

September 22, 1992

Docket No. 50-346

Mr. Donald C. Shelton
Vice President, Nuclear - Davis-Besse
Centerior Service Company
c/o Toledo Edison Company
300 Madison Avenue
Toledo, Ohio 43652

Dear Mr. Shelton:

SUBJECT: AMENDMENT NO.174 TO FACILITY OPERATING LICENSE NO. NPF-3
(TAC NO. M82089)

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The Commission has issued Amendment No.174 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This action amends the license and revises the Technical Specifications (TS) in response to your application dated November 1, 1991, as supplemented December 26, 1991 and June 1, 1992.

This amendment revises the TS by removing the Fire Protection Program, Sections 3/4.3.3, Fire Detection Instrumentation, 3/4.7.9, Fire Suppression Systems and Bases, and 3/4.7.10, Fire Barriers and Bases, and by revising Fire Protection Program Administrative Controls, Sections 6.2.2.f, Facility Staff, 6.4.2, Training, 6.5.1.6, Responsibilities, and 6.9.2, Special Reports, and by modifying Operating License Condition 2.C.(4), Fire Protection.

A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely, **Original Signed By:**
J. B. Hopkins

Jon B. Hopkins, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.174 to License No. NPF-3
2. Safety Evaluation

cc: See next page

*See Previous Concurrence

LA/PDIII-3/DRPW
PKreutzer
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JBH for
PM/PDIII-3/DRPW
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9/22/92

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 22, 1992

Docket No. 50-346

Mr. Donald C. Shelton
Vice President, Nuclear - Davis-Besse
Centerior Service Company
c/o Toledo Edison Company
300 Madison Avenue
Toledo, Ohio 43652

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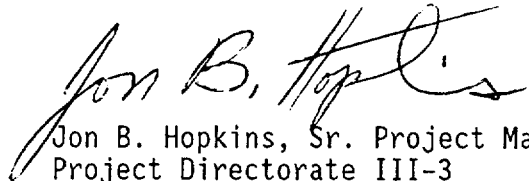
SUBJECT: AMENDMENT NO. 174 TO FACILITY OPERATING LICENSE NO. NPF-3
(TAC NO. M82089)

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This amendment revises the TS by removing the Fire Protection Program, Sections 3/4.3.3, Fire Detection Instrumentation, 3/4.7.9, Fire Suppression Systems and Bases, and 3/4.7.10, Fire Barriers and Bases, and by revising Fire Protection Program Administrative Controls, Sections 6.2.2.f, Facility Staff, 6.4.2, Training, 6.5.1.6, Responsibilities, and 6.9.2, Special Reports, and by modifying Operating License Condition 2.C.(4), Fire Protection.

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Sincerely,


Jon B. Hopkins, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 174 to License No. NPF-3
2. Safety Evaluation

cc: See next page

Mr. Donald C. Shelton
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Davis-Besse Nuclear Power Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

CENTERIOR SERVICE COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 174
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company, Centerior Service Company, and the Cleveland Electric Illuminating Company (the licensees) dated November 1, 1991 as supplemented December 26, 1991, and June 1, 1992 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraphs 2.C.(2)(a) and 2.C.(4) of Facility Operating License No. NPF-3 to read as follows:

(a) Technical Specifications

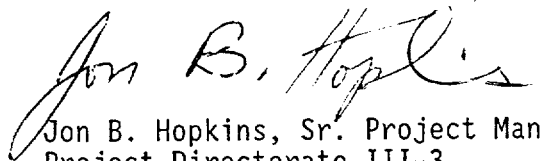
The Technical Specifications contained in Appendix A, as revised through Amendment No. 174, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

2.C(4) Toledo Edison shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Safety Analysis Report and as approved in the SERs dated July 26, 1979, and May 30, 1991, subject to the following provisions:

Toledo Edison may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 90 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jon B. Hopkins, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachments: 1. Amended page 8
to License No. NPF-3
2. Changes to the Technical
Specifications

Date of issuance: September 22, 1992

ATTACHMENT TO LICENSE AMENDMENT NO.174

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Remove</u>	<u>Insert</u>
Index IV	Index IV
Index VII	Index VII
Index XII	Index XII
3/4 3-51	3/4 3-51
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3/4 7-37 through 3/4 7-48	3/4 7-37
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6-1a	6-1a
6-5	6-5
6-7	6-7
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2.C.(4) Fire Protection

Toledo Edison shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Safety Analysis Report and as approved in the SERs dated July 26, 1979, and May 30, 1991, subject to the following provision:

Toledo Edison may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

2.C.(5) Toledo Edison Company shall maintain in effect and implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. The program shall include:

- (a) Identification of a sampling schedule for the critical parameters and control points for these parameters;
- (b) Identification of the procedures used to quantify parameters that are critical to control points;
- (c) Identification of process sampling points;
- (d) Procedure for the recording and management of data;
- (e) Procedures defining corrective actions for off control point chemistry conditions; and
- (f) A procedure identifying the authority responsible for the interpretation of the data, and the sequence and timing of administrative events required to initiate corrective action.

2.C.(6) Antitrust Conditions

Centerior Service Company shall comply with the antitrust conditions delineated in Condition 2.E of this license as if named therein. Toledo Edison Company is responsible and accountable for the actions of Centerior Service Company to the extent that Centerior Service Company's actions contravene the antitrust license conditions of Condition 2.E of this license.

2.D The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Davis-Besse Nuclear Power Station Physical Security Plan," with revisions submitted through January 29, 1988; "Davis-Besse Nuclear Power Station Guard Training and Qualification Plan," with revisions submitted through February 20, 1987; and "Davis-Besse Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through February 20, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

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Pages 3/4 3-51 through 3/4 3-56 deleted. Next page is 3/4 3-57.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

1. With a half-life greater than 30 days (excluding Hydrogen 3) and
 2. In any form other than gas.
- b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources and fission detectors transferred without a certificate indicating the last test date shall be tested prior to being placed into use.
- c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source.
- 4.7.8.1.3 Reports - A report shall be prepared and submitted to the Commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of ≥ 0.005 microcuries of removable contamination.

3/4.3 INSTRUMENTATION

BASES

REMOTE SHUTDOWN INSTRUMENTATION (Continued)

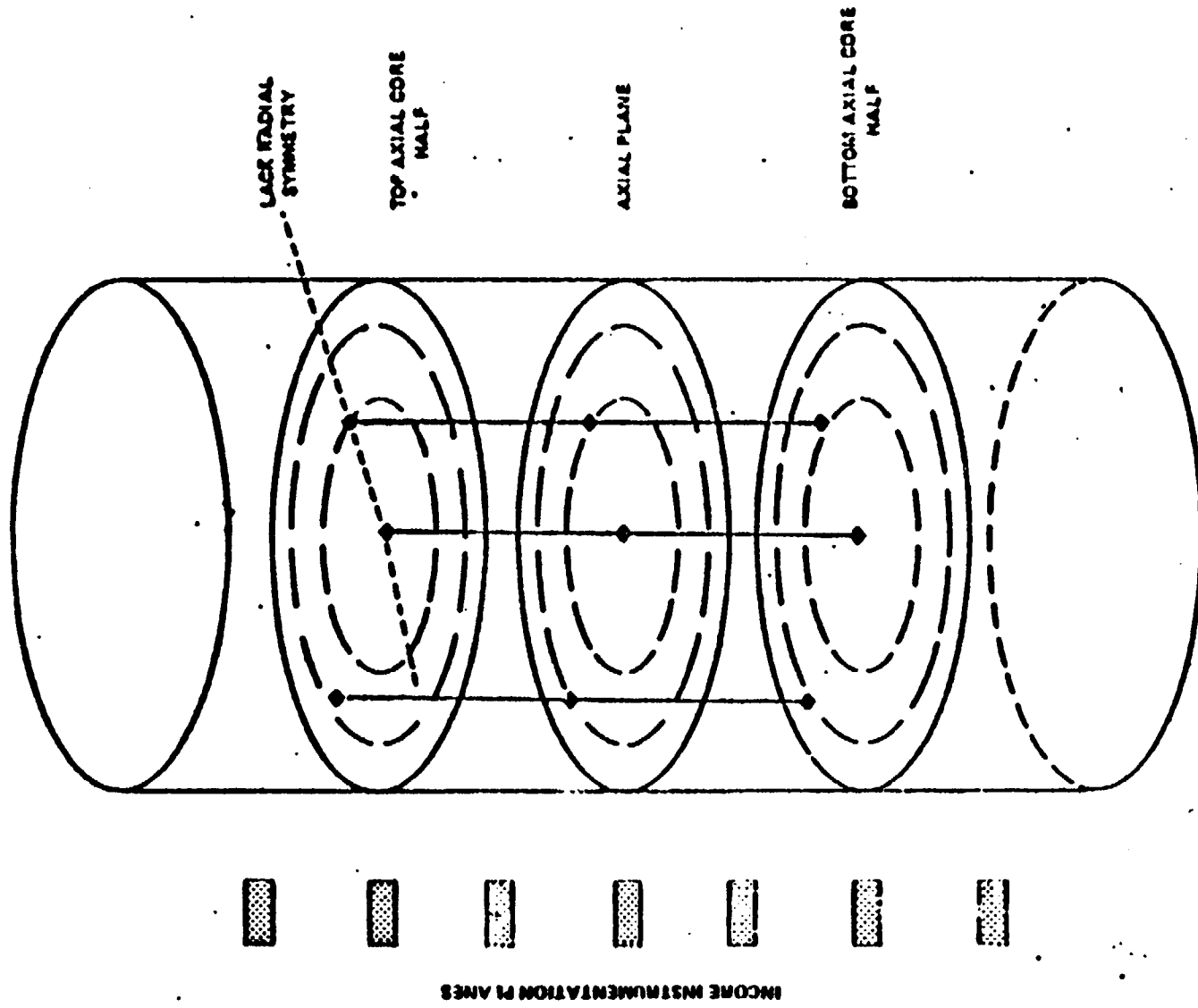
HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost.

3/4.3.3.6 POST-ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the post-accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. The containment Hydrogen Analyzers, although they are considered part of the plant post-accident monitoring instrumentation, have their OPERABILITY requirements located in Specification 3/4.6.4.1, Hydrogen Analyzers.

3/4.3.3.7 CHLORINE DETECTION SYSTEMS - Deleted

3/4.3.3.8 FIRE DETECTION INSTRUMENTATION - Deleted



Based Figure 3-1 In-core Instrumentation Specification
Acceptable Minimum AXIAL POWER IMBALANCE Arrangement

PLANT SYSTEMS

BASES

3/4.7.8 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from by-product, source, and special nuclear material sources will not exceed allowable intake values.

3/4.7.9 FIRE SUPPRESSION SYSTEMS -- DELETED

3/4.7.10 FIRE BARRIERS -- DELETED

6.0 ADMINISTRATIVE CONTROLS

6.2.2 FACILITY STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control panel area when fuel is in the reactor.
- c. At least two licensed Operators, one of which has a Senior Reactor Operator license, shall be present in the control room while in MODES 1, 2, 3, or 4.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor[#].
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. Deleted
- g. The Manager-Plant Operations shall either hold or have held a senior reactor operator's license on a pressurized water reactor. The Operations Superintendent shall hold a senior reactor operator license for the Davis-Besse Nuclear Power Station.

[#] The individual qualified in radiation protection procedures may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence, provided immediate action is taken to fill the required position.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Manager - Radiological Control who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents, and (3) the Manager - Plant Operations whose requirement for a senior reactor operator license is as stated in Specification 6.2.2.g.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Manager - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and of 10 CFR 55.59.

6.4.2 Deleted

6.5 REVIEW AND AUDIT

6.5.1 STATION REVIEW BOARD (SRB)

FUNCTION

6.5.1.1 The Station Review Board (SRB) shall function to advise the Plant Manager on all matters related to nuclear safety.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.1.2 The Station Review Board (SRB) shall be composed of at least six members of the Davis-Besse onsite management organization. The members shall be as a minimum, managers or individuals reporting directly to managers from each of the following disciplines: plant operations, maintenance, planning, radiological controls, engineering, and quality assurance. The members shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.4, or 4.6 for applicable required experience.

The SRB Chairman shall be drawn from the SRB members and designated in writing by the Plant Manager.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SRB Chairman; however, no more than two alternates shall participate as voting members in SRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SRB shall meet at least once per calendar month and as convened by the SRB Chairman or his designee.

QUORUM

6.5.1.5 A quorum of the SRB shall consist of the Chairman or his designee and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Review Board shall be responsible for:

- a. Review of plant administrative procedures and changes thereto.
- b. Review of the safety evaluation for 1) procedures, 2) changes to procedures, equipment or systems, and 3) tests or experiments completed under the provisions of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question.
- c. Review of proposed procedures and changes to procedures and equipment determined to involve an unreviewed safety question as defined in 10 CFR 50.59.

ADMINISTRATIVE CONTROLS

- d. Review of proposed tests or experiments determined to involve an unreviewed safety question as defined in 10 CFR 50.59.
- e. Review of reports of violations of codes, regulations, orders, Technical Specifications, or Operating License requirements having nuclear safety significance or reports of abnormal degradation of systems designed to contain radioactive material.
- f. Review of all proposed changes to the Technical Specifications or the Operating License.
- g. Deleted
- h. Review of reports of significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect plant safety.
- i. Review of the Industrial Security Plan, the Security Training and Qualification Plan, and the Security Contingency Plan, and changes thereto.
- j. Review of the Davis-Besse Emergency Plan and changes thereto.
- k. Review of items which may constitute potential nuclear safety hazards as identified during review of facility operations.
- l. Investigations or analyses of special subjects as requested by the Company Nuclear Review Board.
- m. Review of all REPORTABLE EVENTS.
- n. Review of all Safety Limit Violation Reports (Section 6.7).
- o. Review of any unplanned, accidental or uncontrolled radioactive releases, evaluation of the event, ensurance that remedial action is identified to prevent recurrence, review of a report covering the evaluation and forwarding of the report to the Plant Manager and to the CNRB.
- p. Review of the changes to the OFFSITE DOSE CALCULATION MANUAL.
- q. Review of the changes to the PROCESS CONTROL PROGRAM.
- r. Review of the Annual Radiological Environmental Operating Report.
- s. Review of the Semiannual Radioactive Effluent Release Report.
- T. Review of the Fire Protection Program and changes thereto.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Station Review Board shall:

- a. Recommend to the Plant Manager written approval or disapproval of items considered under Sections 6.5.1.6 a, c, d, f, i and j.
- b. Render determinations in writing with regard to whether or not each item considered under Sections 6.5.1.6 a, c, d and f above constitutes an unreviewed safety question as defined in 10 CFR 50.59.
- c. Provide written notification within 24 hours to the Vice President, Nuclear and the Company Nuclear Review Board of disagreement between the SRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.
- d. Make recommendations in writing to the Plant Manager that actions reviewed under Section 6.5.1.6 b above do not constitute an unreviewed safety question.

RECORDS

6.5.1.8 The Station Review Board shall maintain written minutes of each meeting and copies shall be provided to the Plant Manager, Vice President, Nuclear and Chairman of the Company Nuclear Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

FUNCTION

6.5.2.1 The Company Nuclear Review Board (CNRB) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the U.S. Nuclear Regulatory Commission in accordance with 10 CFR 50.4 within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specifications:

- a. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- b. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. Seismic event analysis, Specification 4.3.3.3.2.
- e. Deleted
- f. Deleted

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.

ADMINISTRATIVE CONTROLS

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient of operational cycles for those facility components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SRB and the CNRB.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analyses at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- n. Records of the service lives of all safety related hydraulic and mechanical snubbers including the date at which the service life commences and associated installation and maintenance records.
- p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 174 TO FACILITY OPERATING LICENSE NO. NPF-3
TOLEDO EDISON COMPANY
CENTERIOR SERVICE COMPANY
AND
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1
DOCKET NO. 50-346

1.0 INTRODUCTION

By letter dated November 1, 1991 and supplemented December 26, 1991, and June 1, 1992 the Toledo Edison Company (the licensee) proposed that the existing license condition on fire protection be replaced with the standard condition noted in Generic Letter 86-10, and also proposed changes to the Appendix A Technical Specifications (TS) for Davis-Besse. The proposed changes would remove detailed requirements for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements as recommended by Generic Letter 86-10. The proposed change would also modify the administrative control requirements of the TS to add requirements for the Fire Protection Program that are similar to requirements for other programs implemented by license conditions. Guidance on these proposed changes to TS was provided to all power reactor licensees and applicants by Generic Letter 88-12, dated August 2, 1988. The supplemental letter confirmed the installation of fire detection equipment and requested a 90 day implementation period and did not alter the proposed action or affect the determination published December 11, 1991.

2.0 EVALUATION

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modifications of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by Generic Letter 86-10 to incorporate the final NRC-approved Fire Protection Program in their Updated Safety Analysis Reports (USAR). In this manner, the Fire Protection Program -- including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection -- would have a status consistent

with that of other plant features described in the USAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the USAR, should be used to ensure uniform enforcement of fire protection requirements. Finally, the Commission stated that, with the requested actions, licensees may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from TS. Additionally, in the licensing review of new plants, the staff has approved applicant requests to remove fire protection requirements from TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in TS can be modified without affecting nuclear safety. Such modification can be made provided that there are suitable administrative controls over these changes. These details, which are presently included in TS and that are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the Fire Protection Program including those technical and administrative requirements removed from the TS to ensure that nuclear safety is not adversely affected. These controls include: (1) the TS administrative controls that are applicable to the Fire Protection Program; (2) the license condition on implementation of, and subsequent change to, the Fire Protection Program; and (3) the 10 CFR 50.59 criteria for evaluating changes to the Fire Protection Program as described in the USAR. The Davis-Besse Fire Hazards Analysis Report (FHAR) has been incorporated by reference into the USAR, Section 9.5.1, "Fire Protection System," and the Fire Protection Program requirements have been incorporated into the FHAR Section 8.0, "Operating Specifications."

The specific details relating to fire protection requirements removed from TS by this amendment include those specifications for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The administrative controls requirements have been modified to include Fire Protection Program implementation as an element for which written procedures must be established, implemented, and maintained. In addition, the review responsibilities of the Station Review Board were expanded to include the review of the Fire Protection Program and changes thereto.

The TS changes proposed by the licensee are in accordance with the guidance provided by Generic Letter 88-12, as addressed in the items below.

- (1) Specification 6.5.1.6, "Station Review Board Responsibilities," was revised to add the review of the Fire Protection Program and changes thereto.
- (2) Specification 6.8, "Procedures and Programs," already includes the requirement for written Fire Protection Program procedures to be established, implemented, and maintained.
- (3) Specification 3.3.3.8, "Fire Detection Instrumentation," its associated Surveillance Requirements, and Bases were removed.
- (4) Specification 3/4.7.9, "Fire Suppression Systems," the associated Surveillance Requirements, and Bases were removed.
- (5) Specification 3/4.7.10, "Fire Barriers," its associated Surveillance Requirements, and Bases were removed.
- (6) Specification 6.2.2.7 on fire brigade staffing requirements was removed.

As required by Generic Letter 86-10, the licensee confirmed that the NRC-approved Fire Protection Program has been incorporated into the USAR Section 9.5.1. Also, the licensee has proposed that the existing licensing condition 2.C.(4) on the Fire Protection Program be replaced with the standard condition noted in Generic Letter 86-10.

In the letter dated November 1, 1991, the licensee committed to inform the NRC when the installation of additional area-type detection in the fuel handling area (Room 300) was complete. The installation of additional area-type detection in Room 300 has been completed and the procedures have been revised to reflect the installation of new detectors. The FHAR, which is now part of the USAR, will be revised during the 1992 annual update to reflect the installation of the new detectors.

According to the guidance of Generic Letter 88-12, fire protection TS related to safe shutdown following a fire should not be included in the removal of detailed fire protection requirements from TS. The licensee's submittal met this guidance, only in that there were no existing TS related to safe shutdown following a fire. Following discussions with the NRC staff, the licensee, by letter dated July 28, 1992, committed to include testing of transfer switches used to meet 10 CFR Part 50 Appendix R safe shutdown requirements, and will submit a license amendment application to include the transfer switch testing in the TS. The licensee has committed to submit the application by December 31, 1992. The NRC staff finds this to be an appropriate action.

The licensee confirmed that the operational conditions, remedial actions, and test requirements associated with the removed fire protection TS have been

included in the Fire Protection Program incorporated into the USAR. This is in accordance with the guidance of Generic Letter 88-12.

On the basis of its review of the above items, the staff concludes that the licensee has met the guidance of Generic Letter 88-12. Therefore, the staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (56 FR 64662). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also involves changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

On the basis of the considerations discussed above, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Dunning
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Date: September 22, 1992