

December 20, 1984

DMB 016

Docket No. 50-313

Mr. John M. Griffin, Senior Vice President
of Energy Supply
Arkansas Power and Light Company
P. O. Box 551

Dear Mr. Griffin:

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The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit 1 (ANO-1). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 9, 1984.

The amendment revises the TSs to allow the ten-year hydrostatic test of the secondary system to be performed using steam in lieu of water.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next Monthly Notice.

Sincerely,

"ORIGINAL SIGNED BY:"

Guy S. Vissing, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 90 to DPR-51
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated October 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

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Arkansas Power & Light Company

50-313, Arkansas Nuclear One, Unit 1

cc w/enclosure(s):

Mr. J. Ted Enos
Manager, Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Mr. Frank Wilson
Director, Division of Environmental
Health Protection
Department of Health
Arkansas Department of Health
4815 West Markham Street
Little Rock, Arkansas 72201

Mr. James M. Levine
General Manager
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Mr. W. D. Johnson
U.S. Nuclear Regulatory Commission
P. O. Box 2090
Russellville, Arkansas 72801

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Nicholas S. Reynolds
Bishop, Liberman, Cook, Purcell & Reynolds
1200 17th Street, NW
Washington, DC 20036

Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Regional Radiation Representative
EPA Region VI
1201 Elm Street
Dallas, Texas 75270

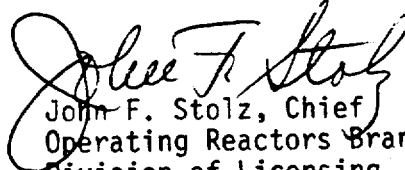
Mr. Robert Martin, Regional Administrator
U. S. Nuclear Regulatory Commission, Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 20, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

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Insert

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3.4 STEAM AND POWER CONVERSION SYSTEM

Applicability

Applies to the turbine cycle components for removal of reactor decay heat.

Objective

To specify minimum conditions of the turbine cycle equipment necessary to assure the capability to remove decay heat from the reactor core.

Specifications

- 3.4.1 The reactor shall not be heated, above 280°F unless the following conditions are met:
1. Capability to remove a decay load of 5% full reactor power by at least one of the following means:
 - a. A condensate pump and a main feedwater (MFW) pump, using turbine by-pass valve.
 - b. A condensate pump and the auxiliary feedwater (AFW) pump using turbine by-pass valve.
 - **2. Fourteen of the steam system safety valves are operable.
 3. A minimum of 16.3 ft. (107,000 gallons) of water is available in the condensate storage tank.
 4. Both emergency feedwater (EFW) pumps and both EFW block valves are capable of automatic actuation, or a dedicated operator is available for their operation.*
 5. Both main steam block valves and both main feedwater isolation valves are operable.
 6. The emergency feedwater valves associated with Specification 3.4.1.4 shall be operable.
- 3.4.2 The Steam Line Break Instrumentation and Control System (SLBIC) shall be operable when main steam pressure exceeds 700 psig and shall be set to actuate at 600 ± 25 psig.
- * One train of EFW may be removed from the control-grade automatic actuation mode for purposes of surveillance testing of the automatic actuation circuitry for a period not to exceed one (1) hour per test without invoking the reporting requirements of Specification 6.12.3.
- ** Except that during hydrotests, with the reactor subcritical, fourteen of the steam system safety valves may be gagged and two (one on each header), may be reset for the duration of the test, to allow the required pressure for the test to be attained.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-51

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT 1

DOCKET NO. 50-313

INTRODUCTION

By letter dated October 9, 1984, Arkansas Power and Light Company (AP&L or the licensee) requested amendment to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit 1 (ANO-1). The proposed changes would modify the TSs to permit the ten-year hydrostatic test of the secondary system to be performed using steam in lieu of water.

DISCUSSION

The Inservice Inspection Program for ANO-1 is based on the 1974 Edition of Section XI of the ASME Code (the Code) which requires Class 2 systems to be hydrostatic tested at 1.25 times the design pressure of the system. The main steam system is unisolated from the steam relief headers and the hydrostatic test pressure is higher than the set pressures of the main steam relief valves. Therefore, to accomplish the test requirements, AP&L proposes to gag 14 relief valves (render the valves such that they would not open), reset two at a higher pressure than the test pressure, and utilize reactor coolant pump heat to produce steam as the pressurizing medium as allowed by the 1980 Edition of Section XI of the Code. As presently written, Technical Specification 3.4.1.2 requires that 14 relief valves be operable if reactor coolant temperature is above 280°F. The proposed change would make an exception to this specification when the reactor is in a subcritical mode of operation and the secondary system hydrostatic test is performed.

EVALUATION

We have reviewed the proposed changes to the hydrostatic test and the Technical Specification. The hydrostatic test will be performed in accordance with the requirements of the 1974 Edition of Section XI except that steam in lieu of water will be used to pressurize the secondary system. This is allowed in the later edition of the Code which has been approved by the Commission. The relieving capacity of the two relief valves is much greater than the energy generated by decay heat and reactor coolant pump heat thereby providing overpressure protection in accordance with Section III of the Code. We, therefore, find that the proposed Technical Specification change to accommodate the performance of the hydrostatic test will not affect plant safety and is acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 20, 1984

Principal Contributors: G. Johnson, W. Jensen and G. Vissing