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cc w/enclosures: See next page

- 1. Amendment No. 34 to DPR-51
- 2. Safety Evaluation
- 3. Notice

Enclosures:
 Robert W. Reid, Chief
 Operating Reactors Branch #4
 Division of Operating Reactors

Sincerely,

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications in response to your license amendment request dated January 20, 1978, and staff discussions. The amendment modifies the ANO-1 Technical Specifications dealing with the organization structure and deletes the requirements of the Technical Specifications concerning respiratory protection which are now stipulated in 10 CFR 20.103. Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

DISTRIBUTION: ✓
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Arkansas Power & Light Company
 ATTN: Mr. William Cavanaugh, III
 Executive Director, Generation
 and Construction Department
 P. O. Box 551
 Little Rock, Arkansas 72203

Docket No.: 50-313
 August 7, 1978

BGrimes

Arkansas Power & Light Company

cc w/enclosures:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated January 20, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

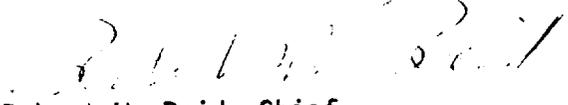
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 7, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Revise Appendix A Technical Specifications as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
ii	ii
117	117
119-127	119-127
129-134	129

Changes on the revised pages are identified by marginal lines.

<u>SECTION</u>	<u>TITLE</u>	<u>PAGE</u>
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4.17	FUEL HANDLING AREA VENTILATION SYSTEM SURVEILLANCE	110h
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4.20	FIRE SUPPRESSION WATER SYSTEM	110q
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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The Plant Manager shall be responsible for overall facility operation. In his absence, the Assistant Plant Manager of Power Plant shall assume all responsibility and perform all duties of the Plant Manager. If both the Plant Manager and his assistant are absent, these responsibilities and duties are assumed by the Operations Superintendent followed by the Technical Support Engineer.

6.2 ORGANIZATION

OFFSITE

- 6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2A and 6.2-2B and each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable position, except for the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for fire protection training shall be maintained and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 with the exception of frequency of training which shall be six times per year.

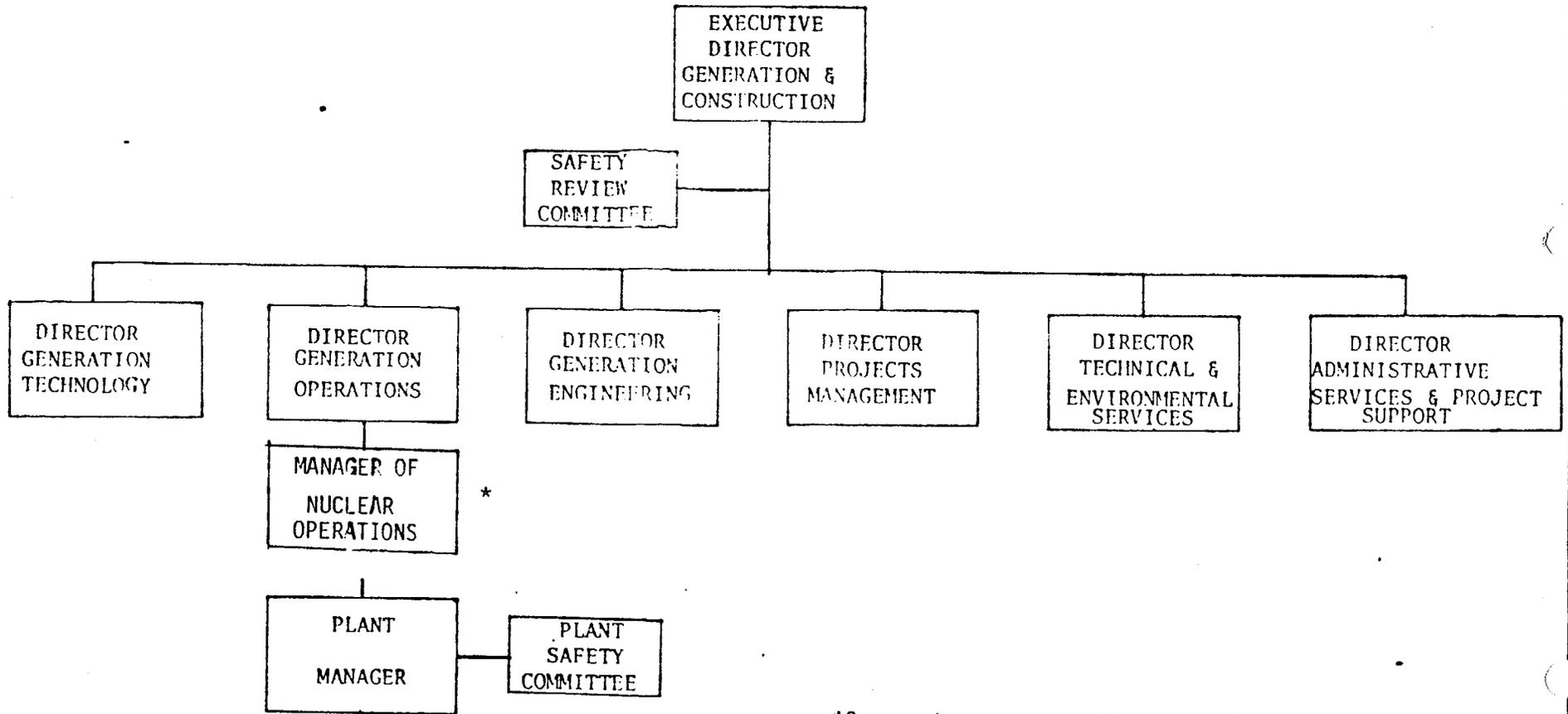
6.5 REVIEW AND AUDIT

6.5.1 Plant Safety Committee (PSC) Function

- 6.5.1.1 The Plant Safety Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

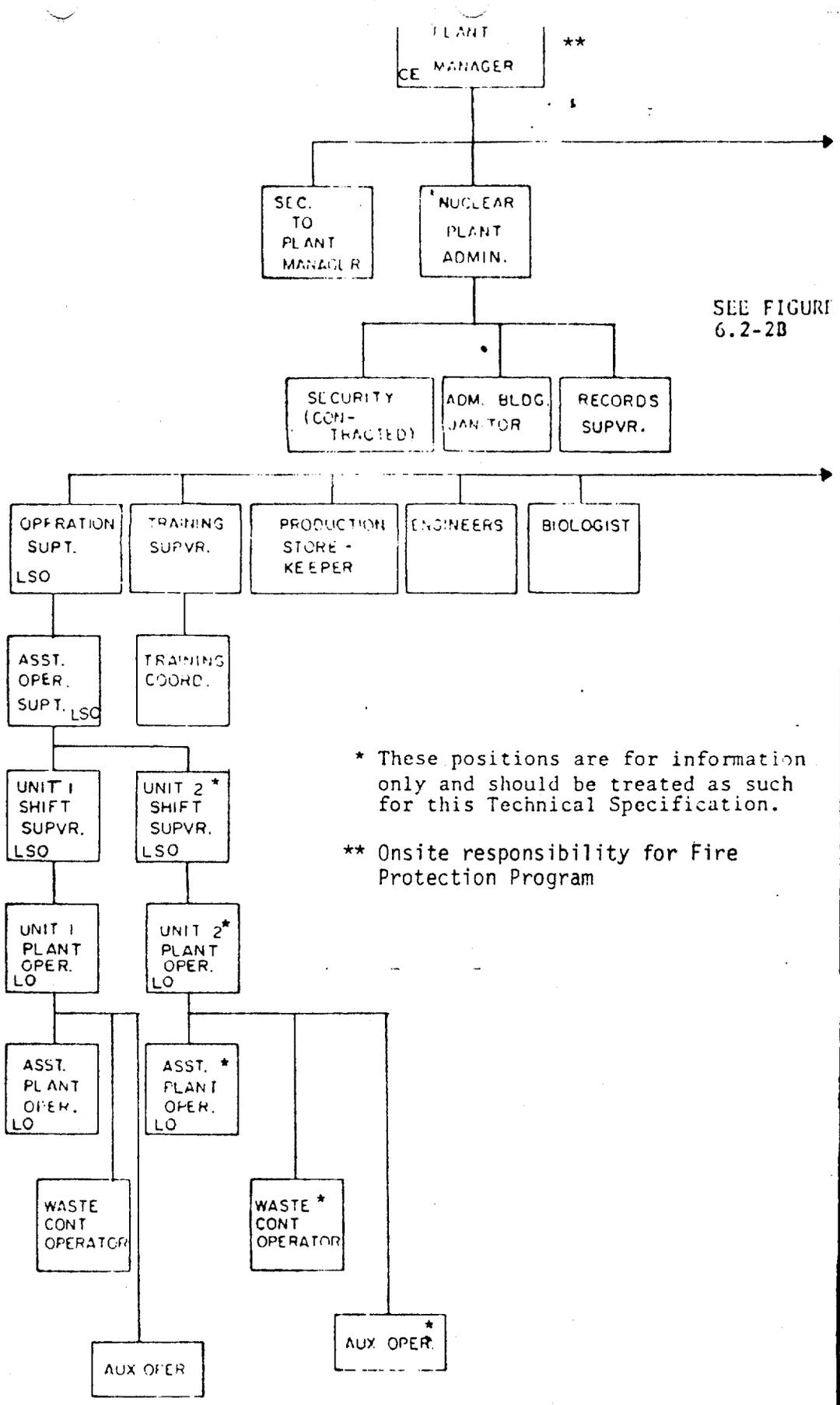
COMPOSITION

- 6.5.1.2 The Plant Safety Committee shall be composed of the:



*Corporate responsibility for Fire Protection Program

FIGURE 6.2-1



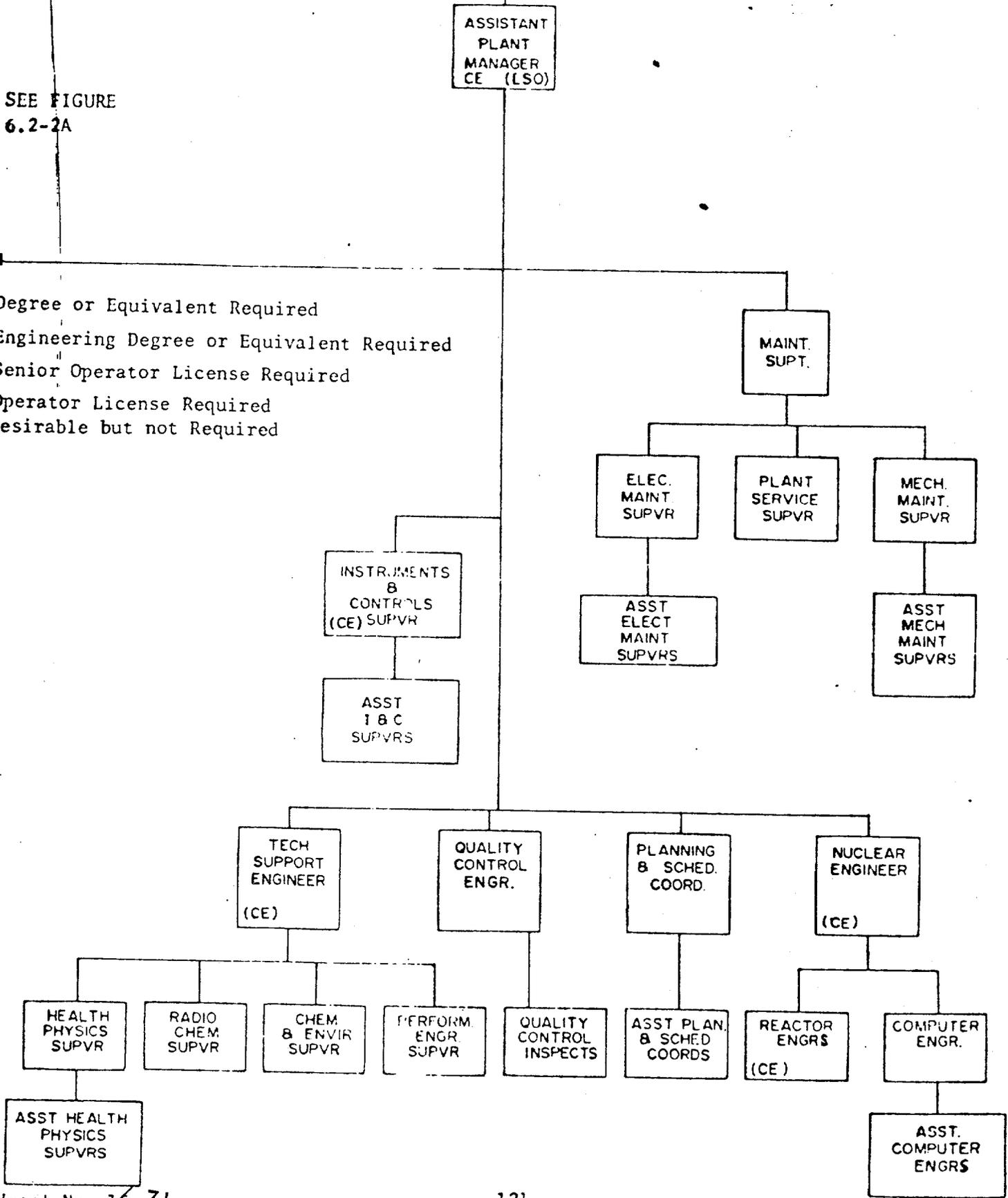
SEE FIGURE 6.2-2B

* These positions are for information only and should be treated as such for this Technical Specification.

** Onsite responsibility for Fire Protection Program

SEE FIGURE
6.2-2A

- C - Degree or Equivalent Required
- CE - Engineering Degree or Equivalent Required
- LSO - Senior Operator License Required
- LO - Operator License Required
-) - Desirable but not Required



Amendment No. 16, 34

Chairman: Assistant Plant Manager
Member: Technical Support Engineer
Member: Instrumentation & Controls Supervisor
Member: Operations Superintendent
Member: Maintenance Superintendent
Member: Nuclear Engineer
Member: Health Physics Supervisor

The Plant Manager shall appoint an acting chairman in the absence of the Assistant Plant Manager.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSC shall meet at least once per calendar month and as convened by the PSC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the PSC shall consist of the chairman and three members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The Plant Safety Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification 6.8 and revisions thereto, 2) any other proposed procedures or revisions thereto as determined by the Plant Manager to affect nuclear safety.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Appendix "A" Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant Manager.
 - f. Review of those Reportable Occurrences requiring 24 hour notification of the Commission.

- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews and investigations and reports thereon as requested by the Plant Manager.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Plant Manager.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Plant Manager.

AUTHORITY

6.5.1.7.1 The Plant Safety Committee shall:

- a. Recommend to the Plant Manager, written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (c) above constitutes an unreviewed safety question.

6.5.1.7.2 In the event of a disagreement between the recommendations of the Plant Safety Committee and the actions contemplated by the Plant Manager, the course determined by the Plant Manager to be more conservative will be followed. Records of the disagreement will be sent for review to the Director, Generation Operations, or Manager, Nuclear Operations and the Chairman of the Safety Review Committee by the Plant Manager on the next working day.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each meeting and copies shall be provided to the Chairman of the Safety Review Committee by the Plant Manager.

6.5.2 Safety Review Committee (SRC)

FUNCTION

- 6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry

- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. environmental considerations
- i. other appropriate fields required by the unique characteristics of the nuclear power plant

COMPOSITION

6.5.2.2 The SRC shall be composed of the:

Chairman: Director, Generation Operations
 Member: Director, Generation Technology
 Member: Director, Technical & Environmental Services
 Member: Manager of Safety
 Member: Arkansas Nuclear One Plant Manager
 Member: Manager, Technical Analysis
 Member: Arkansas Nuclear One Nuclear Engineer
 Member: Director, Generation Engineering
 Member: Radiation and Health Physics Consultant
 Member: Nuclear Safety Consultant

In his absence, the Chairman shall appoint an Acting Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or his designated alternate and four members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Reportable occurrences requiring 24 hour notification to the Commission.
- h. Reports and meeting minutes of the PSC.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance and retraining of all members of the plant management and operations staff, and the performance, training, and qualifications of new members of the entire plant staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The Facility Emergency Plan and implementing procedures at least once per two years.
- e. The Facility Fire Protection Program and implementing procedures at least once per 24 months.

- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the SRC or the Executive Director, Generation & Construction (EDG&C).

6.5.2.9 Special Inspections and Audits

- A. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

AUTHORITY

- 6.5.2.10 The SRC shall report to and advise the Executive Director, Generation & Construction (EDG&C) within 30 days following each meeting.

RECORDS

- 6.5.2.11 Records of SRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Executive Director, Generation & Construction (EDG&C) within 30 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Executive Director, Generation & Construction (EDG&C) within 30 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Director, Generation & Construction (EDG&C) and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for Reportable Occurrences:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.12.
- b. Each Reportable Occurrence requiring 24 hour notification to the Commission shall be reviewed by the PSC and submitted to the SRC and the Manager, Nuclear Operations by the Plant Manager.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least hot shutdown within one hour.
- b. The Nuclear Regulatory Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.36 and Specification 6.12.3.1

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PSC and approved by the Plant Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PSC and approved by the Plant Manager - within 14 days of implementation.

- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by Section 17 of the Quality Assurance Manual for Operations.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PSC and the SRC.

6.10 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.11 DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-51

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE, UNIT NO. 1

DOCKET NO. 50-313

Introduction

By letter dated January 20, 1978, Arkansas Power and Light Company (the licensee or AP&L) requested amendment of the Technical Specifications appended to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The change would reflect a change in the licensee's organization structure.

On November 29, 1976, the Commission published in the Federal Register an amended Section 20.103 of 10 CFR 20, which became effective on December 29, 1976. One effect of this revision was that in order to receive credit for limiting the inhalation of airborne radioactive material, respiratory protective equipment must be used as stipulated in Regulatory Guide 8.15. Another requirement of the amended regulation is that licensees authorized to make allowance for use of respiratory protective equipment prior to December 29, 1976, must bring the use of their respiratory protective equipment into conformance with Regulatory Guide 8.15 by December 29, 1977.

The licensee is presently authorized by the Technical Specifications for ANO-1 to make allowance for use of respiratory protective equipment at that facility. The present program, however, differs from that stipulated in Regulatory Guide 8.15. Accordingly, by letter dated August 12, 1977, we advised the licensee that if they desired to receive credit for use of respiratory protective equipment at ANO-1 after December 28, 1977, such use must be as stipulated in Regulatory Guide 8.15 rather than as specified in the current Technical Specifications.

Also, based on the revocation provision of the current Technical Specifications on respiratory protection and in the absence of written objections from the licensee, we advised the licensee that we would delete this specification in an amendment of the Technical Specifications for ANO-1 after December 28, 1977. No written objections have been received from the licensee.

Evaluation

The evaluations relating to the proposed organization change and the deletion of the Respiratory Protection Program are discussed separately below.

A. Proposed Organization Changes

We have reviewed the proposed changes to the ANO-1 Administrative Controls Section of the Technical Specifications. This change involves a reorganization in the Generation and Construction Department and changes to the ANO-1 plant organization. As a consequence of this reorganization, changes have also been proposed to the Plant Safety Committee (PSC) and to the Safety Review Committee (SRC). Additional information related to these changes used in our review is contained in Amendments 44 and 45 to Section 13.1 of AP&L's FSAR for Arkansas One, Unit 2.

We have evaluated these changes in terms of the relevant portions of Regulatory Guide 1.8 and 1.33 and find the proposed changes provide no decrease in safety margin and are acceptable.

B. Deletion of Respiratory Protection

Deletion of current Technical Specification requirements, Section 6.11, regarding respiratory protection is necessary to eliminate conflict with 10 CFR §20.103, as revised November 29, 1976. This agrees with the revocation provision in Section 6.11.3 of the current Technical Specifications which requires that Section 6.11 be revoked upon adoption of the proposed change to 10 CFR §20.103. In the future, as specified in the regulations, allowance may be made for the use of respiratory protective equipment only if its use is as stipulated in Regulatory Guide 8.15, Acceptable Programs for Respiratory Protection. Based on the above, we find this change acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 7, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-313ARKANSAS POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 34 to Facility Operating License No. DPR-51, issued to Arkansas Power & Light Company (AP&L or the licensee), which revised the Technical Specifications for operation of Arkansas Nuclear One, Unit No. 1 (ANO-1 or the facility) located in Pope County, Arkansas. The amendment is effective as of its date of issuance.

This amendment modifies the ANO-1 Technical Specifications dealing with the organization structure and deletes the requirements of the Technical Specifications concerning respiratory protection which are now stipulated in 10 CFR 20.103.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter 1, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the licensee's application for amendment dated January 20, 1978, (2) Amendment No. 34 to License No. DPR-51, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Arkansas Polytechnic College, Russellville, Arkansas. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 7th day of August 1978.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors