

January 17, 1989

Docket No. 50-346  
Serial No. DB-89-002

Mr. Donald C. Shelton  
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Dear Mr. Shelton:

SUBJECT: CORRECTION TO AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE  
NO. NPF-3 (TAC NO. 66727)

On December 28, 1988, the Commission issued Amendment No. 128 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The amendment consisted of changes to the Technical Specifications (TS) in response to your application dated February 1, 1988 (Serial No. 1464) as supplemented with information contained in a letter dated February 26, 1988 (Serial No. 1496).

It has been brought to our attention that two TS pages issued with the amendment did not incorporate previous changes issued by Amendment No. 123. Enclosed find corrected TS pages 2-2 and 2-5 with the corresponding overleaf pages.

Sincerely,

/s/

Thomas V. Wambach, Sr. Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III,  
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Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
See next page

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TWambach/mr  
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PDR ADOCK 05000346  
P PNU

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1/1

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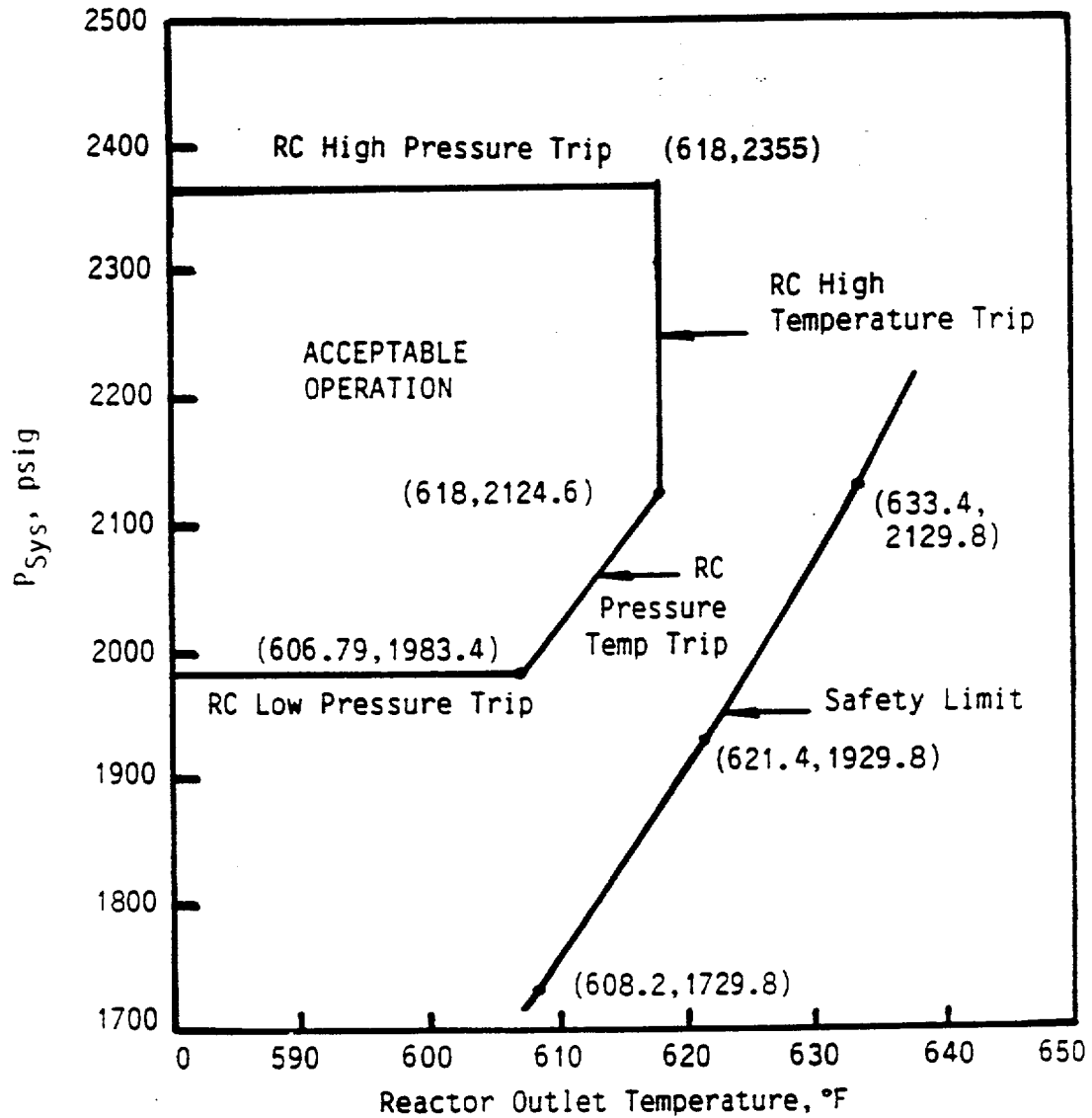
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Figure 2.1-1 Reactor Core Safety Limit



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## 2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

### 2.1 SAFETY LIMITS

#### REACTOR CORE

2.1.1 The combination of the reactor coolant core outlet pressure and outlet temperature shall not exceed the safety limit shown in Figure 2.1-1.

APPLICABILITY: MODES 1 and 2.

#### ACTION:

Whenever the point defined by the combination of reactor coolant core outlet pressure and outlet temperature has exceeded the safety limit, be in HOT STANDBY within one hour.

#### REACTOR CORE

2.1.2 The combination of reactor THERMAL POWER and AXIAL POWER IMBALANCE shall not exceed the safety limit shown in Figure 2.1-2 for the various combinations of two, three and four reactor coolant pump operation.

APPLICABILITY: MODE 1.

#### ACTION:

Whenever the point defined by the combination of Reactor Coolant System flow, AXIAL POWER IMBALANCE and THERMAL POWER has exceeded the appropriate safety limit, be in HOT STANDBY within one hour.

#### REACTOR COOLANT SYSTEM PRESSURE

2.1.3 The Reactor Coolant System pressure shall not exceed 2750 psig.

APPLICABILITY: MODES 1, 2, 3, 4 and 5.

#### ACTION:

- |                       |   |
|-----------------------|---|
| MODES 1 and 2 -       | Whenever the Reactor Coolant System pressure has exceeded 2750 psig, be in HOT STANDBY with the Reactor Coolant System pressure within its limit within one hour. |
| MODES 3, 4<br>and 5 - | Whenever the Reactor Coolant System pressure has exceeded 2750 psig, reduce the Reactor Coolant System pressure to within its limit within 5 minutes.             |

Table 2.2-1 Reactor Protection System Instrumentation Trip Setpoints

Functional unit	Trip setpoint	Allowable values
1. Manual reactor trip	Not applicable.	Not applicable.
2. High flux	$<104.94\%$ of RATED THERMAL POWER with four pumps operating  $<80.6\%$ of RATED THERMAL POWER with three pumps operating	$<104.94\%$ of RATED THERMAL POWER with four pumps operating#  $<80.6\%$ of RATED THERMAL POWER with three pumps operating#
3. RC high temperature	$\leq 618^{\circ}\text{F}$	$\leq 618^{\circ}\text{F}\#$
4. Flux -- $\Delta\text{flux}/\text{flow}^{(1)}$	Four pump trip setpoint not to exceed the limit line of Figure 2.2-1. For three pump operation, see Figure 2.2-1	Four pump allowable values not to exceed the limit line of Figure 2.2-1#. For three pump operation, see Figure 2.2-1
5. RC low pressure <sup>(1)</sup>	$\geq 1983.4$ psig	$\geq 1983.4$ psig* $\geq 1983.4$ psig**
6. RC high pressure	$\leq 2355$ psig	$\leq 2355.0$ psig* $\leq 2355.0$ psig**
7. RC pressure-temperature <sup>(1)</sup>	$\geq (12.60 T_{\text{out}}^{\circ}\text{F} - 5662.2)$ psig	$\geq (12.60 T_{\text{out}}^{\circ}\text{F} - 5662.2)$ psig#
8. High flux/number of RC pumps on <sup>(1)</sup>	$\leq 55.1\%$ of RATED THERMAL POWER with one pump operating in each loop  $\leq 0.0\%$ of RATED THERMAL POWER with two pumps operating in one loop and no pumps operating in the other loop  $\leq 0.0\%$ of RATED THERMAL POWER with no pumps operating or only one pump operating	$\leq 55.1\%$ of RATED THERMAL POWER with one pump operating in each loop#  $\leq 0.0\%$ of RATED THERMAL POWER with two pumps operating in one loop and no pumps operating in the other loop#  $\leq 0.0\%$ of RATED THERMAL POWER with no pumps operating or only one pump op- erating#
9. Containment pressure high	$\leq 4$ psig	$\leq 4$ psig#

Table 2.2-1. (Cont'd)

- (1) Trip may be manually bypassed when RCS pressure  $\leq 1820$  psig by actuating shutdown bypass provided that:
- a. The high flux trip setpoint is  $\leq 5\%$  of RATED THERMAL POWER.
  - b. The shutdown bypass high pressure trip setpoint of  $\leq 1820$  psig is imposed.
  - c. The shutdown bypass is removed when RCS pressure  $> 1820$  psig.

\*Allowable value for CHANNEL FUNCTIONAL TEST.

\*\*Allowable value for CHANNEL CALIBRATION.

#Allowable value for CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION.