Operations in that by delegating some of his responsibilities to the General Manager Plant Operations and the General Manager Plant Support, his attention can be focused on activities related to important safety issues. In addition, the proposed site organization still maintains clear lines of authority to the Director, Site Nuclear Operations and responsibility for all activities important to the safe operation of the facilities. Therefore, these changes meet the applicable acceptance criteria of Section 13.1.2, "OPERATING ORGANIZATION", and are acceptable to the staff.

3. Section 6.5.2.2 - The proposed change would replace job titles referenced in the Specification for the composition of the SRC with the statement that the SRC shall be composed of a chairman and eight members which collectively have the experience and competence required by ANSI/ANS - 3.1 - 1981 to review problems in the areas specified in Section 6.5.2.1, a-h. This change would reduce the need for future Technical Specification changes due to minor restructuring of the organization or title changes.

The applicable criteria of Section 13.4, "OPERATIONAL REVIEW", of the SRP specify that ANSI/ANS - 3.1 is an acceptable industry standard for qualification levels of persons performing independent reviews of plant operations. The proposed change is acceptable to the staff since it meets the applicable acceptance criteria of Section 13.4. As a further clarification to Specification 6.5.2.2, the licensee agreed during a telephone discussion of February 21, 1986, that this Specification requires that each member of the SRC meet the qualification levels as described in Section 4.4 of ANSI/ANS - 3.1 - 1981.

4. There are miscellaneous changes including title changes throughout Section 6.0 to reflect the reorganization and editorial changes. We reviewed these changes and find them acceptable since they are purely administrative changes to reflect the new organization, to correct a typographical error and to make minor editorial changes.

3.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal contributor: F. Allenspach

Dated: June 10, 1986