

FEB 11 1985

DMB 016

Docket No. 50-313

Mr. John M. Griffin, Senior Vice President
of Energy Supply
Arkansas Power and Light Company
P. O. Box 551
Little Rock, Arkansas 72203

DISTRIBUTION	Gray File+4
<u>Docket File</u>	LHarmon
NRC PDR	ACRS-10
L PDR	TBarnhart-4
ORB#4 Rdg	WJones
DEisenhut	DBrinkman
OELD	RDiggs
EJordan	JPartlow
PMcKee	RIngram
CMiles	GVissing
EBlackwood	HOornstein

Dear Mr. Griffin:

On March 11, 1983, the Commission issued Amendment No. 72 to Facility Operating License No. DPR-51 for the Arkansas Nuclear One, Unit 1 (ANO-1). This amendment authorized a change to delete Appendix B Technical Specifications (TS) Sections 6.3 and 6.4 related to water quality. These sections were inadvertently not deleted. Corrected TS pages 6-3 and 6-4 are enclosed.

Likewise, when Amendment No. 76 was issued on April 15, 1983, changes to Appendix A TS 3.8.17 were authorized but were mistakenly omitted. An updated TS page 59a is also enclosed.

We regret any inconvenience that these oversights may have caused.

Sincerely,

"ORIGINAL SIGNED BY"

Guy S. Vissing, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Appendix B TS pages 6-3 and 6-4
2. Appendix A TS page 59a

cc w/enclosures:
See next page

ORB#4:DL
RIngram
2/4/85

ORB#4:DL
GVissing;cf
2/4/85

ORB#4:DL
JStor
2/4/85

8502220364 850211
PDR ADDCK 05000313
P PDR

Arkansas Power & Light Company

50-313, Arkansas Nuclear One, Unit 1

cc w/enclosure(s):

Mr. J. Ted Enos
Manager, Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Mr. Frank Wilson
Director, Division of Environmental
Health Protection
Department of Health
Arkansas Department of Health
4815 West Markham Street
Little Rock, Arkansas 72201

Mr. James M. Levine
General Manager
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Mr. W. D. Johnson
U.S. Nuclear Regulatory Commission
P. O. Box 2090
Russellville, Arkansas 72801

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Nicholas S. Reynolds
Bishop, Liberman, Cook, Purcell & Reynolds
1200 17th Street, NW
Washington, DC 20036

Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Regional Radiation Representative
EPA Region VI
1201 Elm Street
Dallas, Texas 75270

Mr. Robert Martin, Regional Administrator
U. S. Nuclear Regulatory Commission, Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

6.3 Deleted

Amendment No. 72

8502220368 850211
PDR ADOCK 05000313
P PDR

6-4

6.4 Deleted

Amendment No. 72

- 3.8.15 The spent fuel shipping cask shall not be carried by the Auxiliary Building crane pending the evaluation of the spent fuel cask drop accident and the crane design by AP&L and NRC review and approval. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
- 3.8.16 Storage in the spent fuel pool shall be restricted to fuel assemblies having initial enrichment less than or equal to 4.1 w/o U-235. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
- 3.8.17 Storage in Region 2 (as shown on Figure 3.8.1) of the spent fuel pool shall be further restricted by burnup and enrichment limits specified in Figure 3.8.2. In the event a checkerboard storage configuration is deemed necessary for a portion of Region 2, vacant spaces adjacent to the faces of any fuel assembly which does not meet the Region 2 burnup criteria (Non-Restricted) shall be physically blocked before any such fuel assembly may be placed in Region 2. This will prevent inadvertent fuel assembly insertion into two adjacent storage locations. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
- 3.8.18 The boron concentration in the spent fuel pool shall be maintained (at all times) at greater than 1600 parts per million.

BASES

Detailed written procedures will be available for use by refueling personnel. These procedures, the above specifications, and the design of the fuel handling equipment as described in Section 9.6 of the FSAR incorporating built-in interlocks and safety features, provide assurance that no incident could occur during the refueling operations that would result in a hazard to public health and safety. If no change is being made in core geometry, one flux monitor is sufficient. This permits maintenance on the instrumentation. Continuous monitoring of radiation levels and neutron flux provides immediate indication of an unsafe condition.

The requirement that at least one decay heat removal loop be in operation ensures that (1) sufficient cooling capacity is available to remove decay heat and maintain the water in the reactor pressure vessel at the refueling temperature (normally 140°F), and (2) sufficient coolant circulation is maintained through the reactor core to minimize the effects of a boron dilution incident and prevent boron stratification. (1)

The requirement to have two decay heat removal loops operable when there is less than 23 feet of water above the core, ensures that a single failure of the operating decay heat removal loop will not result in a complete loss of decay heat removal capability. With the reactor vessel head removed and 23 feet of water above the core, a large heat sink is available for core cooling, thus in the event of a failure of the operating decay heat removal loop, adequate time is provided to initiate emergency procedures to cool the core.

The shutdown margin indicated in Specification 3.8.4 will keep the core subcritical, even with all control rods withdrawn from the core. (2) Although the refueling boron concentration is sufficient to maintain the core $k_{eff} < 0.99$ if all the control rods were removed from the core, only a few control rods will be removed at any one time during fuel shuffling and