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Docket No. 50-313

# REGULATORY DOCKET FILE COPY

Mr. William Cavanaugh, III  
Vice President, Generation  
and Construction  
Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications in response to your license amendment requests dated April 24, 1979, and February 4, 1980.

The amendment changes the Technical Specifications to reflect a power operated relief valve (PORV) setpoint of 2450 psig, a high pressure reactor trip setpoint of 2300 psig, and a containment isolation initiation setpoint on low reactor coolant system pressure (1500 psig).

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by  
Robert W. Reid  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

- Enclosures:  
1. Amendment No. 49  
2. Safety Evaluation  
3. Notice

cc w/enclosures: See next page

8101090 950 p

\*See previous yellow for concurrences.

*Comments by IE  
via Telephone call  
from Don Kirkpatrick  
12/18/80*

CP 1

OPERATING REACTORS BRANCH

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and Construction		Heltemes, AEOD
Arkansas Power & Light Company		Gray File +4
P. O. Box 551		HOrnstein
Little Rock, Arkansas 72203		EBlackwood

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications in response to your license amendment requests dated April 24, 1979, and February 4, 1980, as supplemented by letters dated February 12, 1980, and October 31, 1980.

The amendment changes the Technical Specifications to reflect a power operated relief valve (PORV) setpoint of 2450 psig, a high pressure reactor trip setpoint of 2300 psig, operating limits and surveillance and reporting requirements for the PORV and the associated block valve, and a containment isolation initiation setpoint on low reactor coolant system pressure (1500 psig).

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,  
Original signed by  
Robert W. Reid

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 49
2. Safety Evaluation
3. Notice

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Docket No. 50-313

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and Construction  
Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

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The amendment changes the Technical Specifications to reflect a power operated relief valve setpoint of 2450 psig, a high pressure reactor trip setpoint of 2300 psig, and a containment isolation initiation setpoint on low reactor coolant system pressure (1500 psig).

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No.
2. Safety Evaluation
3. Notice

cc: w/enclosures:  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 19, 1980

Docket No. 50-313

Mr. William Cavanaugh, III  
Vice President, Generation  
and Construction  
Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications in response to your license amendment requests dated April 24, 1979, and February 4, 1980.

The amendment changes the Technical Specifications to reflect a power operated relief valve (PORV) setpoint of 2450 psig, a high pressure reactor trip setpoint of 2300 psig, and a containment isolation initiation setpoint on low reactor coolant system pressure (1500 psig).

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

*Morton B. Fairtile for*

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 49
2. Safety Evaluation
3. Notice

cc w/enclosures: See next page

Arkansas Power & Light Company

cc w/enclosure(s):

Mr. David C. Trimble  
Manager, Licensing  
Arkansas Power & Light Company  
P. O. Box 551  
Little Rock, Arkansas 72203

Mr. James P. O'Hanlon  
General Manager  
Arkansas Nuclear One  
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Russellville, Arkansas 72801

Mr. William Johnson  
U.S. Nuclear Regulatory Commission  
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Russellville, Arkansas 72801

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 420, 7735 Old Georgetown Road  
Bethesda, Maryland 20014

Mr. Nicholas S. Reynolds  
Debevoise & Liberman  
1200 17th Street, NW  
Washington, DC 20036

Arkansas Polytechnic College  
Russellville, Arkansas 72801

Honorable Ermil Grant  
Acting County Judge of Pope County  
Pope County Courthouse  
Russellville, Arkansas 72801

Mr. Paul F. Levy, Director  
Arkansas Department of Energy  
3000 Kavanaugh  
Little Rock, Arkansas 72205

Director, Criteria and Standards  
Division  
Office of Radiation Programs (ANR-460)  
U. S. Environmental Protection Agency  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Region VI Office  
ATTN: EIS COORDINATOR  
1201 Elm Street  
First International Building  
Dallas, Texas 75270

cc w/enclosure(s) & incoming dtd.:

4/24/79 & 2/4/80

Director, Bureau of Environmental  
Health Services  
4815 West Markham Street  
Little Rock, Arkansas 72201



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49  
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Arkansas Power & Light Company (the licensee) dated April 24, 1979 and February 4, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 49, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Morton B. Faircliff for*  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 49

FACILITY OPERATING LICENSE NO. DPR-51

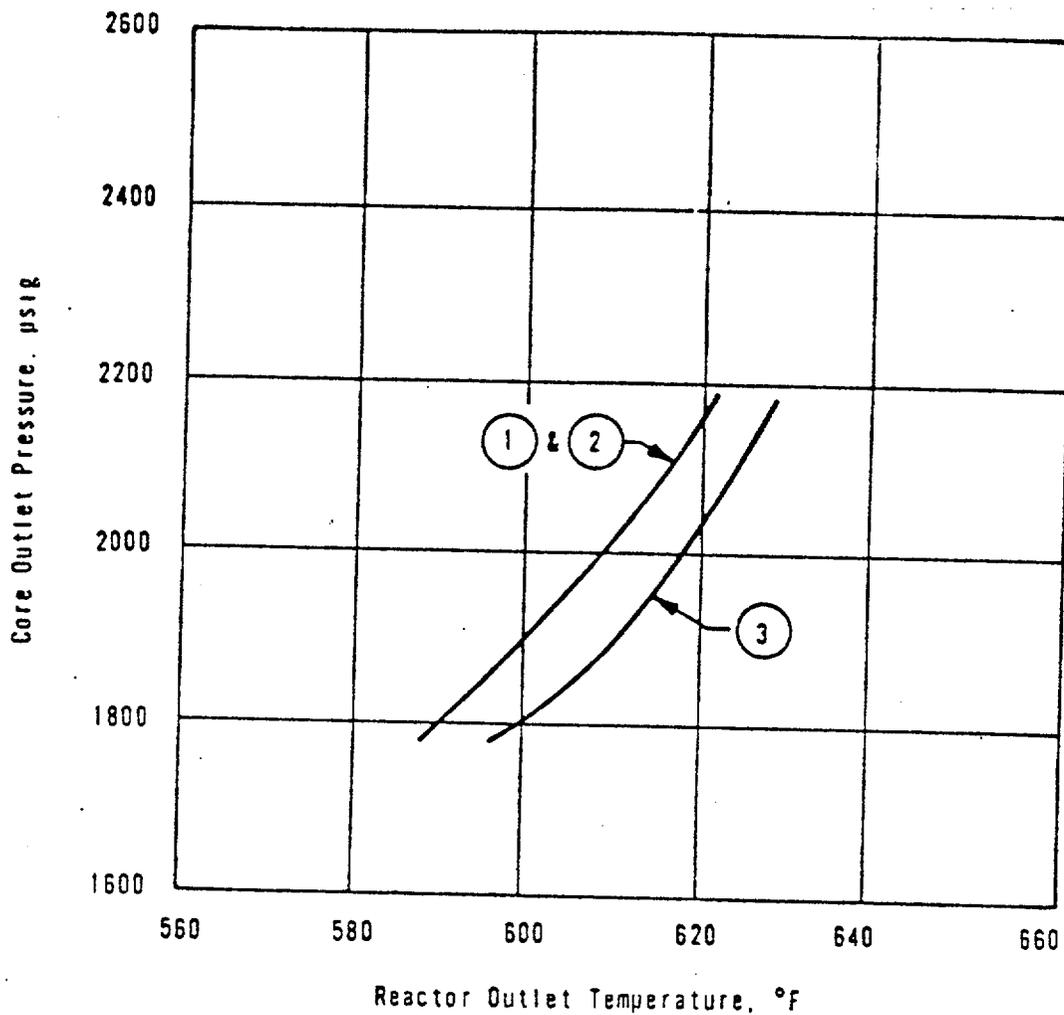
DOCKET NO. 50-313

Revise the Appendix A Technical Specifications as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
9c*	9c*
10	10
13	13
14a	14a
15	15
49	49
50	50

Changes on the revised pages are identified by marginal lines.

\*Overleaf page - no changes.



CURVE	GPM	POWER	PUMPS OPERATING (TYPE OF LIMIT)
1	374,880 (100%)*	112%	FOUR PUMPS (DNBR LIMIT)
2	280,035 (74.7%)	86.7%	THREE PUMPS (DNBR LIMIT)
3	184,441 (49.2%)	59.0%	ONE PUMP IN EACH LOOP (QUALITY LIMIT)

\*106.5% OF DESIGN FLOW

ARKANSAS POWER & LIGHT CO. ARKANSAS NUCLEAR ONE-UNIT 1	CORE PROTECTION SAFETY LIMITS	FIG. NO. 2.1-3
-----------------------------------------------------------	-------------------------------	-------------------

## 2.2 SAFETY LIMITS -- REACTOR SYSTEM PRESSURE

### Applicability

Applies to the limit on reactor coolant system pressure.

### Objective

To maintain the integrity of the reactor coolant system and to prevent the release of significant amounts of fission product activity.

### Specification

- 2.2.1 The reactor coolant system pressure shall not exceed 2750 psig when there are fuel assemblies in the reactor vessel.
- 2.2.2 The setpoint of the pressurizer code safety valves shall be in accordance with ASME, Boiler and Pressurizer Vessel Code, Section III, Article 9, Summer 1968.

### Bases

The reactor coolant system<sup>(1)</sup> serves as a barrier to prevent radionuclides in the reactor coolant from reaching the atmosphere. In the event of a fuel cladding failure, the reactor coolant system is a barrier against the release of fission products. Establishing a system pressure limit helps to assure the integrity of the reactor coolant system. The maximum transient pressure allowable in the reactor coolant system pressure vessel under the ASME code, Section III, is 110 percent of design pressure.<sup>(2)</sup> The maximum transient pressure allowable in the reactor coolant system piping, valves, and fittings under ANSI Section B31.7 is 110 percent of design pressure. Thus, the safety limit of 2750 psig (110 percent of the 2500 psig design pressure) has been established.<sup>(2)</sup> The settings for the reactor high pressure trip (2300 psig) and the pressurizer code safety valves (2500 psig  $\pm 1\%$ )<sup>(3)</sup> have been established to assure that the reactor coolant system pressure safety limit is not exceeded. The initial hydrostatic test is conducted at 3125 psig (125 percent of design pressure) to verify the integrity of the reactor coolant system. Additional assurance that the reactor coolant system pressure does not exceed the safety limit is provided by setting the pressurizer electromatic relief valve at 2450 psig.<sup>(4)</sup>

### REFERENCES

- (1) FSAR, Section 4
- (2) FSAR, Section 4.3.10.1
- (3) FSAR, Section 4.2.4
- (4) FSAR, Table 4-1

shown in Figure 2.3-1 for high reactor coolant system pressure (2300 psig) has been established to maintain the system pressure below the safety limit (2750 psig) for any design transient. (2) The low pressure (1800 psig) and variable low pressure (11.75T<sub>out</sub>-5103) trip setpoint shown in Figure 2.3-1 have been established to maintain the DNB ratio greater than or equal to 1.3 for those design accidents that result in a pressure reduction. (2,3)

Due to the calibration and instrumentation errors, the safety analysis used a variable low reactor coolant system pressure trip value of (11.75T<sub>out</sub> 5143).

D. Coolant outlet temperature

The high reactor coolant outlet temperature trip setting limit (619F) shown in Figure 2.3-1 has been established to prevent excessive core coolant temperatures in the operating range. Due to calibration and instrumentation errors, the safety analysis used a trip set point of 620F.

E. Reactor building pressure

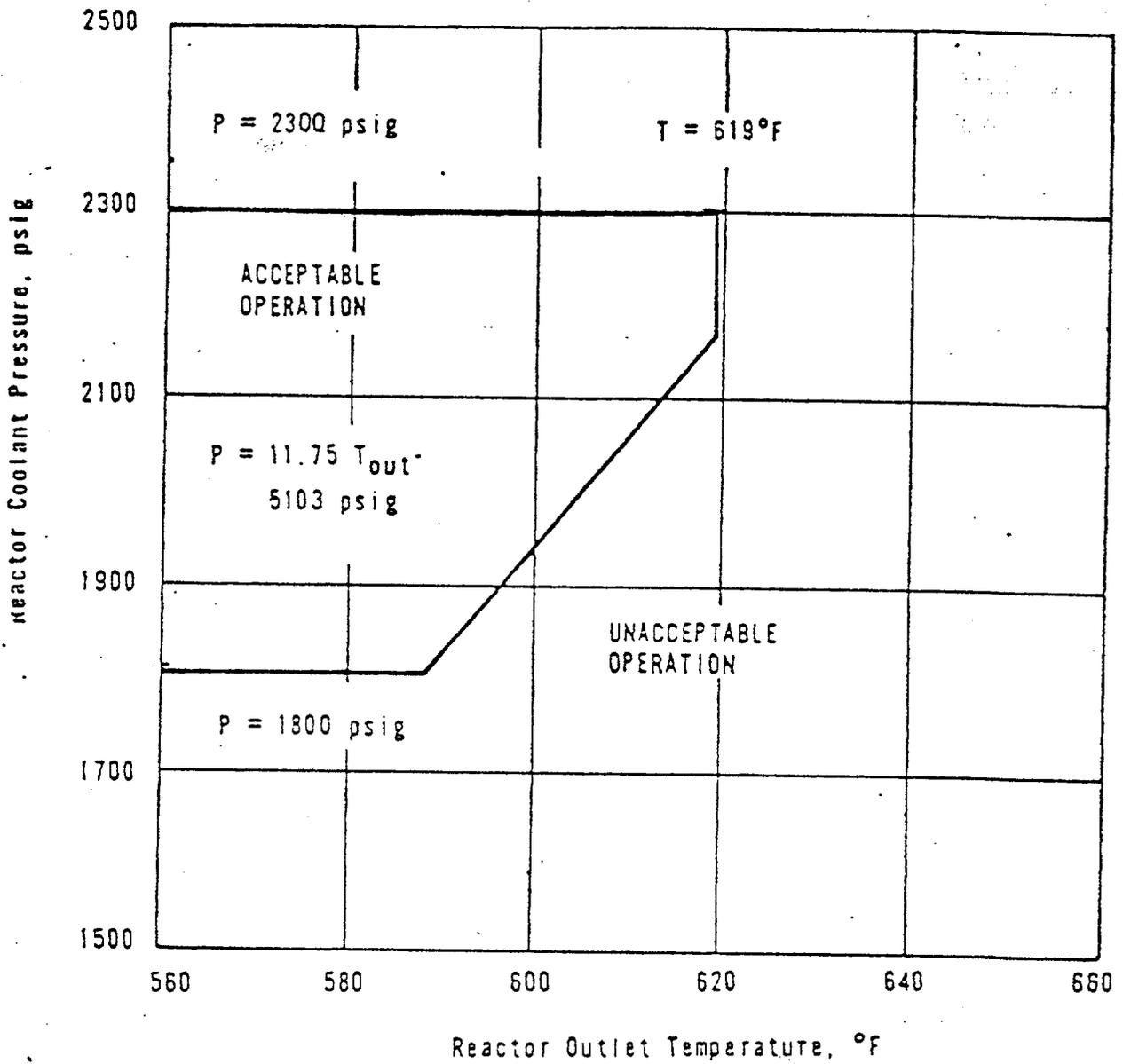
The high reactor building pressure trip setting limit (4 psig) provides positive assurance that a reactor trip will occur in the unlikely event of a steam line failure in the reactor building or a loss-of-coolant accident, even in the absence of a low reactor coolant system pressure trip.

F. Shutdown bypass

In order to provide for control rod drive tests, zero power physics testing, and startup procedures, there is provision for bypassing certain segments of the reactor protection system. The reactor protection system segments which can be bypassed are shown in Table 2.3-1. Two conditions are imposed when the bypass is used:

1. A nuclear overpower trip set point of  $\leq 5.0$  percent of rated power is automatically imposed during reactor shutdown.
2. A high reactor coolant system pressure trip set point of 1720 psig is automatically imposed.

The purpose of the 1720 psig high pressure trip set point is to prevent normal operation with part of the reactor protection system bypassed. This high pressure trip set point is lower than the normal low pressure trip set point so that the reactor must be tripped before the bypass is initiated. The overpower trip set point of  $\leq 5.0$  percent prevents any significant reactor power from being produced when performing the physics tests. Sufficient natural circulation (5) would be available to remove 5.0 percent of rated power if none of the reactor coolant pumps were operating.



ARKANSAS POWER & LIGHT CO. ARKANSAS NUCLEAR ONE-UNIT 1	PROTECTIVE SYSTEM MAXIMUM ALLOWABLE SET POINT	FIG. NO. 2.3-1
-----------------------------------------------------------	--------------------------------------------------	-------------------

Table 2.3-1  
Reactor Protection System Trip Setting Units

	Four Reactor Coolant Pumps Operating (Nominal Operating Power - 100%)	Three Reactor Coolant Pumps Operating (Nominal Operating Power - 75%)	One Reactor Coolant Pump Operating in Each Loop (Nominal Operating Power - 49%)	Shutdown Bypass
Nuclear power, % of rated, max	105.5	105.5	105.5	5.0 <sup>(3)</sup>
Nuclear power based on flow <sup>(2)</sup> and imbalance, % of rated, max	1.057 times flow minus reduction due to imbalance(s)	1.057 times flow minus reduction due to imbalance(s)	1.057 times flow minus reduction due to imbalance(s)	Bypassed
Nuclear power based on pump monitors <sup>(4)</sup> , % of rated, max	NA	NA	55%	Bypassed
High reactor coolant system pressure, psig, max	2300	2300	2300	1720 <sup>(3)</sup>
Low reactor coolant sys- tem pressure, psig, min.	1800	1800	1800	Bypassed
Variable low reactor coolant system pressure, psig, min.	$(11.75T_{out} - 5103)(1)$	$(11.75T_{out} - 5103)(1)$	$(11.75T_{out} - 5103)(1)$	Bypassed
Reactor coolant temp, F, max	619	619	619	619
High reactor building pressure, psig, max	4(18.7 psia)	4(18.7 psia)	4(18.7 psia)	4(18.7 psia)

(1)  $T_{out}$  is in degrees Fahrenheit (F)

(2) Reactor coolant system flow, %

(3) Automatically set when other segments of the RPS (as specified) are bypassed

(4) The pump monitors also produce a trip on: (a) loss of two reactor coolant pumps in one reactor coolant loop, and (b) loss of one or two reactor coolant pumps during two-pump operation.

### 3.5.3 Safety Features Actuation System Setpoints

#### Applicability

This specification applies to the safety features actuation system actuation setpoints.

#### Objective

To provide for automatic initiation of the safety features actuation system in the event of a breach of reactor coolant system integrity.

#### Specification

The safety features actuation setpoints and permissible bypasses shall be as follows:

<u>Functional Unit</u>	<u>Action</u>	<u>Setpoint</u>
High Reactor Building Pressure*	Reactor Building Spray	$\leq$ 30 psig (44.7 psia)
	High Pressure Injection	$\leq$ 4 psig (18.7 psia)
	Start of Reactor Building Cooling and Reactor Building Isolation	$\leq$ 4 psig (18.7 psia)
	Reactor Bldg. Ventilation	$\leq$ 4 psig (18.7 psia)
	Low Pressure Injection	$\leq$ 4 psig (18.7 psia)
	Penetration Room Ventilation	$\leq$ 4 psig (18.7 psia)
Low Reactor Coolant System Pressure**	High Pressure Injection	$\geq$ 1500 psig
	Low Pressure Injection	$\geq$ 1500 psig
	Start of Reactor Building Cooling and Reactor Building Isolation	$\geq$ 1500 psig

\*May be bypassed during reactor building leak rate test.

\*\*May be bypassed below 1750 psig and is automatically reinstated above 1750 psig.

## Bases

### High Reactor Building Pressure

The basis for the 30 psig and 4 psig setpoints for the high pressure signal is to establish a setting which would be reached in adequate time in the event of a DBA, cover a spectrum of break sizes and yet be far enough above normal operation maximum internal pressure to prevent spurious initiation.

### Low Reactor Coolant System Pressure

The basis for the 1500 psig low reactor coolant pressure setpoint for high and low pressure injection initiation is to establish a value which is high enough such that protection is provided for the entire spectrum of break sizes and is far enough below normal operating pressure to prevent spurious initiation. (1)

## REFERENCE

- (1) FSAR, Section 14.2.2.5



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 49 TO

FACILITY OPERATING LICENSE NO. DPR-51

ARKANSAS POWER AND LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT NO. 1

DOCKET NO. 50-313

Introduction

This Safety Evaluation supports two separate issues as follows:

1. By letter dated April 24, 1979, Arkansas Power and Light Company (the licensee or AP&L) requested amendment to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1) which would change the Technical Specifications to reflect a power operated relief valve (PORV) setpoint of 2450 psig and a high pressure reactor trip setpoint of 2300 psig.
2. By letter dated February 4, 1980, the licensee requested amendment to Facility Operating License No. DPR-51 for ANO-1 which would change the Technical Specifications to reflect a containment isolation setpoint for low reactor coolant system pressure.

Discussion and Evaluation

1. PORV and High Pressure Reactor Trip Setpoints  
Item 3 of IE Bulletin 79-05B requested the licensee to describe the modifications to design and procedures which the licensee has implemented to assure the reduction of the frequency of automatic actuation of the pressurizer PORV and thus the likelihood of the PORV sticking open during anticipated transients. By letter dated May 4, 1979, the licensee stated that the setpoint of the PORV had been raised from 2244 psig to 2450 psig and the setpoint of the reactor trip had been lowered from 2355 psig to 2300 psig as was suggested in IE Bulletin 79-05B. The proposed amendment dated April 24, 1979, would implement the change of the reactor trip setpoint and identified the proposed PORV setpoint in the Bases section of the Technical Specifications.

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In the past, during turbine trip and loss of feedwater transients the PORV was lifted and the reactor would not trip unless the pressure setpoint was exceeded. With the new setpoints, these transients do not result in lifting of this valve. Therefore, this valve does not open as frequently and the likelihood of the valve failing to close, i.e. producing a small break loss of coolant event, is reduced. However, the likelihood of a reactor trip is increased. We do not consider this increase in frequency of a reactor trip to be significant over the lifetime of the plant. The PORV function is to control an operational transient and not to protect the reactor coolant system pressure boundary. The safety valves provide this function. The new setpoints would not reduce the margin of safety or increase the probability or consequences of accidents. We find the proposed changes to the Technical Specifications, as modified, acceptable.

## 2. Containment Isolation Setpoint

On January 2, 1980, the Commission issued a Confirmatory Order for ANO-1 which confirmed the licensee's commitment to implement the "Category A" lessons learned requirements (NUREG-0578) prior to plant operation after January 31, 1980. In response to Section 2.1.4 of the NUREG-0578, which required diversity in the parameters sensed for the initiation of containment isolation for non-essential systems, the licensee implemented a means of containment isolation for non-essential systems on low reactor coolant system pressure (1500 psig). The licensee previously provided containment isolation for many non-essential systems only on high reactor building pressure. This did not provide diversity in parameters sensed for initiation of containment isolation. The modifications implemented as a requirement of the Confirmatory Order now provide isolation of containment on high reactor building pressure and low reactor coolant system pressure which are diverse parameters. We found this to meet our requirements and, therefore, acceptable in our letter dated March 10, 1980. Therefore, we find it acceptable to implement the new containment isolation initiation setpoint into the Technical Specifications.

## Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power levels and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 19, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-313ARKANSAS POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No.49 to Facility Operating License No. DPR-51, issued to Arkansas Power and Light Company (the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Unit No. 1 (the facility) located in Pope County, Arkansas. The amendment is effective as of the date of issuance.

The amendment changes the Technical Specifications to reflect a power operated relief valve (PORV) setpoint of 2450 psig, a high pressure reactor trip setpoint of 2300 psig, and a containment isolation initiation setpoint on low reactor coolant system pressure (1500 psig).

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4), and environmental impact statement or negative declaration and

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environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the licensee's applications for amendment dated April 24, 1979, and February 4, 1980,

(2) Amendment No. 49 to License No. DPR-51, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Arkansas Polytechnic College, Russellville, Arkansas. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of December 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing