

Docket No. 50-313

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MAY 27 1977

Arkansas Power & Light Company
ATTN: Mr. J. D. Phillips
Senior Vice President
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Engineering
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Gentlemen:

The Commission has issued the enclosed Amendment No. 24 to Facility Operating License No. DPR-51 for Arkansas Nuclear One - Unit No. 1. This amendment revises the provisions of the Technical Specifications in response to your application of November 22, 1976.

The amendment to the Technical Specifications will add provisions for steam generator tube inspection which are consistent with the guidance contained in Regulatory Guide 1.83, Revision 1, dated July 1975. Certain changes to your proposal were necessary to comply with our regulatory position. These changes were discussed with and agreed to by your staff.

Copies of our related Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. Amendment No. 24 to License No. DPR-51
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

OFFICE →	DOR:ORB-2	DOR:ORB-2	DOR:ORB-4	OELD	DOR:ORB-2	DOR:EB/OT
SURNAME →	RMDiggs	RPSnaider:esp	MFairtile	DMstead	DKDavis	LShao
DATE →	4/20/77	4/20/77	4/26/77	5/16/77	4/27/77	5/29/77

MAY 27 1977

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DATE >					



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 24
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated November 22, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.c(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 24, are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 27, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 24

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Add the following new pages to the Appendix A portion of the Technical Specifications:

110j
110k
110l
110m
110n
110o
142a

Replace existing page ii of the Table of Contents and page 142 of the Appendix A portion of the Technical Specifications with the attached revised pages ii and 142. The changed area on the revised pages is identified by a marginal line.

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4.18 STEAM GENERATOR TUBING SURVEILLANCE

Applicability

Applies to the surveillance of tubing of each steam generator.

Objective

To define the inservice surveillance program for steam generator tubing.

Specification

4.18.1 Baseline Inspection

The first steam generator tubing inspection performed according to Specifications 4.18.2 and 4.18.3 shall be considered as constituting the baseline condition for subsequent inspections.

4.18.2 Examination Methods

Inservice inspection of steam generator tubing shall include non-destructive examination by eddy-current testing or other equivalent techniques. The inspection equipment shall provide a sensitivity that will detect defects with a penetration of 20 percent or more of the minimum allowable as-manufactured tube wall thickness.

4.18.3 Selection and Testing

The steam generator sample size is specified in Table 4.18.1. The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.18.2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.18.4 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.18.5. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in both steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of of the tubes inspected shall be from these critical areas.
- b. The first sample inspection during each inservice inspection (subsequent to the baseline inspection) of each steam generator shall include:
 1. All nonplugged tubes that previously had detectable wall penetrations (>20%), and
 2. Tubes in those areas where experience has indicated potential problems.

- c. The second and third sample inspections during each inservice inspection as required by Table 4.18-2 may be less than a full tube inspection by concentrating the inspection on those areas of the tube sheet array and on those portions of the tubes where tubes with imperfections were previously found.
- d. The tube inspection shall be performed according to 4.18.5a.8. If the entire length of a tube does not permit passage of the eddy current inspection probe, this shall be recorded and an adjacent tube shall be selected.

The results of each sample inspection shall be classified into one of the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.

4.18.4 Inspection Intervals

The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The baseline inspection shall be performed during the first refueling shutdown. Subsequent inservice inspections shall be performed at intervals of not less than 10 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under all volatile treatment (AVT) conditions result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of 40 months.
- b. If the results of the inservice inspection of a steam generator performed in accordance with Table 4.18.2 at 40 month intervals fall in Category C-3, subsequent inservice inspections shall be performed at intervals of not less than 10 nor more than 20 calendar months after the previous inspection. The increase in inspection frequency shall apply until a subsequent inspection meets the conditions specified in 4.18.4.a and the interval can be extended to 40 months.

c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.18.2 during the shutdown subsequent to any of the following conditions.

1. Primary-to-secondary leakage in excess of the limits of Specification 3.10 (inservice inspection not required if leaks originate from tube-to-tube sheet welds),
2. A seismic occurrence greater than the Operating Basis Earthquake,
3. A loss-of-coolant accident requiring actuation of the engineered safeguards, or
4. A main steam line or feedwater line break.

4.18.5 Acceptance Criteria

a. As used in this specification:

1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.
2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either the inside or outside of a tube.
3. Degraded Tube means a tube containing imperfections $\geq 20\%$ of the nominal wall thickness caused by degradation.
4. % Degradation means the percentage of the tube wall thickness affected or removed by degradation.
5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective.
6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection; it is equal to 40% of the nominal tube wall thickness.
7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in Specification 4.18.4.c.

8. Tube Inspection means an inspection of the steam generator tube from the point of entry completely to the point of exit.
 - b. The steam generator shall be determined operable after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.18.2.

4.18.6 Reports

Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the NRC per Specification 6.12.3. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the NRC shall be reported per Specification 6.12.3.

Bases

The surveillance requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

TABLE 4.18-1

MINIMUM NUMBER OF STEAM GENERATORS TO BE
INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	No
No. of Steam Generators per Unit	Two
First Inservice Inspection	Two
Second & Subsequent Inservice Inspections	One ¹

Table Notation:

1. The inservice inspection may be limited to one steam generator on alternating schedule encompassing $3N\%$ of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions.

TABLE 4.18-2

STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S.G	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect addi- tional 2S tubes in this S.G. Notify NRC pursuant to Specification 6.12.3	C-1	None	N/A	N/A
			C-2	(II) Plug defective tubes and in- spect additional 4S tubes in this S.G. Notify NRC pur- suant to Specifi- cation 6.12.3	C-1	None
					C-2	Plug defective tubes Notify NRC pursuant to Specification 6.12.3
			C-3	Perform action for C-3 result of first sample (See I)		
	C-3	Perform action for C-3 result of first sample(See I)	N/A	N/A		
	C-3	(I) Inspect all tubes in this S.G., plug defective tubes and inspect 2S tubes in the other S.G. Prompt notification to NRC pursuant to specification 6.12.3	The other S.G. is C-1	None	N/A	N/A
			Other S.G. is C-2	Perform action for C-2 result of second sample (See II)	N/A	N/A
			Other S.G. is C-3	Inspect all tubes in each S.G. and plug defective tubes. Prompt notification to NRC pursuant to specification 6.12.3	N/A	N/A

$S=3 \frac{N}{n} \%$ Where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection.

The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

- (d) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.
- (e) The complete results of the steam generator tube inservice inspection. The results shall include:
 - 1. Number and extent of tubes inspected,
 - 2. Location and percent of wall-thickness penetration for each indication of an imperfection,
 - 3. Identification of tubes plugged.

6.12.2.3 Monthly Operating Report

Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, with a copy to the appropriate Regional Office, by the tenth of each month following the calendar month covered by the report.

6.12.3 Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC as required below. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

6.12.3.1 Prompt Notification With Written Followup

The types of events listed below shall be reported as expeditiously as possible, but within 24 hours, by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. A copy of the confirmation and the written followup report shall also be sent to the Director, Office of Management Information and Program Control, USNRC. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (a) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the Technical Specifications or failure to complete the required protective function.

NOTE:

Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items (e), (f), or 6.12.3.2(a).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. DPR-51

ARKANSAS POWER & LIGHT COMPANY

ARKANSAS NUCLEAR ONE - UNIT 1

DOCKET NO. 50-313

Introduction

By letter dated January 10, 1975, Arkansas Power & Light Company (AP&L) responded to our October 29, 1974 initial request for technical specifications governing steam generator tube surveillance at Arkansas Nuclear One - Unit No. 1 (ANO-1). By letter dated September 9, 1976, we forwarded recommended revised technical specifications based upon revised Regulatory Guide 1.83, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes". AP&L answered this request by letter dated November 22, 1976, which forwarded revised technical specifications. The proposed change would add technical specification provisions for steam generator tube inspection to be consistent with the guidance contained in Regulatory Guide 1.83, Revision 1, dated July 1975.

Modifications to the proposed amendment were necessary to assure compliance with our regulatory position. These modifications were discussed with and agreed to by the AP&L staff.

Evaluation

Structures, systems, and components important to safety of a nuclear power plant are designed, fabricated, constructed, and tested so as to provide reasonable assurance that the facility can be operated without undue risk to the health and safety of the public. To continuously maintain such assurance, General Design Criterion 32 requires that components which are part of the reactor coolant pressure boundary be designed to permit periodic inspection and testing of important areas and features to assess their structural and leaktight integrity. The steam generator tubing is part of the reactor coolant system pressure boundary and is an important part of a major barrier against fission product release to the environment. It also acts as a barrier against

steam release to the containment in the event of a loss-of-coolant accident (LOCA). To act as an effective barrier, this tubing must be free of cracks, perforations, and general deterioration. For this reason, a program of periodic inservice inspection is being established to assure the continued integrity of the steam generator tubes over the service life of the plant.

Generally, the major elements of the proposed steam generator tube inservice inspection program for ANO-1 consist of specified: (a) sample selection, (b) examination methods, (c) inspection intervals, (d) acceptance criteria, and (e) reporting requirements. Each of these major elements of the program is separately evaluated below.

(a) Sample Selection

The proposed sampling scheme is generally patterned after Regulatory Guide 1.83, Revision 1, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes". However, there are some deviations from Regulatory Guide 1.83 that we require to improve the program and/or reduce the potential radiation exposure of personnel that must perform the inspections. The principal deviations from Regulatory Guide 1.83 supplementary sampling requirements are evaluated below:

- (i) Regulatory Position C.5.a, "Supplementary Sampling Requirements" recommends that if the eddy current inspection results during an inservice inspection indicate any tubes with previously undetected imperfections of 20% or greater depth, additional steam generators, if any, should be inspected. In other words, because of a single tube in one steam generator with previously undetected imperfection of 20% or greater depth but still well below the plugging limit, all steam generators in the plant should be inspected. Although the detection of any defect warrants further inspection to determine the extent of degradation in the steam generators, we believe that this inspection should be expanded initially to determine the extent of any further degradation in the steam generator under inspection. If the expanded inspection indicates more extensive defect conditions, then expansion to the other steam generator is required. This approach will provide careful stepwise expansion of inspection based on the results of successive steps, while tending to minimize the exposure of inspection personnel resulting from initial positioning of inspection equipment in a steam generator. This inspection approach is appropriate for this facility in which system characteristics are such that all steam generators are expected to perform in a similar manner.

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- (i) Regulatory Position C.5.a, "Supplementary Sampling Requirements" recommends that if the eddy current inspection results during an inservice inspection indicate any tubes with previously undetected imperfections of 20% or greater depth, additional steam generators, if any, should be inspected. In other words, because of a single tube in one steam generator with previously undetected imperfection of 20% or greater depth but still well below the plugging limit, all steam generators in the plant should be inspected. Although the detection of any defect warrants further inspection to determine the extent of degradation in the steam generators, we believe that this inspection should be expanded initially to determine the extent of any further degradation in the steam generator under inspection. If the expanded inspection indicates more extensive defect conditions, then expansion to the other steam generator is required. This approach will provide careful stepwise expansion of inspection based on the results of successive steps, while tending to minimize the exposure of inspection personnel resulting from initial positioning of inspection equipment in a steam generator. This inspection approach is appropriate for this facility in which system characteristics are such that all steam generators are expected to perform in a similar manner.

- (ii) Regulatory Guide 1.83, Revision 1 requires additional inspections if the initial inspection results indicate that more than 10% of the inspected tubes have detectable wall penetration of greater than 20% or that one or more tubes inspected have an indication in excess of the plugging limit. The additional inspections require a complete tube inspection of an additional 3% and if required a third inspection of 6% of the tubes. The programs set forth in the ANO-1 Technical Specifications require a second inspection doubling the number of tubes inspected in the first sample. Again if more than 10% of the tubes show a detectable penetration greater than 20% or 1% are defective tubes, a third sample is required again doubling the number of tubes inspected in the second sample. In the first sample, sampling is to concentrate on areas of the tube array where prior inspections or experience have indicated potential problems, and full length traverse of each inspected tube is required. For a second or third sample, if required, the inspection may concentrate on areas of the tube array and portions of the tube in which the first sample or the second sample indicated potential problems.

Based on the considerations discussed above, we have concluded that the sample selection scheme is acceptable.

(b) Examination Method

The proposed examination methods include nondestructive examination by eddy current testing. The specified methods are capable of locating and identifying stress corrosion cracks and tube wall thinning from chemical wastage, mechanical damage or other causes. Based on our review of these methods, and experience gained using these methods by the industry, we have concluded that the examination methods are acceptable.

(c) Inspection Intervals

The proposed inspection intervals are compatible with those recommended in Regulatory Guide 1.83, with the exception that AP&L has proposed a minimum of 10 months between inspections instead of the 12 months recommended by the Regulatory Guide. We consider this proposal acceptable because it more accurately reflects the refueling shutdown interval for ANO-1 and will prevent unnecessary shutdown solely for steam generator tube inspection.

(d) Acceptance Criteria

The principal parameter used to determine whether any one steam generator tube is acceptable for continued service is the measured imperfection depth. In order to specify what level of imperfection is acceptable, a tube "plugging limit" is established. The "plugging limit" is defined in the Technical Specifications as the imperfection depth beyond which the tube must be removed from service, because the tube may become defective prior to the next scheduled inspection. For ANO-1 the "plugging limit" is 40% of the nominal tube wall thickness.

AP&L and the NRC staff have mutually agreed upon this 40% plugging limit in the definitions section of the Technical Specification. Although no B&W steam generator tube has to the present time exhibited any wastage corrosion, this plugging limit will provide, in the staff's opinion, conservative protection against wastage corrosion tube degradation. The licensee will be required in the future to recalculate the plugging limit using the recommendations of Regulatory Guide 1.121.^{1/}

Based on our review, the acceptance criteria are satisfactory.

(e) Reporting Requirements

Regulatory Guide 1.83, Revision 1, requires a licensee to report to the Commission and to await resolution and approval of the proposed remedial action when the inspection results exceed the limits specified in the Guide. It also states that additional sampling and more frequent inspection may be required. In the proposed Technical Specifications, it is clearly stated what additional inspection AP&L must perform without reporting to the NRC and requires (1) a prompt report on the number of tubes plugged in each steam generator following the steam generator tube inspection, (2) a complete report on the inspection in the next annual operating report, and (3) in the most severe cases described in the Technical Specifications, prompt notification of the NRC must be made together with a written followup.

^{1/} Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes," August, 1976.

It is our position that the reporting requirements are reasonable and will facilitate reporting of pertinent information without unnecessarily increasing plant downtime. Therefore, they are acceptable.

In summary, we have concluded that the proposed steam generator tube inservice inspection program will provide added assurance of the continued integrity of the steam generator tubes, and thus is acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 27, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-313

ARKANSAS POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 24 to Facility Operating License No. DPR-51, issued to Arkansas Power & Light Company (the licensee), which revised Technical Specifications for operation of Arkansas Nuclear One - Unit No. 1 (the facility) located in Pope County, Arkansas. The amendment is effective as of its date of issuance.

This amendment authorized changes in the Technical Specifications to add provisions for steam generator tube inspections consistent with the guidance of Regulatory Guide 1.83, Revision 1 (July 1975).

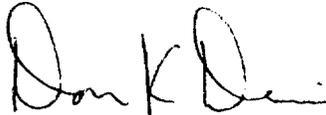
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 22, 1976, (2) Amendment No. 24 to Facility Operating License No. DPR-51 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Arkansas Polytechnic College, Russellville, Arkansas 72801. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 27th day of May, 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors