Mr. Guy G. Campbell, Vice President - Nuclear
FirstEnergy Nuclear Operating Company
5501 North State Route 2
Oak Harbor, OH 43449-9760

May 19, 1999

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - ISSUANCE OF AMENDMENT RE: TECHNICAL SPECIFICATIONS, SECTION 6, ADMINISTRATIVE REQUIREMENTS (TAC NO.MA3961)

Dear Mr. Campbell:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 231 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit 1. The amendment revises the Technical Specifications (TS) in response to your application dated October 28, 1998 (Serial Number 2559), as modified by your letter of March 19, 1999 (Serial Number 2588).

This amendment revises administrative requirements relating to: TS 6.5.1.6, Station Review Board Responsibilities; TS 6.8.4.d, Radioactive Effluent Controls Program; TS 6.10, Records Retention; TS 6.11, Radiation Protection Program; TS 6.12, High Radiation Area; and TS 6.15, Offsite Dose Calculation Manual.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:

William O. Long, Sr. Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. Amendment No.231 to License No. NPF-3
2. Safety Evaluation

cc w/encls: See next page

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NAME	WLong		EBarnhill				SHom*		AMendiola	
DATE	5/18/99		5/18/99		/ /99		05/12/99		5/19/99	

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DATE	5/18/99		5/18/99		1 /99		05/12/99		5/19/99	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 19, 1999

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FirstEnergy Nuclear Operating Company
5501 North State Route 2
Oak Harbor, OH 43449-9760

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William O. Long

William O. Long, Sr. Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. Amendment No. 231 to License No. NPF-3
2. Safety Evaluation

cc w/encls: See next page

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180 East Broad Street
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Commissioners of Ottawa County
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 231
License No. NPF-3

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the FirstEnergy Nuclear Operating Company (the licensee) dated October 28, 1998, as modified by letter dated March 19, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

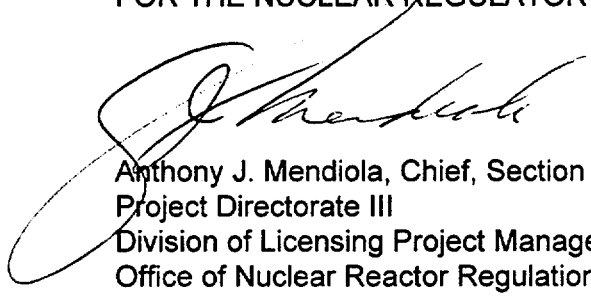
   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 231, are hereby incorporated in the license. FirstEnergy Nuclear Operating Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 120 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 19, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 231

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
XVI	XVI
6-7	6-7
6-14	6-14
6-14a	6-14a
6-19	6-19
6-20	6-20
	6-20a
	6-20b
6-22	6-22

d. Review of proposed tests or experiments determined to involve an unreviewed safety question as defined in 10 CFR 50.59.

e. Review of reports of violations of codes, regulations, orders, Technical Specifications, or Operating License requirements having nuclear safety significance or reports of abnormal degradation of systems designed to contain radioactive material.

f. Review of all proposed changes to the Technical Specifications or the Operating License.

g. Deleted

h. Review of reports of significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect plant safety.

i. Review of changes to the Industrial Security Plan, the Security Training and Qualification Plan, and the Security Contingency Plan.

j. Review of changes to the Davis-Besse Emergency Plan.

k. Review of items which may constitute potential nuclear safety hazards as identified during review of facility operations.

l. Investigations or analyses of special subjects as requested by the Company Nuclear Review Board.

m. Review of all REPORTABLE EVENTS.

n. Review of all Safety Limit Violation Reports and Protective Limit Violation Reports (Section 6.7).

o. Review of any unplanned, accidental or uncontrolled radioactive releases, evaluation of the event, ensurance that remedial action is identified to prevent recurrence, review of a report covering the evaluation and forwarding of the report to the Plant Manager and to the CNRB.

p. Review of the changes to the OFFSITE DOSE CALCULATION MANUAL.

q. Review of the changes to the PROCESS CONTROL PROGRAM.

r. Review of the Annual Radiological Environmental Operating Report.

s. Review of the Radioactive Effluent Release Report.

t. Review of changes to the Fire Protection Program.

6.8.4 (Continued)

c. Post-Accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

(i) Training of personnel,

(ii) Procedures for sampling and analysis,

(iii) Provisions for maintenance of sampling and analysis equipment.

d. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM.

2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 CFR Part 20, Appendix B, Table II, Column 2,

3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM.

4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of radiation exposure for all individuals entering radiation control areas.

d. Records of gaseous and liquid radioactive material released to the environs.

e. Records of transient or operational cycles for those facility components identified in USAR Table 5.1-8.

f. Records of reactor tests and experiments.

g. Records of training and qualification for current members of the plant staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the SRB and the CNRB.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.

m. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analyses at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.

o. Records of the service lives of all safety related hydraulic and mechanical snubbers including the date at which the service life commences and associated installation and maintenance records.

p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.11 Deleted

6.12 HIGH RADIATION AREA

As provided in paragraph 20.1601(c) of 10 CFR Part 20, the following controls shall be applied to high radiation areas in place of the controls required by paragraph 20.1601(a) and (b) of 10 CFR Part 20:

6.12.1 High radiation areas with dose rates not exceeding 1.0 rem/hour at 30 centimeters from the radiation source or from any surface penetrated by the radiation:

a. Each entry way to such an area shall be barricaded and conspicuously posted as a high radiation area. Such barricades may be opened as necessary to permit entry or exit of personnel or equipment.

b. Access to, and activities in, each such area shall be controlled by means of a Radiation Work Permit (RWP) or equivalent that includes specification of radiation dose rates in the immediate work area(s) and other appropriate radiation protection equipment and measures.

c. Individuals qualified in radiation protection procedures (e.g., health physics personnel) and personnel continuously escorted by such individuals may be exempted from the requirement for a RWP or equivalent while performing their assigned duties provided that they are following plant radiation protection procedures for entry to, exit from, and work in such areas.

d. Each individual (whether alone or in a group) entering such an area shall possess:

   1. A radiation monitoring device that continuously displays radiation dose rates in the area; or

   2. A radiation monitoring device that continuously integrates the radiation dose rates in the area and alarms when the device's dose alarm setpoint is reached, with an appropriate alarm setpoint, or

   3. A radiation monitoring device that continuously transmits dose rate and cumulative dose information to a remote receiver monitored by radiation protection personnel responsible for controlling personnel radiation exposure within the area, or

   4. A self-reading dosimeter (e.g., pocket ionization chamber or electronic dosimeter) and be under the surveillance, as specified in the RWP or equivalent, while in the area, by means of closed circuit television, by personnel qualified in radiation protection procedures responsible for controlling personnel radiation exposure in the area.

6.12.1 (continued)

e. Except for individuals qualified in radiation protection procedures, entry into such areas shall be made only after dose rates in the area have been determined and entry personnel are knowledgeable of them.

6.12.2 Locked high radiation areas with dose rates greater than 1.0 rem/hour at 30 centimeters from the radiation source or from any surface penetrated by the radiation, but less than 500 rads/hour at 1 meter from the radiation source or from any surface penetrated by the radiation:

a. Each entryway to such an area shall be conspicuously posted as a high radiation area and shall be provided with a locked door, gate, or other barrier that prevents unauthorized entry, and, in addition:

1. All keys to such doors, gates, or other barriers shall be maintained under the administrative control of the shift supervisor, radiation protection manager, or his or her designee.

2. Doors, gates, or other barriers shall remain locked except during periods of personnel or equipment entry or exit.

b. Access to, and activities in, each such area shall be controlled by means of an RWP or equivalent that includes specification of radiation dose rates in the immediate work area(s) and other appropriate radiation protection equipment and measures.

c. Individuals qualified in radiation protection procedures may be exempted from the requirement for a RWP or equivalent while performing radiation surveys in such areas provided that they are following plant radiation protection procedures for entry to, exit from, and work in such areas.

d. Each individual (whether alone or in a group) entering such an area shall possess:

1. A radiation monitoring device that continuously integrates the radiation dose rates in the area and alarms when the device's dose alarm setpoint is reached, with an appropriate alarm setpoint, or

2. A radiation monitoring device that continuously transmits dose rate and cumulative dose information to a remote receiver monitored by radiation protection personnel responsible for controlling personnel radiation exposure within the area with the means to communicate with and control every individual in the area, or

3. A self-reading dosimeter (e.g., pocket ionization chamber or electronic dosimeter) and,

(i) Be under the surveillance, as specified in the RWP or equivalent, while in the area, by an individual qualified in radiation protection procedures, equipped with a radiation monitoring device that continuously displays radiation dose rates in the area; who is responsible for controlling personnel exposure within the area, or

6.12.2 (continued)

(ii) Be under the surveillance as specified in the RWP or equivalent, while in the area, by means of closed circuit television, by personnel qualified in radiation protection procedures responsible for controlling personnel radiation exposure in the area and with the means to communicate with and control every individual in the area, or

4. In those cases where options (2) and (3), above, are impractical or determined to be inconsistent with the "As Low As is Reasonably Achievable" principle, a radiation monitoring device that continuously displays radiation dose rates in the area.

e. Except for an individual qualified in radiation protection procedures, entry into such areas shall be made only after dose rates in the area have been determined and entry personnel are knowledgeable of them.

f. Such individual areas that are within a larger area that is controlled as a high radiation area, where no enclosure exists for the purpose of locking and where no enclosure can reasonably be constructed around the individual area, need not be controlled by a locked door or gate, but shall be barricaded and conspicuous, and a clearly visible flashing light shall be activated at the area as a warning device.

6.14 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.p. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s), and

2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

b. Shall become effective after review and acceptance by the SRB and the approval of the Plant Manager.

6.15 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.p. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s), and

2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose or setpoint calculations.

b. Shall become effective after review and acceptance by the SRB and the approval of the Plant Manager.

c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 231 TO FACILITY OPERATING LICENSE NO. NPF-3

FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

1.0 INTRODUCTION

By letters dated October 28, 1998, and March 19, 1999, Davis-Besse Nuclear Power Station, Unit 1 (the licensee), submitted proposed changes to the facility Technical Specifications (TS). The March 19, 1999, changes to the application did not modify the application beyond the scope of the description of proposed changes provided in the Federal Register notice identified in Section 4.0 below, or alter the proposed no significant hazards consideration determination.

The proposed changes include:

A. Revising the Station Review Board (SRB) responsibilities in TS 6.5.1.6 regarding the review of changes to plans and programs.

B. Updating and correcting various references in TS 6.8.4.d.2, TS 6.8.4.d.3, TS 6.8.4.d.7, TS 6.10.2.e, and TS 6.15.a.2.

C. Relocating TS 6.11 "Radiation Protection Program" to the Updated Safety Analysis Report.

D. Revising TS 6.12, "High Radiation Area," to reference the current revision to 10 CFR Part 20, Subpart G, "Control of Exposure from External Sources in Restricted Areas," and specify alternative methods for controlling access to high radiation areas and monitoring personnel exposure.

2.0 DISCUSSION AND EVALUATION

This amendment revises administrative requirements relating to: TS 6.5.1.6, Station Review Board Responsibilities; TS 6.8.4.d, Radioactive Effluent Controls Program; TS 6.10, Records Retention; TS 6.11, Radiation Protection Program; TS 6.12, High Radiation Area; and TS 6.15, Offsite Dose Calculation Manual. A discussion and evaluation of each of the proposed TS changes follows:

2.1 TS 6.5.1.6, Station Review Board Responsibilities

TS 6.5.1.6 specifies, in part, that the SRB is responsible for review of certain program plans and changes to those plans. These plans and changes include the Industrial Security Plan, Security Training and Qualification Plan, Security Contingency Plan, Emergency Plan, and Fire Protection Program. The licensee has proposed that the SRB be responsible only for changes to the plans and the programs. This TS revision is being proposed in order to clearly delineate the extent of the SRB's review of the aforementioned programs and plans, and to more efficiently utilize the SRB's review time. The licensee's basis for acceptability is that each of these plans and programs is audited by other individuals and organizations, as required by other TS and regulations, at least every 12 months, and thus the SRB's review efforts are best focused on changes to the plans and programs.

The staff finds this change acceptable on the basis that the necessary periodic review function for these plans and programs is adequately implemented by the auditing activities identified in the application.

2.2 TS 6.8.4.d, Radioactive Effluent Controls Program

TS 6.8.4.d identifies certain programs that must be established, implemented, and maintained. It includes references to "10 CFR Part 20, Appendix B, Table II." The licensee initially requested that references to "10 CFR Part 20, Appendix B, Table II" be changed to "10 CFR Part 20, Appendix B, Table 2." The licensee also requested that "10 CFR 20.106" be updated to "10 CFR 20.1302." The application stated that these are editorial corrections to reflect the revised regulations.

In the letter dated March 19, 1999, the licensee withdrew the request to change "10 CFR Part 20, Appendix B, Table II" to "10 CFR Part 20, Appendix B, Table 2", because the concentrations in Table 2 are too low and are therefore impractical to base effluent monitor setpoint calculations for many liquid effluent release situations when monitor background, monitor sensitivity, and monitor performance must be taken into account. The use of Table II concentration values continues to be an acceptable method to maintain operational flexibility and ensure regulatory compliance with the dose objectives contained in Appendix I to 10 CFR Part 50 concurrent with the implementation of the revised 10 CFR Part 20.

2.3 TS 6.10, Records Retention

TS 6.10.2.e requires that "Records of transient of operational cycles for those facility components identified in Table 5.7-1" be retained for the duration of the operating licensee. Table 5.7-1 was relocated from the TS to the USAR by Amendment No. 204 dated December 8, 1995. The licensee proposes to correct the requirement so as to apply to "Records of transient or operational cycles for those facility components identified in the USAR Table 5.1-8.

The replacement of "of" with "or" is editorial and of no safety significance. The Table reference is consistent with previously approved License Amendment No. 204 which approved the relocation of the content of TS Table 5.7-1 to USAR Table 5.1-8. Therefore, the staff finds the 6.10 changes acceptable.

2.4 TS 6.11 Radiation Protection Program

TS 6.11 states:

> Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

The licensee requested deletion of TS 6.11 on the basis of its redundancy to current USAR Section 12.3.1 "Health Physics-Program Objectives," and 10 CFR Part 20 requirements. The licensee noted that the deletion would simplify the TS and that its removal would be consistent with the staff guidance of NUREG-1430 (Standard Technical Specifications).

The proposed removal of TS 6.11 is consistent with NUREG 1430, Revision 1, "Improved Standard Technical Specifications for Babcock and Wilcox Plants." The TS requirements to be deleted from the TS are adequately implemented through regulations and licensee controlled documents. Therefore, the staff finds the proposed change to be acceptable.

2.5 TS 6.12 High Radiation Areas

TS 6.12.1 identifies radiation protection requirements applicable to those high radiation areas that do not require locked doors. The licensee proposes the following changes to TS 6.12.1:

(1) change a reference to "10 CFR Section 20.203(c)(2)" to "10 CFR Section 20.1601(a), (b) and (c)";
(2) change the threshold dose rate condition that defines those high radiation areas for which TS 6.12.1 applies, from "less than or equal to 1000 mrem/hr" to "not exceeding 1.0 rem/hr at 30 centimeters";
(3) allow barricades for high radiation areas to be opened as necessary to permit entry or exit of personnel or equipment;
(4) add a requirement that Radiation Work Permits (RWPs) identify radiation dose rates in the immediate work areas and other appropriate radiation equipment and measures;
(5) provide RWP exemption of personnel continuously escorted by individuals qualified in radiation protection procedures (e.g., by health physics personnel);
(6) allow (as an additional alternative to presently specified person or group monitoring devices), the use of a radiation monitoring device that transmits dose rate and cumulative dose to a remote receiver monitored by radiation protection personnel; and
(7) allow for use of only a self-reading dosimeter by an individual who is also under the surveillance of qualified radiation protection personnel by means of closed circuit television.

TS 6.12.2 identifies additional radiation protection requirements applicable to certain high radiation areas that require locked doors. The licensee proposes the following changes to TS 6.12.2:

(1) change the threshold dose rate condition that defines applicability of TS 6.12.2, from "greater than 1000 mrem/hr" to "greater that 1.0 rem/hr at 30 centimeters, but less than 500 rads/hr at 1 meter";

(2) allow the use of a locked gate or other locked barrier as alternatives to locked doors for preventing unauthorized entry;
(3) add a requirement that Radiation Work Permits (RWPs) identify radiation dose rates in the immediate work areas and other appropriate radiation equipment and measures;
(4) allow for entry using a radiation monitoring device transmitting dose rate and cumulative dose to a remote receiver monitored by radiation protection personnel, as long as there is a means to communicate with and control every individual in the area;
(5) allow for entry using a self-reading dosimeter while being under the surveillance of an individual qualified in radiation protection procedures that is equipped with a radiation monitoring device that continuously displays radiation dose rates in the area, or while being under surveillance by means of a closed circuit television by qualified radiation protection personnel equipped with the means to communicate and control every individual in the area; and
(6) allow for use of a conspicuous barricade and clearly visible flashing light as a warning device for a high radiation area that is within a larger area where no enclosure exists for the purpose of locking and where no enclosure can reasonably be constructed.

The Technical Specification changes proposed above are consistent with Regulatory Guide 8.38; therefore, the staff finds them acceptable.

2.6 TS 6.15 Offsite Dose Calculation Manual

The licensee has proposed to change the reference "10 CFR 20.106" to "10 CFR 20.1302." The change is editorial in nature and does not alter any substantive requirements. Thus, the staff finds it acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a surveillance requirement, or changes recordkeeping, reporting, or administrative procedures or requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 64126). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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