

April 25, 1989

Docket No. 50-346
Serial No. DB-89-005

DISTRIBUTION

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Dear Mr. Shelton:

SUBJECT: AMENDMENT NO.132 TO FACILITY OPERATING LICENSE NO. NPF-3
CHANGE IN LIMITING CONDITION FOR MAIN STEAM SAFETY VALVES
(TAC NO. 60103)

The Commission has issued Amendment No. 132 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TS's) in response to your application dated June 17, 1985 (No. 1159), and letters dated November 22, 1985 (No. 1213), and March 20, 1986 (No. 1159).

The amendment revises Technical Specification 3.7.1.1 concerning the Limiting Condition for Operation for the main steam line safety valves. The change will require the plant to go to Mode 4 (hot shutdown) during valve inoperability, rather than to Mode 5 (cold shutdown) within 12 hours following entry to Mode 3 (hot standby). As requested in your letter dated August 29, 1988, this change has been processed for Davis-Besse as the lead plant of the B&W Owners Group.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Thomas V. Wambach, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV,
V & Special Projects
Office of Nuclear Reactor Regulation

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- Enclosures:
1. Amendment No.132to License No. NPF-3
 2. Safety Evaluation

cc: See next page

*SEE PREVIOUS CONCURRENCE

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Surname:	*PKreutzer	ADeAgazio	TWambach	*EButcher	*JCraig	JHannon
Date:	05/04/88	05/04/88	03/29/89	03/16/89	03/17/89	4/25/88 89
Office:	OGC-WF1					
Surname:						
Date:	3/27/89					

JVM
OGC files with changes in SER + notice 4/17/89
DF01
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Mr. Donald C. Shelton
Toledo Edison Company

Davis-Besse Nuclear Power Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 132
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated June 17, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

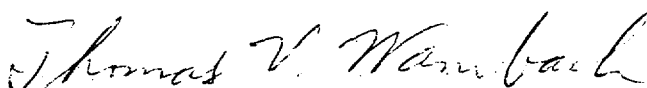
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(a) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 132, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than June 9, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

for 

John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III, IV,
V, & Special Projects
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 25, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove

3/4 7-1

Insert

3/4 7-1

3/4.7 PLANT SYSTEMS

3/4.7.1 TURBINE CYCLE

SAFETY VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.1 All main steam line code safety valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

With one or more main steam line code safety valves inoperable,

- a. operation in MODES 1, 2 and 3 may proceed provided, that within 4 hours, either,
 1. the inoperable valve is restored to OPERABLE status, or,
 2.
 - a) the High Flux Trip Setpoint is reduced per Table 3.7-1, and,
 - b) there are a minimum of two OPERABLE safety valves per steam generator, at least one with a setpoint not greater than 1050 psig (+/-1%)*, and,
 - c) no OPERABLE safety valve has a setpoint greater than 1100 psig (+/-1%)*;
- otherwise,
- b. be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 12 hours.
 - c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.1 No additional Surveillance Requirements other than those required by Specification 4.0.5, are applicable for the main steam line code safety valves of Table 4.7-1.

* The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

TABLE 3.7-1

MAXIMUM ALLOWABLE HIGH FLUX TRIP SETPOINT WITH INOPERABLE
STEAM LINE SAFETY VALVES

<u>Maximum Number of Inoperable Safety Valves on Any Steam Generator</u>	<u>Maximum Allowable High Flux Trip Setpoint* (Percent of RATED THERMAL POWER)</u>
1	0.95W
2	0.82W
3	0.69W
4	0.56W
5	0.43W
6	0.31W
7	0.18W

*Based on High Flux Trip Setpoint for four pump operation, W, as per Table 2.2-1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 132 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

1.0 INTRODUCTION

By letter dated June 17, 1985 (No. 1159), the Toledo Edison Company (TE) proposed an amendment to Appendix A Technical Specification (TS) 3.7.1.1. The proposed change would revise the TS Action Statement concerning the Limiting Condition for Operation for the main steam line safety valves. The revision would require the plant to go to Mode 4 (hot shutdown) rather than Mode 5 (cold shutdown) during valve inoperability within 12 hours following entry to Mode 3 (hot standby). TE has proposed this change to eliminate an inconsistency between the Applicability requirement and the Action Statement. In response to the TE request in a letter dated August 29, 1988 (Serial No. 1570), this action has been processed as the lead plant for the B&W Owners Group.

2.0 DISCUSSION

The Main steam overpressure protection system is designed with nine main steam safety valves on each of the two steam (generator) loops. The main steam safety valves are needed to relieve excess steam in the event of various transients to ensure overpressure protection for the plant's secondary side. Pressure relief is required at 1050 psig system design pressure.

The TS 3.7.1.1 Action Statement requires that one or more main steam safety valves inoperable operation in Modes 1, 2, and 3 may proceed if the inoperable valve is restored to operable status or the High Flux Trip Setpoint is reduced (power level reduced) per Table 3.7-1 within 4 hours. Otherwise, the existing TS states that the unit be placed in Mode 3 (hot standby) within the next 6 hours and in Mode 5 (cold shutdown) within the following 30 hours. The proposed revision would, instead, require the plant to enter Mode 4 (hot shutdown) within 12 hours following entry to Mode 3 (hot standby).

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T.S. 3.7.1.1 is applicable only in Modes 1, 2, and 3, since the main steam safety valves are required to provide overpressure protection in these modes. Other operating modes (4 and 5) do not exceed a reactor coolant system temperature of 280°F, which corresponds to a saturation pressure of 49.2 psia, substantially below the main steam system design pressure. Therefore, when main steam safety valves are inoperable, operation in Mode 4 is safe since overpressure protection is not required, and it is unnecessary to go to Mode 5. If the operating temperature inadvertently would exceed 280°F in Mode 4, the system would still be protected because of the action of the Reactor Coolant System low temperature overpressure protection features. The staff issued a safety evaluation of the low temperature overpressure protection feature on July 25, 1980. The action of these features would limit pressure to 438 psig.

The proposed 12-hour time requirement to reach Mode 4 is consistent with other TS's that require the plant to go to Mode 4. There is not accident initiating from Mode 4 that would require the operation of the main steam safety valves, and the safety function of TS 3.7.1.1 or the plant is not being degraded by this change. Therefore, the staff finds the proposed changes acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lynn Kelly

Dated: April 25, 1989