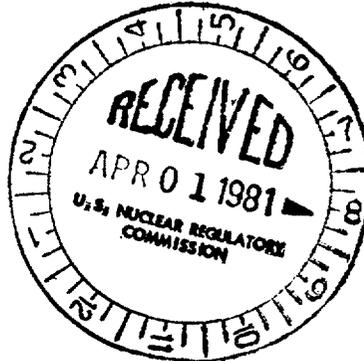


MARCH 23 1981

Dockets Nos. 50-313
and 50-368

Mr. William Cavanaugh, III
Vice President, Generation
and Construction
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203



Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendments Nos. 54 and 22 to Facility Operating Licenses Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1 and 2, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated June 26, 1980. During our review of your proposed amendments, we found that certain modifications were necessary to meet our requirements. Your staff has agreed to these modifications and they have been incorporated in these amendments.

These amendments modify the ANO-1&2 Appendix A Technical Specifications dealing with the surveillance frequency of fire detectors in the reactor buildings.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Printed name by

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Original signed by

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures & cc:
See next page

OFFICE ▶
SURNAME ▶
DATE ▶	8104020	706

Mr. W. Cavanaugh

-2-

Enclosures:

- 1. Amendment No. 54 DPR-51
- 2. Amendment No. 22 NPF-6
- 3. Safety Evaluation
- 4. Notice

cc w/enclosures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

March 24, 1981

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Docket No. 50-313
and 50-368

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: **ARKANSAS POWER AND LIGHT COMPANY**

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).

Other: **Amendments Nos. 54 and 22**

Referenced documents have been provided PDR

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

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DATE →	3/24/81				



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

March 24, 1981

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Docket No. **50-313**
and **50-368**

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: **ARKANSAS POWER AND LIGHT COMPANY**

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (2) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
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Other: Amendments Nos. 54 and 22
Referenced documents have been provided PDR

Division of Licensing
Office of Nuclear Reactor Regulation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket
50-313

March 23, 1981

Dockets Nos. 50-313
and 50-368

Mr. William Cavanaugh, III
Vice President, Generation
and Construction
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

The Commission has issued the enclosed Amendments Nos. 54 and 22 to Facility Operating Licenses Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1 and 2, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated June 26, 1980. During our review of your proposed amendments, we found that certain modifications were necessary to meet our requirements. Your staff has agreed to these modifications and they have been incorporated in these amendments.

These amendments modify the ANO-1&2 Appendix A Technical Specifications dealing with the surveillance frequency of fire detectors in the reactor buildings.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script that reads "Robert A. Clark".

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

A handwritten signature in cursive script that reads "John F. Stolz".

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures & cc:
See next page

Mr. W. Cavanaugh

-2-

Enclosures:

1. Amendment No.54 DPR-51
2. Amendment No.22 NPF-6
3. Safety Evaluation
4. Notice

cc w/enclosures:

See next page

Arkansas Power & Light Company

cc w/enclosure(s):

Mr. David C. Trimble
Manager, Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Mr. James P. O'Hanlon
General Manager
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Mr. William Johnson
U.S. Nuclear Regulatory Commission
P. O. Box 2090
Russellville, Arkansas 72801

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 420, 7735 Old Georgetown Road
Bethesda, Maryland 20014

Mr. Nicholas S. Reynolds
Debevoise & Liberman
1200 17th Street, NW
Washington, DC 20036

Arkansas Tech University
Russellville, Arkansas 72801

Honorable Ermil Grant
Acting County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

Mr. Paul F. Levy, Director
Arkansas Department of Energy
3000 Kavanaugh
Little Rock, Arkansas 72205

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region VI Office
ATTN: EIS COORDINATOR
1201 Elm Street
First International Building
Dallas, Texas 75270

cc w/enclosure(s) & incoming dtd.:
6/26/80

Director, Bureau of Environmental
Health Services
4815 West Markham Street
Little Rock, Arkansas 72201



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated June 26, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

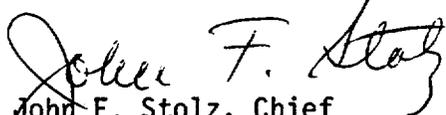
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 23, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Page

110p

4.19 FIRE DETECTION INSTRUMENTATION

Applicability

Applies to surveillance of fire detection instrumentation required operable by Specification 3.5.5.

Objective

To assure that the fire detection instrumentation required operable by Specification 3.5.5 is available and operable when needed.

Specification

- 4.19.1 Each required fire detection instrument* shall be demonstrated operable at least once per six months by performance of a channel functional test.
- 4.19.2 The NFPA Code 72D Class A supervised circuits supervision* associated with the detector alarms of each required fire detection instrument shall be demonstrated operable at least once per six months.
- 4.19.3 The non-supervised circuits between the local panels in Specification 4.19.2 and the control room shall be demonstrated operable at least once per 31 days.

Bases

These required demonstrations will assure operability of the fire detection instrumentation.

*The minimum number of fire detectors and circuits inside the containment shall be functionally tested during each cold shutdown exceeding 24 hours unless performed during the previous six months. All fire detectors located inside the containment and their associated circuits shall be tested during each refueling outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE - UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power and Light Company (the licensee) dated June 26, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 22, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 23, 1981

ATTACHMENT TO LICENSE AMENDMENT NO.22

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

3/4 3-43

B 3/4 3-3

INSTRUMENTATION

FIRE DETECTION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.8 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-11 shall be OPERABLE.

APPLICABILITY: Whenever equipment in that fire detection zone is required to be OPERABLE.#

ACTION:

With one or more of the fire detection instrument(s) shown in Table 3.3-11 inoperable:

- a. Within 1 hour establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour. Fire zones located inside primary containment are exempt from this requirement during Modes 1 and 2.
- b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument(s) to OPERABLE status.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.8.1 Each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST.*

4.3.3.8.2 The NFPA Code 72D Class A supervised circuits supervision associated with the detector alarms of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.*

4.3.3.8.3 The non-supervised circuits between the local panels in Specification 4.3.3.8.2 and the control room shall be demonstrated OPERABLE at least once per 31 days.

The fire detection instruments located within the containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

*The minimum number of fire detectors and circuits required inside the containment shall be functionally tested during each cold shutdown exceeding 24 hours unless performed during the previous six months. All fire detectors located inside the containment and their associated circuits shall be tested during each refueling outage.

TABLE 3.3-11

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>	
	<u>HEAT</u>	<u>SMOKE</u>
1. Containment		
North Penetration Area-Upper	2/Area	2/Area
North Penetration Area-Lower	2/Area	2/Area
South Penetration Area-Upper	2/Area	2/Area
South Penetration Area-Lower	2/Area	2/Area
2. Cable Spreading	Protective Wire	4
3. Switchgear Room		
Room 2100-Elev. 372		2
Room 2101-Elev. 372		2
4. Remote Shutdown Panels		1
5. Emergency Diesel Generator Rooms		
Room 2093-Elev. 369	1	2
Room 2094-Elev. 369	1	2
6. Diesel Fuel Storage Vaults		1/Vault
7. Penetration Rooms		
North Penetration Area-Upper	2/Room	1/Room
North Penetration Area-Lower	2/Room	1/Room
South Penetration Area-Upper	2/Room	1/Room
South Penetration Area-Lower	2/Room	1/Room

INSTRUMENTATION

BASES

3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Plants to Assess Plant Conditions During and Following an Accident," December 1975.

3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection system ensures that sufficient capability is available to promptly detect and initiate protective action in the event of an accidental chlorine release. This capability is required to protect control room personnel and is consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release," February 1975.

3/4.3.3.8 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, except for detectors located in the containment during Modes 1 and 2, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY.

3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment or structures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 54 AND 22 TO
FACILITY OPERATING LICENSES NOS. DPR-51 AND NPF-6

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNITS NOS. 1&2

DOCKETS NOS. 50-313 & 50-368

Introduction

By letter dated June 26, 1980, Arkansas Power & Light Company (the licensee or AP&L) requested amendment of the Technical Specifications (TS), Appendix A, appended to Facility Operating Licenses Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Units Nos. 1 and 2 (ANO-1&2). The amendments would expand the Technical Specifications to accommodate the inspection of detectors located in relatively inaccessible areas such as inside the containment during periods of reactor criticality.

Discussion

The current TS for each unit requires the testing of fire detection instrumentation at least once per six months by performance of channel functional test. For those fire detectors located in the reactor building the testing requires personnel entry into the reactor building. During reactor operation, such entry into the reactor building would expose the testing personnel to a radiation field of approximately 300 MR which the licensee finds inconsistent with ALARA practices. The intent of the proposed change would require the verification of the operability of the fire detectors during cold shutdown unless testing had been performed during the previous six months. However, the wording of the proposed change did not convey the intent. We discussed this with the licensee's staff and the licensee's staff agreed to a modification to the proposed change which would require the verification of the operability of the fire detectors during each cold shutdown exceeding 24 hours unless testing had been performed during the previous six months. The licensee's staff also agreed to a modification to the proposed change which would require all fire detectors located within the containment and their associated circuits to be verified as operable during each refueling outage.

Evaluation

We have reviewed the operating histories of ANO Units 1 and 2 during the past several years and have found that each of the units generally has a cold shutdown frequency greater than once per six months. We expect that future operating experience will be comparable to the experience of the past several years in this regard. Therefore, it is expected that although the licensee will have greater flexibility in scheduling the testing of the detectors inside containment the mean testing frequency will be comparable to the current Technical Specification requirements of once each six months. The additional requirement to verify the operability of all detectors located inside containment during refueling shutdowns provides additional assurance that the required minimum number of detectors will be operable throughout each refueling cycle. Therefore we conclude that (1) it is unlikely that any net reduction in the capability of the fire detection system to perform its function will result from the proposed changes and (2) it is appropriate to implement these changes to reduce the exposure of testing personnel to practices consistent with ALARA principles.

On these bases we conclude that the proposed changes, as modified, would not reduce the margin of safety or increase the consequences or probability of an accident and are therefore acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 23, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKETS NOS. 50-313 AND 50-368
ARKANSAS POWER & LIGHT COMPANY
NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 54 and 22 to Facility Operating Licenses Nos. DPR-51 and NPF-6, issued to Arkansas Power & Light Company (the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Units Nos. 1 and 2 (ANO-1&2) located in Pope County, Arkansas. The amendments are effective as of the date of issuance.

The amendments modify the ANO-1&2 Appendix A Technical Specifications dealing with the surveillance frequency of fire detectors in the reactor buildings.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

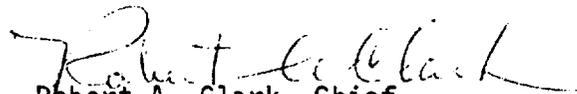
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

-2-

For further details with respect to this action, see (1) the licensee's application dated June 26, 1980, (2) Amendment No. 54 to License No. DPR-51 and Amendment No. 22 to License No. NPF-6, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Arkansas Tech University, Russellville, Arkansas. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of March 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing